



Cleve Reasoner
Site Vice President

March 9, 2017
WO 17-0009

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Subject: Docket No. 50-482: Revision 30 of the Wolf Creek Generating Station
Updated Safety Analysis Report

To Whom It May Concern:

Pursuant to the updating requirements of 10 CFR 50.71(e), Wolf Creek Nuclear Operating Corporation (WCNOC) is providing its Updated Safety Analysis Report (USAR), Revision 30. This submittal satisfies the Final Safety Analysis Report (FSAR) updating requirements of the aforementioned regulation. Per the requirement of 10 CFR 54.37(b), there are no newly identified systems, structures, and components that are subject to an aging management review or evaluation of time-limited aging analyses.

Attachment I to this letter provides information relative to changes in regulatory commitments. This information is provided in accordance with the guidance of Nuclear Energy Institute (NEI) 99-04, "Guidelines for Managing NRC Commitment Changes," Revision 0, July 1999.

Attachment II to this letter describes specific technical changes that have been processed since issuance of the USAR, Revision 29. In addition to these technical changes, editorial changes have been made and are included in Revision 30.

Attachment III to this letter provides a discussion of changes made in Revision 63 of the Technical Requirements Manual (TRM).

Enclosure I to this letter provides the CD-ROM submittal of the Wolf Creek Generating Station (WCGS) USAR, Revision 30. This submittal satisfies the Final Safety Analysis Report updating requirements of 10 CFR 50.71(e)(4).

Enclosure II to this letter provides a CD-ROM containing the station-controlled drawings that are considered incorporated by reference into the USAR. According to the guidance of NEI 98-03, Revision 1, "Guidelines for Updating Final Safety Analysis Reports," Revision 1, June 1999, the USAR figures that are identical to controlled drawings were relocated from the USAR in Revision 17. Enclosure II is considered sensitive unclassified information and therefore warrants withholding under 10 CFR 2.390.

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Enclosure III to this letter provides a CD-ROM containing the Quality Program Manual, USAR Fire Hazards Analysis, The USAR Fire Hazards Analysis is considered sensitive unclassified information and therefore warrants withholding under 10 CFR 2.390.

Enclosure IV to this letter provides a CD-ROM containing the EQSD-I, EQ Summary Document Section I Program Description, and EQSD-II, Equipment Qualification Summary Document Master List Section II, both of which are incorporated by reference into the USAR. EQSD-I and EQSD-II are considered sensitive unclassified information and therefore warrant withholding under 10 CFR 2.390.

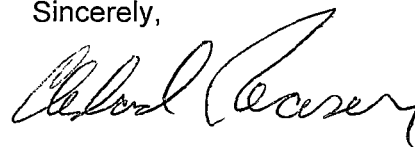
Information from USAR Table 3.11(B)-1, Plant Environmental Normal Conditions; USAR Table 3.11(B)-2, Environmental Qualification Parameters for SNUPPS NUREG-0588 (LOCA, MSLB and HELB); USAR Table 3.11(B)-3, Identification of Safety-Related Equipment and Components: Equipment Qualification; USAR Table 3.11(B)-4, Containment Worst Case Radiation Levels (MRADs); USAR Table 3.11(B)-5, Containment Spray Requirements; USAR Table 3.11(B)-8, Exemptions from NUREG-0588 Qualification; USAR Table 3.11(B)-10, Equipment Added for NUREG-0737; and USAR Figures 3.11(B)-1 through 3.11(B)-49, were relocated from the USAR into EQSD-I and EQSD-II in USAR Revision 28 and are incorporated by reference into the USAR.

Enclosure V to this letter provides those changes made to the WCGS Unit 1 TRM (Revision 63) and includes a List of Effective Pages. The WCGS TRM is incorporated by reference into the USAR.

This letter contains no commitments. WCNOG has historically submitted updates to the USAR on March 11 of each year to coincide with the date of issuance of the WCGS operating license and to comply with the requirements of 10 CFR 50.71(e)(4). WCNOG considers that submittals made prior to or on March 11 satisfy the requirements of 10 CFR 50.71(e)(4).

If you have any questions concerning this matter, please contact me at (620) 364-4171, or Cynthia R. Hafenstine at (620) 364-4204.

Sincerely,



Cleveland Reasoner

COR/rlt

Attachment I – Regulatory Commitment Management System (RCMS) Changes

Attachment II – USAR Changes Processed Since Revision 29

Attachment III – Revision 63 to the Technical Requirements Manual

- Enclosure I – CD-ROM containing Updated Safety Analysis Report, Revision 30
- Enclosure II – CD-ROM containing Updated Safety Analysis Report Controlled Figure Drawings
- Enclosure III – CD-ROM containing Quality Program Manual, Updated Safety Analysis Report Fire Hazards Analysis

- Enclosure IV – CD-ROM containing EQSD-I and EQSD-II
- Enclosure V – Technical Requirements Manual Replacement Pages and List of Effective Pages

cc: K. M. Kennedy (NRC), w/a, w/e
B. K. Singal (NRC), w/a, w/e
N. H. Taylor (NRC), w/a, w/e
Senior Resident Inspector (NRC), w/a, w/e

Attachment I to WO 17-0009

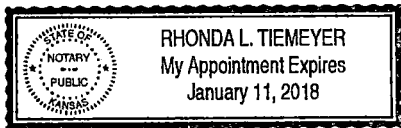
Regulatory Commitment Management System (RCMS) Changes

STATE OF KANSAS)
) SS
COUNTY OF COFFEY)

Cleveland O. Reasoner, of lawful age, being first duly sworn upon oath says that he is Site Vice President of Wolf Creek Nuclear Operating Corporation; that he has read the foregoing document and knows the contents thereof; that he has executed the same for and on behalf of said Corporation with full power and authority to do so; and that the facts therein stated are true and correct to the best of his knowledge, information and belief.

By *Cleveland Reasoner*
Cleveland Reasoner
Site Vice President

SUBSCRIBED and sworn to before me this 9th day of March, 2017.



Rhonda L. Tiemeyer
Notary Public

Expiration Date *January 11, 2018*

Commitment Changes

Commitment No. RCMS 1989-049 from WM 89-0071, dated March 1, 1989

Commitment Description: In the Regulatory Commitment Management System (RCMS), the commitment is stated as "A check of screws will be added to the I&C Preventative Maintenance Program prior to restart following Refueling Outage 4."

This commitment was initiated as a result of root cause analysis and LER 89-004-00, "Loose Terminal Connections Cause Main Steam Isolation Valve (MSIV) Closure Resulting in Reactor Trip." The cause of the reactor trip was determined to be a loose screw on a terminal strip in the solid state protection system (SSPS). During unrelated maintenance, this loose termination was bumped resulting in a fast close signal to MSIV C. The root cause was never identified. A corrective action included validating tightness of all SSPS terminal board connections, all vendor connections on the individual slave relays located in these cabinets, the Balance of Plant ESFAS cabinets, and majority of the Control Room cabinets.

Change to Commitment: The commitment was closed and archived.

Justification: This preventative maintenance (PM) was performed every 18 months for 13 years on the main control board cabinets. In 2003, the PM was extended to every 36 months based on a review of documentation from the initial 13 years of this activity which showed zero loose connections had been found. After 22 years of performing this preventative maintenance activity, no trend of loose wiring connections has been established. There is no identified mechanism that would cause connections, once tightened, to loosen. Stringent work control activities are in place to provide assurance that any maintenance activities affecting terminals will also ensure that the as left terminals are securely fastened.

Commitment No. RCMS 1990-179 from NO 90-0104, dated April 9, 1990

Commitment Description: In the RCMS, the commitment is stated as "To prevent reoccurrence of these events, the functions of administering the fitness-for-duty program and the access screening program have been combined under one supervisor in the human resources group."

Change to Commitment: The commitment was changed to reflect an organizational change.

Justification: The organizational change resulted in the responsible supervisor changing from Supervisor in Human Resources to the current supervisor is Supervisor Access Screening, and the Manager Security is the responsible manager

Commitment No. RCMS 1993-229 from WM 93-0113, dated September 17, 1993

Commitment Description: In the RCMS, the commitment is stated as "Leakage rates will be verified periodically through the use of the thermocouples to determine if valve BGHV8145 requires rework."

The purpose of NRC Bulletin 88-08 was to request licensees review their reactor coolant systems to identify any connected, un-isolable piping that could be subjected to temperature

distributions which would result in unacceptable thermal stresses and take action, where such piping is identified, to ensure that the piping will not be subjected to unacceptable thermal stresses. The auxiliary pressurizer spray lines were initially subject to analysis due to potential unacceptable thermal stresses.

