

MEETING SUMMARY DISTRIBUTION

LWR-1

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LWR 1 File

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Trip Participants

MAR 3 1980

275 ✓
Docket Nos. 50-285
and 50-323

APPLICANT: Pacific Gas and Electric Company
FACILITY: Diablo Canyon Nuclear Power Plant (DCNPP) Units 1 and 2
SUBJECT: SUMMARY OF DCNPP SITE VISIT AND REVIEW BY NRR SHORT-TERM
LESSONS LEARNED REVIEW TEAM

Meetings were held at the Diablo Canyon site to continue the review process involving implementation of the short-term lessons learned (STLL) requirements. The meetings were held on January 30 and 31, 1980 and attendance lists are enclosed.

The principal areas of review covered were as follows:

1. An extensive plant tour was conducted to permit the staff reviewers to see the STLL hardware, the locations where hardware is to be installed and plant areas involved in STLL emergency response requirements, e.g., onsite technical support center and control room.
2. Draft copies of Administrative Procedures required by STLL were reviewed and discussed with the applicant.
3. Each STLL item was discussed from the standpoint of review status and known information still required from the applicant. This discussion included the updated status information provided by the applicant on January 28, 1980. The required information included the following:

NUREG-0578 Section

2.1.1 Emergency Power Supply Requirements for the Pressurizer Heaters.

Relate the capacity of these heaters to the diesel-generator rating and provide procedure which governs loading of heaters on to diesels.

2.1.3.a Direct Indication of Power-Operated Relief Valve (PORV) and Safety Valve (S/V) Position.

Assure that procedures include the use of thermocouple readings downstream of the PORV's and SV's and the pressurizer relief tank pressure as backup indication of valve position.

Describe the calibration of the acoustic monitors including the means to account for feedback from common downstream piping.

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DATE

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2.1.3.b Instrumentation for Detection of Inadequate Core Cooling.

The subcooling meter installation proposed by PG&E appears acceptable.

The basis for the statement (p. 13 of the January 25, 1980 submittal) that the level indication system will provide unambiguous indication of inadequate core cooling should be provided.

2.1.4 Diverse Containment Isolation.

An explanation of the method to accomplish individual valve control should be provided.

2.1.5.a Dedicated Penetrations for External Recombiner or Post-Accident External Purge System.

2.1.5.c Capability to Install Hydrogen Recombiners at each LWR at each plant.

The PG&E proposal for redundant Westinghouse recombiners inside containment plus provisions for backup purging and connections to the purge system outside containment for a backup recombiner (if needed), appears acceptable. As part of their documentation, PG&E should add a schematic of the system, procedures regarding the criterion for placing the system in operation, and instructions on valve control.

2.1.6.a Integrity of Systems Outside Containment.

A summary of the results to date of the leak testing program should be provided.

2.1.8.a Improved Post-Accident Sampling Capability.

As a contingency measure in the event of a delay in the installation of the remote sampling system, an interim sampling procedure is needed.

2.1.8.b Increased Range of Radiation Monitors.

This item needs further development by PG&E for staff evaluation.

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2.1.9 Analysis of Design and Off-Normal Transients and Accidents.

The emergency procedure guidelines used by Westinghouse and based on revised analyses must be incorporated into plant emergency operating procedures. The staff must review these procedures. A drawing of the containment pressure indicator system should be provided. A sketch and description of the containment sump wide range level indicator should be provided.

2.2.1.a
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2.2.2.c Procedures and Support Centers.

The staff reviewed draft administrative procedures covering shift personnel responsibilities, shift manning, shift turnover and control room planning. In addition, the support centers were discussed with emphasis on communications and instrumentation to be available in the Technical Support Center. In general, the PG&E efforts to date in these areas appear to be in line with staff requirements. The staff will continue its review in these areas.

Original Signed by:

John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

Enclosures:
As stated.

cc: See next page

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MAR 3 1980

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ENCLOSURE

PG&E - NRC MEETING

January 30, 1980

at

Diablo Canyon Power Plant

PG&E

R. A. Young
R. F. Locke
Gary H. Moore
T. N. Crawford
R. Lavery
S. M. Skidmore
W. E. Coley
R. Patterson
J. Shiffer
M. Ramsay
John Hoch
John Gislson

NRC

H. E. P. Krug, Jr.
W. C. Milstead
P. Stoddart
J. J. Holman
J. F. Stolz
F. J. Williams, Jr.
T. Young, Jr.

Westinghouse

D. Payne

ENCLOSURE

PG&E - NRC MEETING

JANUARY 31, 1980

AT

DIABLO CANYON POWER PLANT

NRC

H. E. P. Krug, Jr.
J. J. Holman
P. G. Stoddart
F. Williams, Jr.
W. C. Milstead
J. Stolz

PG&E

R. F. Locke
W. E. Coley
P. Crawford
R. A. Young
G. Moore
M. Ramsay
R. Patterson
J. Shiffer
S. Skidmore
J. B. Hoch
R. M. Lavery
J. Gisclon

Westinghouse

D. Payne

MAR 18 1980

S. A. Varga

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This review does not address the lessons learned from Three Mile Island. These matters will be addressed in a future supplement to the SER.

Richard P. Denise, Acting Assistant Director
for Reactor Safety
Division of Systems Safety

Enclosure:
As stated

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J. Knox
W. Butler
W. Milshead
F. Eltawila
J. Zwolinski
J. Pulsipher

Central files
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NRR Rdg
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DATE	3/14/80	3/14/80	3/14/80	3/18/80	

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