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TEDESCO,R.L. Assistant Director for Licensing

SUBJECT: Forwards response to NRC 810306 request for addl info re Q-list,Reactor cavity sump pumps & containment structure sump pumps will be reclassifed Lo Class IIA.

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	EMERG PREP	22	1	0	EQUIP QUAL BR	13	3	3
	GEOSCIENCES	14	1	1	HUM FACT ENG BR		1	1
	HYD/GEO BR	15	2	2	I&C SYS BR	16	1	1
	I&E	06	3	3	LIC GUID BR		1	1
	LIC QUAL BR		1	1	MATL ENG BR	17	1	1
	MECH ENG BR	18	1	1	MPA		1	0
	NRC PDR	02	1	1	OELD		1	0
	OP LIC BR		1	1	POWER SYS BR	19	1	1
	PROC/TST REV	20	1	1	QA BR	21	1	1
	RAD ASSESS BR	22	1	1	REAC SYS BR	23	1	1
	REG FILE	01	1	1	SIT ANAL BR	24	1	1
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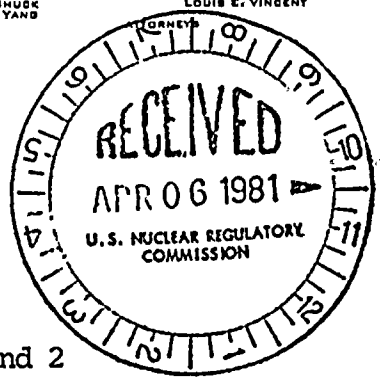
April 2, 1981



Mr. Robert L. Tedesco
Assistant Director for Licensing
Division of Licensing
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Re: Docket No. 50-275
Docket No. 50-323
Diablo Canyon Units 1 and 2

Subject: Diablo Canyon "Q" List

Dear Mr. Tedesco:

Attached is our response to question 260.17 forwarded with your letter dated March 6, 1981. We have previously addressed questions 260.11 - 260.16 in our March 18, 1981 letter to you.

In a future amendment we will modify the Diablo Canyon FSAR to include the information contained in our responses to these questions.

Kindly acknowledge receipt of the above material on the enclosed copy of this letter and return it to me in the enclosed addressed envelope.

Very truly yours,

Philip A. Crane, Jr.

Attachment
CC w/attachment: Service List

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260.17.(a) ADDITIONS

The following items will be added to FSAR Table 3.2-4 in an appropriate section. Those items or activities which do not fulfill the requirements of Category I as described in paragraph 3.2.1 of the FSAR are designated Design Class IIA.

Structures, systems or components designated Design Class IIA may not have been designed or constructed under a quality program meeting all requirements of Chapter 17. However, activities such as repair, replacement, maintenance or testing shall be performed under the operational quality assurance program. Quality requirements administered shall be commensurate with the safety function of the structure, system or component.

260-17 (a)

ITEM	DESCRIPTION	CLASSIFICATION
1	Safety related masonry walls	IIA
2	Breakwaters	III (See NOTE 1)
3	Reactor coolant boundary leak detection system	IIA
4	Missile barriers which protect safety-related items	IIA
5a	Electrical penetration of containment (non-vital)	I
	Backup fault current protective devices	IIA
5b	Raceway fire stops and seals	IIA
5c	Emergency lighting battery packs	I
10	Personnel radiation monitoring equipment including internal and external personnel monitoring	IIA
6,7,9	Radiation monitoring equipment including contamination measurement and analysis equipment (fixed and portable)	IIA
8	Post accident radioactivity sampling (air, surface, liquids)	IIA
12	Accident related decontamination of facilities, personnel, and equipment	IIA
13	Respiratory protection equipment	IIA
14	Accident related contamination control	IIA
15	Radiation shielding (permanently installed)	IIA
16	Meteorological data collection equipment	IIA
17	Expendable and consumable items necessary for the functional performance of safety-related structures, systems or components (i.e., weld rod, fuel oil, boric acid, snubber oil, etc.)	IIA
19	Ground slope east of building complex	IIA
20	Raw water reservoir	IIA
21	Hydrogen recombiner, including piping and valves	I
22	Containment pressure indication system	I
23	Containment water level indication systems	I
24	Containment hydrogen indication system	I
25	Valve operators for safety-related valves	I
26	Motors for safety-related pumps	I

The following items will not be added to Table 3.2-4

260-17 (a)

ITEM	DESCRIPTION
11(See Note 2)	Instrument Storage, Calibration and Maintenance
18(See Note 2)	Measuring and test equipment used for safety-related structures, systems, and components

Note 1 See our letter to Frank J. Miraglia, Jr., dated March 27, 1981 regarding "Safety Implication of Damaged Breakwaters" for justification of Design Classification III.

Note 2 These activities and equipment are already controlled by our established quality program. The existing procedures provide the degree of quality assurance necessary for storing, calibrating and testing this type of equipment and conducting these activities. This is established under Procedure 8.1 "Control of Measuring and Test Equipment" of the Quality Assurance Manual. The manual is established in accordance with the requirements of 10CFR50, Appendix B.

260.17.b CLARIFICATIONS

The following revision will be made to the FSAR Table 3.2-4. If no change is indicated, the justification is provided.

260.17 (b)
ITEM

1 Note one on sheet 4 will be revised as follows: "The Turbine Generator Building was designed and erected as a Design Class II structure except for those areas housing safety-related equipment. The areas containing safety-related equipment were designed to Design Class I criteria but without Quality Assurance documentation. The complete structure has been modified to meet Design Class I criteria. A thorough Quality Assurance program was applied to the design and construction of the modifications."