Change to Commitment: The commitment was closed and archived.

Justification: The Electric Power Research Institute (EPRI) Materials Reliability Program (MRP) published "Management of Thermal Fatigue in Normally Stagnant Non-isolable Reactor Coolant Branch Lines (MRP 146, Revision 1)" in 2011. This program was established by the industry as a result of NRC Bulletin 88-08. The work performed by EPRI, under MRP 146, Revision 1, found conclusively that the auxiliary pressurizer spray lines are not subject to unacceptable thermal stresses. The MRP 146 report found that the, "evaluation of plant configurations will generally show that leakage of auxiliary pressurizer spray control valves can result in in-leakage toward the main spray lines. Analysis and testing conducted in this program have demonstrated that the velocities in the main spray lines are not sufficient to produce thermal fatigue cycling due to insufficient swirl penetration into the auxiliary spray line. Thus, auxiliary pressurizer spray lines should be screened out for normal operating conditions, and no further evaluation of these lines is required."

Commitment No. RCMS 2006-210 from ET 07-0020, dated May 25, 2007

Commitment Description: In the RCMS, the commitment is stated as "A review of the calibration surveillance test results will be completed before the period of extended operation and every 10 years thereafter. (Electrical Cables Not Subject to 10 CFR 50.49 Environmental Qualification Requirements Used in Instrumentation Circuits, LRA Appendix A, Section A1.25.)"

Change to Commitment: The commitment was changed from reviewing calibration surveillance test results to requiring cable testing to be performed.

Justification: The commitment is being revised to be consistent with NUREG-1801, Rev. 1 (GALL) and NUREG-1915 (SER). NUREG-1801 Rev. 1, XI.E2 Element 4 states "in cases where a calibration or surveillance program does not include the cabling system in the testing circuit, or as an alternative to the review of calibration results described above, the applicant will perform cable system testing."

Commitment No. RCMS 2007-251 from ET 07-0016, dated May 10, 2007

Commitment Description: In the RCMS, the commitment is stated as "The WCNOG Nickel Alloy Aging Management Program will be supplemented with implementation of applicable (1) NRC Orders, Bulletins and Generic Letters associated with nickel alloys and (2) staff-accepted industry guidelines, (3) participate in the industry initiatives, such as owners group programs and the EPRI Materials Reliability Program, for managing aging effects associated with nickel alloys, (4) upon completion of these programs, but not less than 24 months before entering the period of extended operation, WCNOG will submit an inspection plan for reactor coolant system nickel alloy pressure boundary components to the NRC for review and approval."

Change to Commitment: The commitment was closed and archived.

Justification: The original intent of the commitment was to incorporate industry documents into the inspection program, as the documents were approved. The NRC has published 10 CFR 50.55a approving ASME Code Cases N-729-1, N-770-1, and N-722-1 with conditions. WCNOG has incorporated the requirements set forth in by 10 CFR 50.55a and the associated Code Cases into WCRE-30, In-Service Inspection Program Plan Interval 4. No additional approvals are required for the inspection plan.

Commitment No. RCMS 2012-438 from WO 12-0044, dated May 9, 2012

Commitment Description: In the RCMS, the commitment is stated as "Provide an assessment of the onsite and augmented staff needed to respond to a large scale natural event meeting the conditions described in the Discussion section of letter from NRC, dated March 12, 2012, Request for Information Pursuant to Title 10 of the code of Federal Regulations 50.54(f) Regarding Recommendations 2.1, 2.3, and 9.3, of the Near-Term Task Force Review of Insights from the Fukushima Dai-Ichi Accident. This assessment should include a discussion of the onsite and augmented staff available to implement the strategies as discussed in the emergency plan and/or described in plant operating procedures.

- The following functions are requested to be assessed: How onsite staff will move back-up equipment (e.g., pumps, generators) from alternate onsite storage facilities to repair locations at each reactor as described in the order regarding the Near-Term Task Force (NTTF) Recommendation 4.2. It is requested that consideration be given to the major functional areas of NUREG-0654, Table B-1 such as plant operations and assessment of operational aspects, emergency direction and control, notification/communication, radiological accident assessment, and support of operational accident assessment, as appropriate.
- New staff or functions identified as a result of the assessment.
- Collateral duties (personnel not being prevented from timely performance of their assigned functions).
- Provide onsite and augmented staffing assessment considering functions related to NTTF Recommendation 4.2. [Phase 2 staffing assessment][Staffing Request # 1]."

Change to Commitment: The commitment was closed and archived.

Justification: The staffing assessment was conducted in November 2015 using draft FLEX Support Guidelines. The assessment concluded that the current on-shift minimum staffing designated in the Emergency Plan is sufficient to perform the tasks associated with a beyond design basis natural event and implement the Emergency Plan. The responses provided to the NRC and the FLEX Staffing Study can be found in letter WO 16-0003 and the enclosure. This commitment has been satisfied.

Commitment No. RCMS 2012-439 from WO 12-0044, dated May 9, 2012

Commitment Description: In the RCMS, the commitment is stated as "Provide an implementation schedule of the time needed to conduct the onsite and augmented staffing assessment, and if any modifications are determined to be appropriate, please include in the schedule the time to implement the changes. Conduct the onsite and augmented staffing assessment: The onsite and augmented staffing assessment considering functions related to NTTF Recommendation 4.2. [Phase 2 staffing assessment][Staffing Request # 2]"

Change to Commitment: The commitment was closed and archived.

Justification: The staffing assessment was conducted in November 2015 using draft FLEX Support Guidelines (FSGs). The Final Integrated Plan and FLEX Compliance Report were submitted in January 2017 by letter ET 17-0003. The staffing assessment concluded that the current on-shift minimum staffing designated in the Emergency Plan is sufficient to perform the

tasks associated with a beyond design basis natural event and implement the Emergency Plan. No changes needed to be made to the Emergency Plan regarding on-shift or augmented staffing necessary to respond to a beyond design basis natural event and implement the Emergency Plan. The responses to this commitment and the FLEX Staffing Study can be found in Letter WO 16-0003. This commitment has been satisfied.

Commitment No. RCMS 2012-440 from WO 12-0044, dated May 9, 2012

Commitment Description: In the RCMS, the commitment is stated as "Provide an implementation schedule of the time needed to conduct the onsite and augmented staffing assessment, and if any modifications are determined to be appropriate, please include in the schedule the time to implement the changes. A schedule of the time needed to implement changes will be provided as follows: Those associated with the Phase 2 staffing assessment. [Staffing Request # 2]"

Change to Commitment: The commitment was closed and archived.

Justification: The staffing assessment was conducted in November 2015 using draft FLEX Support Guidelines. The Final Integrated Plan and FLEX Compliance Report for full implementation of FLEX were submitted in January 2017 by letter ET 17-0003. The staffing assessment concluded that the current on-shift minimum staffing designated in the Emergency Plan is sufficient to perform the tasks associated with a beyond design basis natural event and implement the Emergency Plan. No changes needed to be made to the Emergency Plan regarding on-shift or augmented staffing necessary to respond to a beyond design basis natural event and implement the Emergency Plan. The NRC responses and FLEX Staffing Study can be found in letter WO 16-0003. This commitment has been satisfied.

Commitment No. RCMS 2012-444 from WO 12-0044, dated May 9, 2012

Commitment Description: In the RCMS, the commitment is stated as "Identify changes that have been made or will be made to your emergency plan regarding the on-shift or augmented staffing changes necessary to respond to a loss of all AC power, multi-unit event, including any new or revised agreements with offsite resource providers (e.g., staffing, equipment, transportation, etc.). Changes will be identified as follows: Those associated with the Phase 2 staffing assessment. [Staffing request #6]"

Change to Commitment: The commitment was closed and archived.

Justification: This commitment started as a request for information from the NRC in the original Fukushima letter for Recommendations 2.1, 2.3, and 9.3, dated March 12, 2012. The response to this commitment is provided in letter WO 16-0003. No changes needed to be made to the Emergency Plan regarding on-shift or augmented staffing necessary to respond to a beyond design basis natural event and implement the Emergency Plan. The response in letter WO 16-0003 satisfies this commitment.

Commitment No. RCMS 2012-465 from WM 12-0041, dated October 31, 2012

Commitment Description: In the RCMS, the commitment is stated as "Modifications resulting from the engineering feasibility evaluation, if any, will be completed consistent with WCNOC's answer to NRC Order EA-12-049."

The NRC requested licensees provide an assessment of the current communications systems and equipment used during an emergency event to identify any enhancements that may be needed to ensure communications are maintained during large-scale natural events. The assessment was performed in accordance with NEI 12-01, "Guideline for Assessing Beyond Design Basis Accident Response Staffing and Communications Capabilities." For this commitment, the assessment resulted in the current private branch exchange telephone system being replaced with the voice over internet protocol (VoIP) system, and an engineering feasibility evaluation of potential plant modification requirements for recharging satellite telephones and operating battery recharging stations at permanent locations

Change to Commitment: The commitment was closed and archived.

Justification: The Iridium phone power supplies and antennas were installed under Design Change Package (DCP) 14529 and Work Order (WO) 13-375523. DCP 14259 was for permanent docking/charging stations and Ultimate Power Sources for the system.

The VoIP phone UPS power supplies were installed in the Olive Ann Beech building, Dwight D. Eisenhower building, operations relief area and radio room, Technical Support Center, and security locations under DCP 14625 (WO 14-384999), DCP 14626 (WO 14-385000), DCP 14627 (WO 14-385001), DCP 14628 (WO 14-385002), and DCP 14629 (WO 14-385003).

This commitment has been satisfied.

Commitment No. RCMS 2012-467 from WM 12-0041, dated October 31, 2012

Commitment Description: In the RCMS, the commitment is stated as "Modifications will be completed consistent with WCNOC's answer to NRC Order EA-12-049."

Change to Commitment: The commitment was closed and archived.

Justification: The Iridium phone power supplies and antennas were installed under Design Change Package (DCP) 14529 and Work Order (WO) 13-375523. DCP 14259 was for permanent docking/charging stations and Ultimate Power Sources for the system.

The VoIP phone UPS power supplies were installed in the Olive Ann Beech building, Dwight D. Eisenhower building, operations relief area and radio room, Technical Support Center, and security locations under DCP 14625 (WO 14-384999), DCP 14626 (WO 14-385000), DCP 14627 (WO 14-385001), DCP 14628 (WO 14-385002), and DCP 14629 (WO 14-385003).

This commitment has been satisfied.

Commitment No. RCMS 2015-500 from ET 15-0026, dated November 12, 2015

Commitment Description: In the RCMS, the commitment is stated as “WCNOC will walkdown inaccessible items, perform High Confidence of Low Probability of Failure (HCLPF) table updates, report any required modifications with proposed completion dates, and provide a letter with this supplemental information.”

Change to Commitment: The commitment was closed and archived.

Justification: In Letter ET 15-0026, WCNOC sent a supplement to the NRC for the ESEP Report. Attachment II of the supplement stated, “The additional items (Items 137-162) provided to the Wolf Creek Generating Station ESEL as a result of WCNOC letter ET 15-0015 (response to NRC request for clarification) are located inside containment, with the exception of Items 141 and 142 (Steam Generator Atmospheric Relief Valves ABPV0001 and ABPV0002). The components inside containment may not be walked down until Refueling Outage 21, currently scheduled for fall 2016. If existing documentation is sufficient, further walkdowns may not be necessary.” Existing documentation in engineering evaluation number FD-16-009 (found in the Curator calculation library) and Purchase Order #777126 were sufficient. Due to sufficient access being provided for all Expedited Seismic Equipment List (ESEL) items, additional walkdowns of inaccessible items were not necessary. Existing documentation and previous walkdowns were used to prepare evaluations for items 137-162, the inaccessible walkdown items provided in Attachment A of submittal ET 15-0026. HCLPF table updates were provided in NRC submittal ET 16-0020. Based on the results obtained, no modifications were required. Due to additional walkdowns and modifications not being necessary, and updates being provided in submittal ET 16-0020, this commitment has been satisfied.

Commitment No. RCMS 2016-503 from WM 16-0006, dated March 10, 2016

Commitment Description: In the RCMS, the commitment is stated as “WCNOC will complete installation of the replacement compressor on the SGK05A Class 1E electrical equipment air-conditioning unit.”

EA-16-018 contains a Notice of Violation (NOV) for not establishing adequate measures to assure that the cause of train A Class 1E electrical equipment air-conditioning system (SGK05A unit) trips that occurred on October 18, 2013, was determined and corrective action to preclude repetition of SGK05A unit trips.

Change to Commitment: The commitment was closed and archived.

Justification: The installation of the replacement SGK05A compressor was performed under WO 13-380760-020, WO 13-380760-016, and Design Change Package 14566. All field change requests have been addressed. Both implementing Work Orders have been completed, reviewed, and found acceptable. Return to Operations (RTO) form APF 05-005-06 was completed on November 4, 2016. The original commitment was a response to NOV EA-16-018 to restore compliance with an obligation, prior to start up from Refueling Outage 21. Compliance was restored during the timeframe provided to the NRC.

Attachment II to WO 17-0009
USAR Changes Processed Since Revision 29

USAR Change Request	Description		
16-007	REVISE THE USAR TO REFLECT REVISION TO PROCEDURE SYS JE-201, EMERGENCY FUEL OIL STORAGE TANK FILLING AND VENTING, AND CHANGE USAR TO READ, "IN THE UNLIKELY EVENT THAT BOTH TANK VENTS ARE COMPLETELY RESTRICTED, EITHER TANK CAN BE VENTED BY ALTERNATIVE MEANS."		
Page: 3.5-13			
16-008	REVISE THE USAR TO REMOVE SCAFFOLD TABLE.		
Page: 1.0-iv	Page: 1.0-v	Page: 1.7-1	
Table: 1.7-4		Table: 1.7-4	Sheet: 2
Table: 1.7-4	Sheet: 3	Table: 1.7-4	Sheet: 4
Table: 1.7-4	Sheet: 5		
16-009	REVISE THE USAR TO REFLECT THAT THE SECTION XI EDITION/ADDENDA THAT WOLF CREEK IS NOW REQUIRED TO USE DOES NOT REQUIRE HYDROSTATIC TESTING FOR SYSTEMS SUBJECT TO PRESSURE TESTS. USAR IS ALSO BEING REVISED TO REFLECT THAT THE ISI PROGRAM CONFORMS TO THE 2007 EDITION THROUGH 2000 ADDENDA FOR THE 4TH ISI INTERVAL PER 10 CFR 50.55a(g)(4)(ii).		
Page: 18A-2	Page: 18A-3	Page: 6.6-5	

USAR Change Request Description

16-010 USAR EDITORIAL CHANGES. THESE INCLUDE GRAMMAR AND SPELLING CORRECTIONS, THE CORRECTION OF USAR CHANGES PREVIOUSLY APPROVED BUT INCORPORATED INCORRECTLY, AND INFORMATION BEING REMOVED UNDER NEI 98-03, GUIDELINES FOR UPDATING FINAL SAFETY ANALYSIS REPORT.

Page: 1.1-2	Page: 1.2-6	Page: 1.2-7	Page: 1.4-12
Page: 2.4-10	Page: 2.4-73	Page: 3.0-xxxiii	Page: 2.5-317
Page: 3.8-45	Page: 3.8-60	Page: 3.8-72	Page: 3.8-76
Page: 3A-23	Page: 3A-40	Page: 3B-7	Page: 3B-10
Page: 3B-11	Page: 3B-12	Page: 3B-16	Page: 6.4-3
Page: 9.2-6	Page: 9.2-8	Page: 9.2-9	Page: 9.2-10
Page: 9.2-32	Page: 9.2-40	Page: 9.4-47	Page: 9.4-50
Page: 9.4-51	Page: 9.4-54	Page: 9.4-94	Page: 11.4-10
Page: 13.1-11	Page: 13.1-17	Page: 13.1-18	Page: 13.1-20
Page: 15.0-xi	Page: 18.2-27	Page: 18.2-44	Page: 18.2-82
Page: 18.2-83	Page: 18.2-85	Page: 18.2-87	Page: 18.2-91
Page: 18A-3	Page: 18A-4	Page: 18A-13	Page: 422-1

Table: 1.6-3	Sheet: 3	Table: 2.5-53	
Table: 3.2-1	Sheet: 6	Table: 3.2-1	Sheet: 20
Table: 3.2-1	Sheet: 21	Table: 3.2-1	Sheet: 3 (Notes)
Table: 3.3-1		Table: 3.6-4	Sheet: 19-73
Table: 3B-3	Sheet: 2	Table: 3B-3	Sheet: 3
Table: 3B-3	Sheet: 4	Table: 3B-3	Sheet: 5
Table: 7.2-3		Table: 9.2-9	Sheet: 2
Table: 9.4-10	Sheet: 7	Table: 9.5.1-2	Sheet: 2
Table: 9.5.1-2	Sheet: 3	Table: 9.5.9-1	
Table: 13.1-1	Sheet: 7	Table: 15.7-5	Sheet: 2

Figure: 11.1A-3 Figure: 13.1-1

16-011 REVISE THE USAR TO CORRECT STATEMENTS IN SECTIONS 9.5.4.2.1, 9.5.5.2.1, AND 9.5.7.1 ON THE FUEL OIL DAY TANK, HEAT EXCHANGERS AND LUBE OIL COOLER MATERIAL.