2 The Design Class of the New Fuel Storage Racks will be changed to Class IIA. The following statement will be added to the footnote. "Any modifications or repairs will be conducted under the operational quality assurance program."

3 The Design Class of the Intake Structure will be changed from II to IIA. The footnote will be deleted.

4 The Containment Recirculation Sump, Sump Screen, and Vortex Suppression will be added to sheet 2 as Design Class I.

5 The Reactor Cavity Sump Pumps and the Containment Structure Sump Pumps will be reclassified IIA as they are part of the Reactor Coolant Leakage Detection System.

6 Sheet 25 will be revised as follows: "Reactor Coolant System Valves - including Safety valves, PORV, PORV block valves and their actuators."

7 The last entry on sheet 23 will be revised as follows: "Valves for Above Portion of System - including safety relief valves and atmospheric dump valves".

8 Instrumentation and control systems described in sections 7.2 through 7.6 of the FSAR are Design Class I and subject to the pertinent requirements of the QA program described in Chapter 17 of the FSAR. Any systems or portions of systems described in sections 7.2 through 7.6 which are not designated as Design Class I on design documents will be designated Design Class IIA. Such exceptions to Design Class I designation will be noted in a future change to FSAR.

9 The Design Class of 250 VDC Motor Control Center SD 121 will be changed to IIA.

10 The Design Class of the Circulating Water Conduits will be changed from II to IIA.

260.17c. NUREG-0737 items.

The following items will be added to FSAR Table 3.2-4 in an appropriate section.

260-17 (c)

ITEM	DESCRIPTION	DESIGN CLASS	NUREG -0737 ITEM
2	Reactor coolant system vent	I	II.B.1
3	Plant shielding (see also 260.17.a item 15)	IIA	II.B.2
4	Post accident sampling equip.	IIA	II.B.3
5	Valve position indication	I	II.D.3
9	Dedicated hydrogen penetrations	I	II.E.4.1
11	Accident monitoring instrumentation		II.F.1
	Noble gas effluent radiological monitor	IIA	(1)
	Radioactive iodines and particulates effluent monitors	IIA	(2)
	Containment high range radiation monitors	I	(3)
	Containment pressure monitor	I	(4)
	Containment water level monitor	I	(5)
	Containment hydrogen concentration monitor	I	(6)
12	Instrumentation for detection of inadequate core cooling	IIA	II.F.2
17	Anticipating reactor trip on turbine trip	IIA	II.K.3(12)
20	Emergency support facilities:		III.A.1.2
	Technical support center	III (See Note 1)	
	Operations support center	III (See Note 1)	
	Emergency operations facility	III (See Note 1)	

The following items are considered to be included in the existing table.

ITEM	DESCRIPTION	TABLE 3.2-4 SHEET
6	Auxiliary feedwater (See Note 2)	8,23 and 29
7	Auxiliary feedwater system initiation and flow (See Note 3)	8,23 and 29
8	Emergency power for pressurizer heaters (See Note 4)	
13	Power supplies for pressurizer relief valves, block valves (See Note 5)	35, 36
22	Control room habitability (See Note 6)	

The following item(s) is considered to be included in the commitment made in our response to 260.17.a:

260-17 (c) ITEM	DESCRIPTION	260.17 (a) ITEM
21	Inplant iodine radiation monitoring	6

The following items have not been incorporated in Diablo Canyon. If and when they are, they will be added and operated under appropriate Quality Assurance programs.

ITEM	DESCRIPTION
1	Plant safety parameter display system
14	Automatic PORV isolation
15	Automatic trip of reactor coolant pump

The following items are inappropriate for inclusion in Table 3.2-4.

ITEM	DESCRIPTION	COMMENT
10	Containment isolation dependability	Dependability is a feature of the design and as such has no ongoing activity associated with it. Any modification to plant design will be covered by the operational quality assurance program.
16	PID controller	The derivitive action setting has been set to zero at Diablo Canyon. The changing of any such setting is covered by plant procedures which require that appropriate quality assurance procedures be followed.
18	Power on pump seals	This NUREG-0737 item requires that "Adequacy of the pump seals be demonstrated." Once demonstrated no further operational quality assurance activities are deemed necessary. Any changes to seal design would covered by the operational quality assurance program.
19	Emergency Plans	Procedures referenced or implemented by the Emergency Plan will be covered by the Operational Quality Assurance program.

Note 1. The appropriate requirements of the operational QA program will be applied to the safety-related features of these facilities and their assorted equipment to assure continued compliance with applicable regulatory requirements. These requirements are currently specified in NUREG-0696, "Functional Criteria for Emergency Program Facilities."

Note 2. The auxiliary feedwater system was designed and constructed as a Design Class I system as indicated in Table 3.2-4.

Note 3. Instrumentation and controls for the Auxiliary Feedwater System are included in section 7.4 of the FSAR and are Design Class I.

Note 4. Two pressurizer heater banks can be powered from the 480V vital distribution system. This electric system is designated Design Class I on Sheet 35 of Table 3.2-4.

Note 5. The pressurizer relief valves (PORV) are powered from the 125/250VDC - vital system listed on sheet 36. The block valves are powered from 480V System - Vital (sheet 35). Both these systems are Design Class I.

Note 6. A control room pressurization has been added at Diablo Canyon. The system is a Design Class I system and is integrated into the Control Room Air Conditioning System which is shown as Design Class I on sheet 4 of Table 3.2-4.