Page: 9.5-47	Page: 9.5-55
Table: 9.5.7-1	Sheet: 1

USAR Change Request	Description
16-012	REVISE THE USAR TO REMOVE DESCRIPTION OF TEMPORARY MODIFICATION 15-015-EM-00 THAT WAS PUT INTO THE USAR UNDER CHANGE REQUEST 15-011. A PERMANENT MODIFICATION OF THE CRIMPED LINE WAS PERFORMED BY REMOVING THE CRIMP AND FURMANITE AND REPLACING THE LINE WITH NEW PIPE AND VALVE EMV0709.
Page: 6.3-39	
16-013	REVISE THE USAR TABLE 11.4-1 TO CLARIFY HOW WOLF CREEK GENERATING STATION (WCGS) PROCESSES WET WASTES.
Table: 11.4-1	Sheet: 2
16-014	REVISE THE USAR, APPENDIX 18A-15, TO REFLECT THAT SURVEILLANCE TEST PROCEDURES HAVE BEEN REVISED SINCE THE LICENSE RENEWAL APPLICATION WAS APPROVED IN SAFETY EVALUATION (SE), NUREG-1915, AND CABLE TESTIING IS NOW REQUIRED TO BE PERFORMED.
Page: 18A-15	
16-015	REVISE THE USAR TO REFLECT THE WEIGHT OF THE TOTAL HEAD ASSEMBLY IS CHANGING FROM 355,892 POUNDS TO 356,322 POUNDS. THIS CHANGE IS A RESULT OF CHANGE TO CALC 0720517.01-CN001, WOLF CREEK GENERATING STATION SIMPLIFIED HEAD ASSEMBLY DROP ANALYSIS.
Table: 9.1-10	
16-016	REVISE THE USAR TO MAKE 18A1.19 CONSISTENT WITH NUREG-1915, SECTION 3.0.3.2.15, PAGE 3-103 WHICH STATES, "LRA SECTION B2.1.19 STATES THAT THERE HAS BEEN NO CRACKING OF STAINLESS STEEL ASME CODE CLASS 1 PIPING WITH NOMINAL PIPE SIZE (NPS) OF LESS THAN OR EQUAL TO 4 INCHES, OR IN PIPING WITH NPS OF GREATER THAN OR EQUAL TO 1 INCH." THE 2007 EDITION WITH THE 2008 ADDENDA OF ASME SECTION XI CODE, IWB-1220, PROVIDES THE GUIDANCE FOR COMPONENTS EXEMPT FROM EXAMINATION. THE PROPOSED CHANGE TO THE PROGRAM REQUIREMENTS EXEMPT INSPECTING COMPONENTS AND PIPING SEGMENTS NPS 1 INCH AND SMALLER.
Page: 18A-12	Page: 18A-13

USAR Change Request Description

16-017 REVISE THE USAR TO UPDATE TABLE 9.4-10 WITH THE MINIMUM REQUIREMENTS NECESSARY FOR TURBINE DECK OFFICE MEZZANINE ROOM AIR CONDITIONING SYSTEM. SUPERCEDED BY USARCR 2016-010.

This USAR package was superceded by 16-010.

16-018 REVISE THE USAR TO REFLECT THAT WITH A SINGLE NON-FUNCTIONAL SGK05A OR SGK05B (CLASS 1E ELEC. EQUIP. AC UNIT), CONCURRENT WITH ACCIDENT CONDITION, THE HEAT LOADING MAY INCREASE BATTERY ROOM TEMPERATURES TO A MAXIMUM OF 104 DEGREES F. THE AMBIENT TEMPERATURE IN THE BATTERY ROOMS, UNDER ANY MODE OF OPERATION IS BETWEEN 60 AND 90 DEGREES F.

Page: 9.4-14

16-019 REVISE THE USAR TO UPDATE REFERENCE IN TABLES 9.4-2 AND 9.4-3 FROM TABLE 5.1 OF ANSI N509-1976 TO TABLE 5.1 OF ANSI N509-1980.

Page: 3A-20

Page: 3A-58

Table: 9.4-3

Sheet: 7

Table: 9.4-3

Sheet: 12

Table: 9.4-3

Sheet: 13

Table: 9.4-2

Sheet: 9

Table: 9.4-2

Sheet: 10

Table: 9.4-2

Sheet: 16

Table: 9.4-2

Sheet: 18

16-020 REVISE THE USAR TO REFLECT THE ADDITION OF AN ANCHOR AND MODIFICATION OF TWO SUPPORTS TO SEISMICALLY RATED AUXILIARY STEAM PIPELINE.

Page: 3.6-14

Page: 3.6-15

Table: 3.6-3

Sheet: 74

Table: 3.6-4

Sheet: 16

Table: 3.6-4

Sheet: 21

Figure: 3.6-1

Sheet: 47

16-021 REVISE THE USAR TO REFLECT THE REPLACEMENT OF OBSOLETE GENERAL ELECTRIC AKR-5AE-30 BREAKERS WITH NLI/SQUARE D MASTERPACT BREAKERS FOR USE IN NG LOAD CENTER CUBICLES NG00308 AND NG00408.

Page: 8.3-7

Table: 3.10(B)-1

USAR Change Request

Description

16-022 **REVISE THE USAR TO REFLECT THE APPROVAL OF ASME CODE CASES IN 10 CFR 50.55a AND TO COMBINE THE TWO NICKEL ALLOY PROGRAMS INTO ONE PROGRAM.**

Page: 18A-3

Page: 18A-4

Page: 18A-20

Page: 18A-21

16-023 **REVISE THE USAR TO REFLECT A FIRE SUPPRESSION SYSTEM INSTALLED IN THE ESSENTIAL SERVICE WATER VERTICAL LOOP CHASE BUILDING.**

Page: 9.5-19

Page: 9.5-9

Table: 9.5.1-2

Sheet: 2

16-024 **REVISE THE USAR TO REFLECT ORGANIZATIONAL TITLE CHANGES, LOCATION CHANGES, CODE OF FEDERAL REGULATION CHANGES, AND INSTRUMENTATION AND ESTIMATED QUANTITY CHANGES FOR RADIATION PROTECTION.**

Page: 12.5-1

Page: 12.5-4

Page: 12.5-5

Page: 12.5-6

Page: 12.5-8

Table: 12.5-2

Table: 12.5-2

Sheet: 2

Table: 12.5-1

Figure: 12.5-1

Figure: 12.5-2

Figure: 12.5-4

16-025 **REVISE THE USAR TO REFLECT THE EXTENSION OF THE CORPORATE NETWORK AS A NEW INTRA PLANT COMMUNICATION SYSTEM AND THAT BOTH METALLIC NETWORK CABLE AND FIBER OPTIC CABLE RUNS IN THE RACEWAY TO SUPPORT THE CORPORATE NETWORK.**

Page: 9.5-38

Page: 9.5-40

16-026 **REVISE THE USAR TO REFLECT THE ABANDONMENT OF THE BULK STORAGE FOR CARBON DIOXIDE (CO₂) AND TO INCLUDE THAT CO₂ IS NOW TRUCKED IN AND OFFLOADED TO THE PLANT PIPING TO PURGE THE TURBINE GENERATOR.**

Page: 2.2-4

Page: 2.2-5

Page: 9.3-70

USAR Change Request	Description		
16-027	REVISE THE USAR TO REFLECT CHANGES IN MOTOR OPERATED VALVES IN CONTAINMENT AND HOW THE ASSOCIATED ELECTRICAL PENETRATION ASSEMBLY IS PROTECTED.		
Page: 8.1-12	Page: 8.1-13	Page: 8.1-14	Page: 8.1-15
16-028	REVISE THE USAR TO REFLECT THE INSTALLATION OF TWO NEW HEATERS IN THE ESSENTIAL SERVICE WATER VERTICAL LOOP CHASE TO SUPPLY SUPPLEMENTAL HEATING WHEN AMBIENT TEMPERATURES DROP BELOW 40 DEGREES F.		
Page: 9.4-94			
Table: 3.2-1	Sheet: 20		
16-029	REVISE THE USAR TO REFLECT MOUNTING SUPPORT STRUCTURE- FIXED GUIDE RAILS AND MOUNTING PLATES- INSTALLED FOR A REFUELING CAVITY ELEVATOR.		
Table: 3.2-1	Sheet: 7		
Figure: 3.8-3		Figure: 3.8-6	
16-030	REVISE THE USAR TO REFLECT THE INCORPORATION OF ASME CODE CASE N-729-1 INTO THE WOLF CREEK INSERVICE INSPECTION PROGRAM, SUBJECT TO THE CONDITIONS SPECIFIED IN 10CFR50.55a(g)(6)(ii)(D).		
Page: 18A-3			
16-031	REVISE THE USAR TO REFLECT THE REMOVAL OF A TEMPORARY DIESEL GENERATOR THAT FEEDS THE NON-SAFETY AUXILIARY FEEDWATER PUMP (NSAFP) WILL BE REMOVED. THE STATION BLACKOUT DIESEL GENERATOR WILL BE MODIFIED TO FEED THE NSAFP.		
Page: 1.2-17	Page: 2.0-xxiv	Page: 8.1-3	Page: 8.3-4
Page: 8.3-5	Page: 8.3-6	Page: 8.3-7	Page: 9.2-42
Page: 9.2-43	Page: 9.5-86	Page: 9.5-87	Page: 10.4-48
Figure: 2.4-3	Sheet: 1		

USAR Change Request

Description

16-032 **REVISE THE USAR TO CORRECT ERRORS AND INCONSISTENCIES IN THE FIRE PROTECTION PROGRAM. CORRECTIONS ARE BEING MADE TO FIRE EXTINGUISHER LOCATIONS AND TYPES, SECTION REFERENCES, AND TEST HEADER LOCATIONS.**

Page: 9.5-7	Page: 9.5-8	Page: 9.5-12	Page: 9.5-17
Page: 9.5-22	Page: 9.5-23	Page: 9.5-25	
Table: 9.5E-1	Sheet: 5	Table: 9.5A-1	Sheet: 3
Table: 9.5A-1	Sheet: 7	Table: 9.5A-1	Sheet: 60

16-033 **REVISE THE USAR TO REMOVE THE SPECIFIC RECEIVER TIMES AND PRESSURES FROM THE USAR FOR THE NON-SAFETY COMPRESSED AIR RECEIVER. THE RECEIVER STARTING PRESSURE IN THE USAR IS A NOMINAL SYSTEM PRESSURE DIFFERING FROM CALCULATION KA-M-002-001-CN002 PRESSURE WHICH CONSIDERS WHEN A COMPRESSOR NORMALLY LOADS. INFORMATION IS BEING REMOVED FOR CLARITY AND REMOVAL OF EXCESSIVE DETAIL.**

Page: 9.3-2	Page: 9.3-3
-------------	-------------

17-001 **REVISE THE USAR TO REFLECT THE RELOCATION OF THE WAVERLY/LACYGNE TRANSMISSION LINE TO ENTER THE 345KV SWITCHYARD FROM THE EAST AND THEN TURN SOUTH TO CONNECT TO A BREAKER-AND-A-HALF POSITION BETWEEN CIRCUIT BREAKERS 345-110 AND 345-120. THE TIE IN POINT FOR TRANSFORMER NO. 7 IS LOCATED TO A BREAKER-AND-A-HALF POSITION BETWEEN CIRCUIT BREAKERS 345-80 AND 345-90.**

Figure: 8.2-1	Figure: 8.2-5
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17-002 **REVISE THE USAR TO REFLECT THE INSTALLATION OF OPEN PHASE DETECTION EQUIPMENT.**

Page: 8.2-6	Page: 8.2-7	Page: 8.2-8
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17-003

REVISE THE USAR TO INCORPORATE THE ANALYSIS OF A TORNADO MISSILE IMPACT ON THE TURBINE DRIVEN AUXILIARY FEEDWATER PUMP TURBINE EXHAUST STACK AS A RESULT OF CALCULATION FC-M-007, REV 0. THIS CALCULATION WAS PERFORMED FOR THE CURRENT DESIGN OF THE EXHAUST STACK AND NOT A MODIFICATION.

Page: 10.4-51

Page: 10.4-52

Page: 10.4-53

Page: 10.4-54

Page: 10.4-55

Page: 10.4-56

Page: 10.4-57

Attachment III to WO 17-0009

Revision 63 to the Technical Requirements Manual

REVISIONS TO THE TECHNICAL REQUIREMENTS MANUAL (TRM)

1. TR B 3.8.1 Required Action B.1 was revised to clearly define when NB01 is considered inoperable. The upgrade to multifunction digital relays allowed the implementation of a dual zone approach to the generator out of step protection. The digital logic capabilities of the relays created a scheme that will provide out of step protection for all transmission line configurations without overreaching the lines. The scheme protects against loss of offsite power events by use of out of step blocking on the transmission line relays for the last line in service.

Enclosure I to WO 17-0009

CD-ROM containing Updated Safety Analysis Report, Revision 30

Subject

Enclosed is the CD-ROM submittal of the Wolf Creek Generating Station Updated Safety Analysis Report (WCGS USAR), Revision 30.

Contact Name Lucille Stone
Mailing Address Wolf Creek Nuclear Operating Corporation
P.O. Box 411
Burlington, KS 66839
E-Mail Address lurocke@wcnoc.com
Phone Number 620-364-8831 ext. 4898

Document Components:

The CD-ROM labeled "Wolf Creek Generating Station Updated Safety Analysis Report, Rev.30" contains the following files:

001_USAR.pdf	249 KB, sensitive unclassified information
002_USARC01.pdf	926 KB, publicly available
003_USARC02.0.pdf	34.8 MB, sensitive unclassified information
004_USARC02 Figures.pdf	35.5 MB, sensitive unclassified information
005_USARC03.pdf	16.7 MB, sensitive unclassified information
006_USARC04.pdf	2.34 MB, publicly available
007_USARC05.pdf	2.40 MB, publicly available
008_USARC06.pdf	11.5 MB, publicly available
009_USARC07.pdf	1.87 MB, publicly available
010_USARC08.pdf	1.93 MB, sensitive unclassified information
011_USARC09.pdf	4.59 MB, sensitive unclassified information
012_USARC10.pdf	1.46 MB, publicly available
013_USARC11.pdf	1.03 MB, publicly available
014_USARC12.pdf	876 KB, sensitive unclassified information
015_USARC13.pdf	1.02 MB, publicly available
016_USARC14.pdf	376 KB, publicly available
017_USARC15.pdf	6.31 MB, publicly available
018_USARC16.pdf	56 KB, publicly available
019_USARC17.pdf	64 KB, publicly available
020_USARC18.pdf	1.23 MB, publicly available
021_USARNRCQ.pdf	376 KB, publicly available
022_USAR Rev. 30-loep.pdf	397 KB, publicly available

Enclosure II to WO 17-0009

CD-ROM containing Updated Safety Analysis Report Controlled Figure Drawings

Subject

Enclosed is the CD-ROM submittal of the Wolf Creek Generating Station Updated Safety Analysis Report (WCGS USAR) controlled figure drawings that are considered incorporated by reference into the WCGS USAR. In accordance with 10 CFR 2.390, this enclosure is considered sensitive unclassified information and therefore warrants withholding.

Contact Name Lucille Stone
Mailing Address Wolf Creek Nuclear Operating Corporation
P.O. Box 411
Burlington, KS 66839
E-Mail Address lurocke@wcnoc.com
Phone Number 620-364-8831 ext. 4898

Document Components:

The CD-ROM labeled "WCGS Updated Safety Analysis Report, Rev.30 Controlled Figure Drawings Only" contains the following files:

001_Chapter 1.pdf	14.6 MB, sensitive unclassified information
002_Chapter 2.pdf	2.06 MB, sensitive unclassified information
003_Chapter 5.pdf	2.06 MB, sensitive unclassified information
004_Chapter 6.pdf	3.09 MB, sensitive unclassified information
005_Chapter 7.pdf	1.25 MB, sensitive unclassified information
006_Chapter 8.pdf	2.38 MB, sensitive unclassified information
007_Chapter 9.pdf	27.8 MB, sensitive unclassified information
008_Chapter 10.pdf	11.6 MB, sensitive unclassified information
009_Chapter 11.pdf	3.69 MB, sensitive unclassified information
010_Chapter 12.pdf	657 KB, sensitive unclassified information
011_Chapter 18.pdf	215 KB, sensitive unclassified information

Enclosure III to WO 17-0009

CD-ROM containing Quality Program Manual,
Updated Safety Analysis Report Fire Hazards Analysis

Subject

Enclosed is the CD-ROM submittal of the Wolf Creek Generating Station Wolf Creek Quality Program Manual (WCQPM), Updated Safety Analysis Report Fire Hazards Analysis, all of which are considered incorporated by reference into the WCGS USAR. In accordance with 10 CFR 2.390, the WCGS USAR Fire Hazards Analysis is considered sensitive unclassified information and therefore warrants withholding.

Contact Name Lucille Stone
Mailing Address Wolf Creek Nuclear Operating Corporation
P.O. Box 411
Burlington, KS 66839
E-Mail Address lurocke@wcnoc.com
Phone Number 620-364-8831 ext. 4898

Document Components:

The CD-ROM labeled "WCGS USAR Fire Hazards Analysis and Quality Program Manual" contains the following files:

001_WCQPM Rev 10A.pdf	246 KB, publicly available
002_E-1F9900 Rev 9.pdf	3.03 KB, sensitive unclassified information
003_E-1F9905 Rev 8.pdf	9.97 KB, sensitive unclassified information
004_E-1F9910 Rev 14.pdf	20.8 MB, sensitive unclassified information
005_E-1F9915 Rev 8.pdf	1.43 KB, sensitive unclassified information
006_XX-E-013 Rev 4.pdf	3.72 KB, sensitive unclassified information
007_XX-E-013 Rev 4_CN002.pdf	440 KB, sensitive unclassified information
008_XX-E-013 Rev 4_CN 003pdf	541 KB, sensitive unclassified information
012_M-663-00017A W05 1 to B1-98.pd	23.7 MB, sensitive unclassified information
013_M-663-00017A W05 B1-99 to B2-40.pdf	31.7 MB, sensitive unclassified information
014_M-663-00017A W05 B2-41 to B6-2.pdf	49.5 MB, sensitive unclassified information
015_M-663-00017A W05 B6-3 to B7-79.pdf	28.3 MB, sensitive unclassified information
016_M-663-00017A W05 B7-80 to B8-147.pdf	45.3 MB, sensitive unclassified information
017_M-663-00017A W05 B8-148 to B13-25.pdf	33.9 MB, sensitive unclassified information
018_M-663-00017A W05 B13-26 to G1A-63.pdf	42.7 MB, sensitive unclassified information
019_M-663-00017A W05 G1A-64 to G1A-123.pdf	17.0 MB, sensitive unclassified information
020_M-663-00017A W05 G1A-124 to G2A-3.pdf	15.7 MB, sensitive unclassified information
021_M-663-00017A W05 G2B-1 to G2B79.pdf	18.8 MB, sensitive unclassified information
022_M-663-00017A W05 Att G3 to G3D-26.pdf	22.0 MB, sensitive unclassified information
023_M-663-00017A W05 G3D-27 to G4D-19.pdf	20.6 MB, sensitive unclassified information
024_M-663-00017A W05 Att G5 to Att H.pdf	20.0 MB, sensitive unclassified information
025_M-663-00017A W05-01.pdf	65 KB, sensitive unclassified information
026_M-663-00017A W05-02.pdf	164 KB, sensitive unclassified information

Enclosure IV to WO 17-0009

CD-ROM containing EQSD-I, EQ Summary Document Section I Program Description,
and EQSD-II, Equipment Qualification Summary Document Master List Section II

Subject

Enclosed is the CD-ROM submittal of the Wolf Creek Generating Station Wolf Creek EQSD-I, EQ Summary Document Section I Program Description, and EQSD-II, Equipment Qualification Summary Document Master List Section II, both of which are considered incorporated by reference into the WCGS USAR. In accordance with 10 CFR 2.390, the WCGS EQSD-I, EQ Summary Document Section I Program Description, and EQSD-II, Equipment Qualification Summary Document Master List Section II, are considered sensitive unclassified information and therefore warrant withholding.

Contact Name Lucille Stone
Mailing Address Wolf Creek Nuclear Operating Corporation
P.O. Box 411
Burlington, KS 66839
E-Mail Address lurocke@wcnoc.com
Phone Number 620-364-8831 ext. 4898

Document Components:

The CD-ROM labeled "WCGS USAR Qualification Design Basis EQSD-I EQSD-II" contains the following files:

009_EQSD-I Rev 11.pdf	5.5 KB, sensitive unclassified information
010_EQSD-II Rev 28, Part 1.pdf	25.8 MB, sensitive unclassified information
011_EQSD-II Rev 28, Part 2.pdf	41.4 MB, sensitive unclassified information

Enclosure V to WO 17-0009

Revision 63 of Technical Requirements Manual Replacement Pages and List of
Effective Pages

(8 Pages)

TECH REQUIREMENTS MANUAL

REVISION: 63

TECHNICAL REQUIREMENTS MANUAL

Wolf Creek Generating Station, Unit 1

Summary of Revision 63:

Released: DC12 10/26/16

TR B 3.8.1 Required Action B.1 is being revised to clearly define when NB01 is considered inoperable. The upgrade to multifunction digital relays will allow the implementation of a dual zone approach to the generator out of step protection. The digital logic capabilities of the relays will create a scheme that will provide out of step protection for all transmission line configurations without overreaching the lines. The scheme protects against LOOP events by use of out of step blocking on the transmission line relays for the last line in service.

B 3.8 ELECTRICAL POWER SYSTEMS

TR B 3.8.1 Offsite Power System – Transmission Network

BASES

BACKGROUND General Design Criterion (GDC) 17 (Ref. 1), "Electric power system," of Appendix A to 10 CFR 50 requires, in part, that an onsite electric power system and an offsite electric power system shall be provided to permit functioning of structures, systems, and components important to safety. GDC 17 further specifies that electric power from the transmission network to the onsite electrical distribution system shall be supplied by two physically independent circuits (not necessarily on separate rights of way) designed and located so as to minimize to the extent practical the likelihood of their simultaneous failure under operating and postulated accident and environmental conditions. A switchyard common to both circuits is acceptable.

There are three 345 kV lines connecting the unit switchyard to the offsite transmission network. The three lines are:

- a. Wolf Creek – Waverly - LaCygne 345 kV line
- b. Wolf Creek – Rose Hill 345 kV line
- c. Wolf Creek – Benton 345 kV line

The above 345 kV lines do not share common rights-of-way, do not have any crossovers, and are not in close proximity of one another, except in the immediate vicinity of the unit switchyard. If any one of the three 345 kV lines faulted, the breakers located at the unit switchyard would trip, deenergizing the line. Any one 345 kV transmission line can carry the total engineered safety features (ESF) load required for safe shutdown. Any transmission line may be aligned by controlled switching to feed either of the two ESF transformers. Having at least two FUNCTIONAL 345 kV transmission lines will minimize the probability of losing electric power from the offsite transmission network.

Offsite power is supplied to the unit switchyard from the offsite transmission network. From the switchyard, two electrically and physically separated circuits (offsite circuits) provide AC power, through the ESF transformers, to the 4.16 kV ESF buses. The OPERABILITY of the offsite circuits and diesel generators are addressed in limiting condition for operation (LCO) 3.8.1, "AC Sources – Operating." A detailed description of the offsite power network and the offsite circuits to the Class 1E ESF buses is found in the USAR, Chapter 8 (Ref. 2).

BASES

APPLICABLE SAFETY ANALYSES The offsite transmission network is designed to provide sufficient capacity, capability, redundancy, and reliability to ensure the availability of necessary power to offsite circuits and Class 1E Electrical Power System so that the fuel, Reactor Coolant System, and containment design limits are not exceeded.

TR TR 3.8.1 requires three 345 kV transmission lines connecting the unit switchyard to the offsite transmission network be FUNCTIONAL. This provision minimizes the probability of losing electric power from the offsite transmission network as a result of, or coincident with, the loss of power generated by the unit.

A 345 kV transmission line is considered FUNCTIONAL when the transmission line is connected to the unit switchyard and capable of supplying power. The Wolf Creek – Waverly – LaCygne 345 kV line is sectioned into two line sections; the Wolf Creek to Waverly Switching Station 345 kV line and the Waverly Switching Station to LaCygne 345 kV line. The Wolf Creek – Waverly – LaCygne 345 kV line is considered FUNCTIONAL when both line sections are energized through both a Wolf Creek and a LaCygne 345 kV breaker and tied together through at least one 345 kV breaker in the Waverly Switching Station.

APPLICABILITY The FUNCTIONALITY of the 345 kV transmission lines is required in MODES 1, 2, 3, and 4.

ACTIONS

A.1

When one 345 kV line is nonfunctional (e.g., loss of one 345 kV line), action shall be taken within 1 hour to verify two 345 kV lines are FUNCTIONAL. Procedure OFN AF-025, "Unit Limitations," may be utilized for verifying the two 345 kV lines are capable of supplying power. A Condition Report should be initiated when this Condition is entered for trending the reliability of the offsite transmission network.

B.1

With two 345 kV lines nonfunctional, immediate entry into Condition A of Technical Specification LCO 3.8.1, "AC Sources – Operating," is required.

The offsite circuit energizing NB01 is considered inoperable when NB01 is only energized from the offsite transmission network through the 345-50 and 345-60 main generator breakers. For this configuration, switchyard breakers 345-120 and 345-80 are open.

LIST OF EFFECTIVE PAGES - TECHNICAL REQUIREMENT MANUAL BASES

PAGE ⁽¹⁾	REVISION NO. ⁽²⁾	CHANGE DOCUMENT ⁽³⁾	DATE EFFECTIVE/ IMPLEMENTED ⁽⁴⁾
TAB – Table of Contents			
i	43	DRR 10-3738	12/28/10
ii	25	DRR 05-1996	9/28/05
iii	54	DRR 13-2529	10/25/13
TAB – B 3.0 APPLICABILITY			
B 3.0-1	41	DRR 10-1702	10/1/10
B 3.0-2	41	DRR 10-1702	10/1/10
B 3.0-3	41	DRR 10-1702	10/1/10
B 3.0-4	41	DRR 10-1702	10/1/10
B 3.0-5	41	DRR 10-1702	10/1/10
B 3.0-6	41	DRR 10-1702	10/1/10
B 3.0-7	41	DRR 10-1702	10/1/10
B 3.0-8	41	DRR 10-1702	10/1/10
B 3.0-9	41	DRR 10-1702	10/1/10
B 3.0-10	41	DRR 10-1702	10/1/10
B 3.0-11	41	DRR 10-1702	10/1/10
B 3.0-12	41	DRR 10-1702	10/1/10
B 3.0-13	41	DRR 10-1702	10/1/10
B 3.0-14	20	DRR 04-1410	10/7/04
TAB – B 3.1 REACTIVITY CONTROL SYSTEMS			
B 3.1.7-1	41	DRR 10-1702	10/1/10
B 3.1.7-2	41	DRR 10-1702	10/1/10
B 3.1.7-3	3	DRR 99-1581	12/18/99
B 3.1.8-1	41	DRR 10-1702	10/1/10
B 3.1.8-2	41	DRR 10-1702	10/1/10
B 3.1.9-1	15	DRR 03-0861	7/10/03
B 3.1.9-2	41	DRR 10-1702	10/1/10
B 3.1.9-3	41	DRR 10-1702	10/1/10
B 3.1.9-4	41	DRR 10-1702	10/1/10
B 3.1.9-5	41	DRR 10-1702	10/1/10
B 3.1.9-6	55	DRR 14-0344	2/27/14
B 3.1.9-7	28	DRR 06-1349	7/24/06
B 3.1.9-8	3	DRR 99-1581	12/18/99
B 3.1.10-1	41	DRR 10-1702	10/1/10
B 3.1.10-2	49	DRR 12-0674	4/10/12
B 3.1.10-3	48	DRR 12-0266	2/2/12
B 3.1.10-4	55	DRR 14-0344	2/27/14
B 3.1.10-5	48	DRR 12-0266	2/2/12
TAB – B 3.3 INSTRUMENTATION			
B 3.3.3-1	41	DRR 10-1702	10/1/10
B 3.3.3-2	41	DRR 10-1702	10/1/10
B 3.3.3-3	41	DRR 10-1702	10/1/10
B 3.3.3-4	41	DRR 10-1702	10/1/10
B 3.3.3-5	41	DRR 10-1702	10/1/10
B 3.3.3-6	28	DRR 06-1349	7/24/06
B 3.3.9-1	41	DRR 10-1702	10/1/10
B 3.3.9-2	41	DRR 10-1702	10/1/10
B 3.3.9-3	41	DRR 10-1702	10/1/10

LIST OF EFFECTIVE PAGES - TECHNICAL REQUIREMENT MANUAL BASES

PAGE ⁽¹⁾	REVISION NO. ⁽²⁾	CHANGE DOCUMENT ⁽³⁾	DATE EFFECTIVE/ IMPLEMENTED ⁽⁴⁾
TAB – B 3.3 INSTRUMENTATION (continued)			
B 3.3.9-4	8	DRR 01-0475	5/1/01
B 3.3.10-1	3	DRR 99-1581	12/18/99
B 3.3.10-2	41	DRR 10-1702	10/1/10
B 3.3.10-3	41	DRR 10-1702	10/1/10
B 3.3.11-1	41	DRR 10-1702	10/1/10
B 3.3.11-2	41	DRR 10-1702	10/1/10
B 3.3.11-3	41	DRR 10-1702	10/1/10
B 3.3.11-4	41	DRR 10-1702	10/1/10
B 3.3.11-5	39	DRR 09-1775	10/28/09
B 3.3.12-1	41	DRR 10-1702	10/1/10
B 3.3.12-2	37	DRR 09-0287	3/20/09
B 3.3.12-3	19	DRR 04-1019	9/1/04
B 3.3.13-1	41	DRR 10-1702	10/1/10
B 3.3.13-2	41	DRR 10-1702	10/1/10
B 3.3.13-3	41	DRR 10-1702	10/1/10
B 3.3.13-4	3	DRR 99-1581	12/18/99
B 3.3.14-1	51	DRR 13-0371	2/26/13
B 3.3.14-2	51	DRR 13-0371	2/26/13
B 3.3.14-3	53	DRR 13-1518	6/26/13
B 3.3.14-4	53	DRR 13-1518	6/26/13
B 3.3.14-5	51	DRR 13-0371	2/26/13
B 3.3.14-6	53	DRR 13-1518	6/26/13
B 3.3.14-7	51	DRR 13-0371	2/26/13
B 3.3.14-8	51	DRR 13-0371	2/26/13
B 3.3.14-9	51	DRR 13-0371	2/26/13
B 3.3.14-10	51	DRR 13-0371	2/26/13
B 3.3.14-11	51	DRR 13-0371	2/26/13
B 3.3.15-1	45	DRR 11-0662	3/21/11
B 3.3.15-2	45	DRR 11-0662	3/21/11
B 3.3.15-3	46	DRR 11-0725	4/11/11
B 3.3.15-4	60	DRR 15-0529	3/26/15
B 3.3.15-5	45	DRR 11-0662	3/21/11
B 3.3.16-1	41	DRR 10-1702	10/1/10
B 3.3.16-2	61	DRR 15-0824	4/28/15
B 3.3.16-3	61	DRR 15-0824	4/28/15
B 3.3.16-4	61	DRR 15-0824	4/28/15
B 3.3.16-5	41	DRR 10-1702	10/1/10
B 3.3.17-1	19	DRR 04-1019	9/1/04
B 3.3.17-2	41	DRR 10-1702	10/1/10
B 3.3.17-3	41	DRR 10-1702	10/1/10
B 3.3.17-4	41	DRR 10-1702	10/1/10
B 3.3.17-5	3	DRR 99-1581	12/18/99
B 3.3.18-1	14	DRR 02-1459	2/12/03
B 3.3.18-2	41	DRR 10-1702	10/1/10
B 3.3.18-3	49	DRR 12-0673	4/10/12
B 3.3.18-4	49	DRR 12-0673	4/10/12
B 3.3.18-5	49	DRR 12-0673	4/10/12
B 3.3.18-6	49	DRR 12-0673	4/10/12
B 3.3.18-7	49	DRR 12-0673	4/10/12
B 3.3.19-1	61	DRR 15-0824	4/28/15
B 3.3.19-2	61	DRR 15-0824	4/28/15

LIST OF EFFECTIVE PAGES - TECHNICAL REQUIREMENT MANUAL BASES

PAGE ⁽¹⁾	REVISION NO. ⁽²⁾	CHANGE DOCUMENT ⁽³⁾	DATE EFFECTIVE/ IMPLEMENTED ⁽⁴⁾
TAB – B 3.3 INSTRUMENTATION (continued)			
B 3.3.19-3	61	DRR 15-0824	4/28/15
B 3.3.19-4	61	DRR 15-0824	4/28/15
TAB – B 3.4 REACTOR COOLANT SYSTEM (RCS)			
B 3.4.3-1	10	DRR 02-0122	2/28/02
B 3.4.3-2	10	DRR 02-0122	2/28/02
B 3.4.3-3	41	DRR 10-1702	10/1/10
B 3.4.3-4	31	DRR 007-0657	5/1/07
TAB – B 3.4 REACTOR COOLANT SYSTEM (RCS) (continued)			
B 3.4.10-1	41	DRR 10-1702	10/1/10
B 3.4.10-2	41	DRR 10-1702	10/1/10
B 3.4.16-1	3	DRR 99-1581	12/18/99
B 3.4.16-2	41	DRR 10-1702	10/1/10
B 3.4.16-3	33	DRR 07-1554	9/28/07
B 3.4.17-1	62	DRR 15-2006	10/26/15
B 3.4.17-2	35	DRR 08-0729	8/28/08
B 3.4.17-3	59	DRR 15-0112	2/10/15
B 3.4.17-4	35	DRR 08-0729	8/28/08
B 3.4.17-5	41	DRR 10-1702	10/1/10
B 3.4.17-6	41	DRR 10-1702	10/1/10
B 3.4.17-7	59	DRR 15-0112	2/10/15
B 3.4.17-8	62	DRR 15-2006	10/26/15
B 3.4.17-9	59	DRR 15-0112	2/10/15
B 3.4.18-1	3	DRR 99-1581	12/18/99
B 3.4.18-2	41	DRR 10-1702	10/1/10
B 3.4.18-3	41	DRR 10-1702	10/1/10
TAB – B 3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)			
B 3.5.1-1	3	DRR 99-1581	12/18/99
B 3.5.1-2	41	DRR 10-1702	10/1/10
B 3.5.1-3	3	DRR 99-1581	12/18/99
TAB – B 3.6 CONTAINMENT SYSTEMS			
B 3.6.1-1	62	DRR 15-2006	10/26/15
B 3.6.1-2	41	DRR 10-1702	10/1/10
B 3.6.1-3	62	DRR 15-2006	10/26/15
B 3.6.1-4	62	DRR 15-2006	10/26/15
B 3.6.1-5	37	DRR 09-0287	3/20/09
B 3.6.1-6	17	DRR 04-0452	5/26/04
B 3.6.1-7	62	DRR 15-2006	10/26/15
TAB – B 3.7 PLANT SYSTEMS			
B 3.7.7-1	57	DRR 14-1878	9/30/14
B 3.7.7-2	57	DRR 14-1878	9/30/14
B 3.7.7-3	58	DRR 14-2330	11/6/14
B 3.7.7-4	8	DRR 01-0475	5/1/01
B 3.7.8-1	41	DRR 10-1702	10/1/10

LIST OF EFFECTIVE PAGES - TECHNICAL REQUIREMENT MANUAL BASES

PAGE ⁽¹⁾	REVISION NO. ⁽²⁾	CHANGE DOCUMENT ⁽³⁾	DATE EFFECTIVE/ IMPLEMENTED ⁽⁴⁾
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TAB – B 3.7 PLANT SYSTEMS (continued)

B 3.7.8-2	52	DRR 13-0658	3/28/13
B 3.7.8-3	52	DRR 13-0658	3/28/13
B 3.7.13-1	61	DRR 15-0824	4/28/15
B 3.7.13-2	41	DRR 10-1702	10/1/10
B 3.7.13-3	41	DRR 10-1702	10/1/10
B 3.7.17-1	26	DRR 06-0050	2/28/06
B 3.7.17-2	41	DRR 10-1702	10/1/10
B 3.7.17-3	26	DRR 06-0050	2/28/06
B 3.7.19-1	5	DRR 00-0958	8/17/00
B 3.7.19-2	37	DRR 09-0287	3/20/09
B 3.7.19-3	5	DRR 00-0958	12/18/99
B 3.7.20-1	3	DRR 99-1581	12/18/99
B 3.7.20-2	62	DRR 15-2006	10/26/15
B 3.7.20-3	50	DRR 12-1537	8/30/12
B 3.7.20-4	62	DRR 15-2006	10/26/15
B 3.7.20-5	62	DRR 15-2006	10/26/15
B 3.7.20-6	62	DRR 15-2006	10/26/15
B 3.7.20-7	62	DRR 15-2006	10/26/15
B 3.7.20-8	62	DRR 15-2006	10/26/15
B 3.7.20-9	62	DRR 15-2006	10/26/15
B 3.7.20-10	62	DRR 15-2006	10/26/15
B 3.7.20-11	62	DRR 15-2006	10/26/15
B 3.7.21-1	3	DRR 99-1581	12/18/99
B 3.7.21-2	37	DRR 09-0287	3/20/09
B 3.7.21-3	3	DRR 99-1581	12/18/99
B 3.7.21-4	3	DRR 99-1581	12/18/99
B 3.7.22-1	3	DRR 99-1581	12/18/99
B 3.7.22-2	41	DRR 10-1702	10/1/10
B 3.7.22-3	49	DRR 12-064	4/10/12
B 3.7.24-1	60	DRR 15-0529	3/26/15
B 3.7.24-2	60	DRR 15-0529	3/26/15
B 3.7.24-3	60	DRR 15-0529	3/26/15
B 3.7.24-4	60	DRR 15-0529	3/26/15
B 3.7.24.5	37	DRR 09-0287	3/20/09
B 3.7.24-6	21	DRR 04-1535	12/1/04

TAB – B 3.8 ELECTRICAL POWER SYSTEMS

B 3.8.1-1	60	DRR 15-0529	3/26/15
B 3.8.1-2	63	DRR 16-1952	10/26/16
B 3.8.1-3	60	DRR 15-0529	3/26/15
B 3.8.11-1	3	DRR 99-1581	12/18/99
B 3.8.11-2	41	DRR 10-1702	10/1/10
B 3.8.11-3	41	DRR 10-1702	10/1/10
B 3.8.11-4	54	DRR 13-2529	10/25/13
B 3.8.11-5	41	DRR 10-1702	10/1/10
B 3.8.11-6	41	DRR 10-1702	10/1/10

TAB – B 3.9 REFUELING OPERATIONS

LIST OF EFFECTIVE PAGES - TECHNICAL REQUIREMENT MANUAL BASES

PAGE ⁽¹⁾	REVISION NO. ⁽²⁾	CHANGE DOCUMENT ⁽³⁾	DATE EFFECTIVE/ IMPLEMENTED ⁽⁴⁾
B 3.9.7-1	3	DRR 99-1581	12/18/99
B 3.9.7-2	3	DRR 99-1581	12/18/99
B 3.9.9-1	41	DRR 10-1702	10/1/10
B 3.9.9-2	41	DRR 10-1702	10/1/10
B 3.9.9-3	17	DRR 04-0452	5/26/04

TAB – B 3.10 EXPLOSIVE GAS AND STORAGE TANK RADIOACTIVITY MONITORING

B 3.10.1-1	3	DRR 99-1581	12/18/99
B 3.10.1-2	37	DRR 09-0287	3/20/09
B 3.10.1-3	3	DRR 99-1581	12/18/99
B 3.10.2-1	3	DRR 99-1581	12/18/99
B 3.10.2-2	37	DRR 09-0287	3/20/09
B 3.10.2-3	41	DRR 10-1702	10/1/10
B 3.10.3-1	3	DRR 99-1581	12/18/99
B 3.10.3-2	37	DRR 09-0287	3/20/09
B 3.10.3-3	37	DRR 09-0287	3/20/09
B 3.10.3-4	3	DRR 99-1581	12/18/99

Note 1 The page number is listed on the center of the bottom of each page.

Note 2 The revision number is listed in the lower right hand corner of each page. The Revision number will be page specific.

Note 3 The change document will be the document requesting the change. Therefore, the change document should be a DRR number in accordance with AP 26A-002.

Note 4 The date effective or implemented is the date the Technical Requirement Bases pages are issued by Document Control.