



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 16, 2017

Mr. Charles R. Pierce
Regulatory Affairs Director
Southern Nuclear Operating Co., Inc.
P.O. Box 1295, Bin 038
Birmingham, AL 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1 AND UNIT 2- REQUEST FOR
ADDITIONAL INFORMATION (CAC NOS. MF8910 AND MF8911)

Dear Mr. Pierce:

By letter dated December 7, 2016 (Agencywide Documents Access and Management System (ADAMS) Package Accession No. ML16342C529), Southern Nuclear Operating Company, Inc. (SNC, the licensee) submitted Relief Request FNP-ISI-ALT-20 for the Joseph M. Farley Nuclear Plant (Farley), Units 1 and 2. The licensee requested relief from the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), and proposed to use ASME Code Case N-800 "Alternative Pressure Testing Requirements for Class 1 Piping Between the First and Second Injection Valves, Section XI, Division 1" for performing the system leakage test at or near the end of inspection interval.

NRC Staff has determined that the enclosed Request for Additional Information is needed to complete its review. Please provide responses to these questions within 30 days of the date of this letter.

Sincerely,

A handwritten signature in cursive script that reads "Shawn Williams".

Shawn Williams, Senior Project Manager
Plant Licensing Branch, II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-348, 50-364

Enclosure:
Request for Additional Information

REQUEST FOR ADDITIONAL INFORMATION

RELIEF REQUEST FNP-ISI-ALT-20, VERSION 1.0, REGARDING SYSTEM LEAKAGE TEST

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

DOCKET NOS. 50-348, 50-364

By letter dated December 7, 2016 (Agencywide Documents Access and Management System (ADAMS) Package Accession No. ML16342C529), Southern Nuclear Operating Company, Inc. (SNC, the licensee) requested relief from the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), and proposed to use ASME Code Case N-800 "Alternative Pressure Testing Requirements for Class 1 Piping Between the First and Second Injection Valves, Section XI, Division 1" for performing the system leakage test at or near the end of inspection interval.

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the application and has determined that the following questions are needed to complete its review.

RAI No. 1

On page E-14 of the relief request, the licensee described the method that each piping segment will be pressurized to accommodate the system leakage test of that segment, but did not state what the test pressure will be. Please provide a numerical value of the test pressure for each piping segment.

RAI No. 2

Are there any area(s) of the piping segments (Tables 2 and 3 of relief request) insulated or inaccessible? If yes, please discuss how the VT-2 visual examination will be conducted to ensure any potential leakage in the insulated or inaccessible area(s) will be identified during examination and discuss the appropriate corrective actions that will be taken.

RAI No. 3

Are there any welded connections (e.g., butt, socket) in each piping segment (Tables 2 and 3 of relief request)? If yes,

- a. Discuss whether any of the welds have been examined by volumetric and/or surface examinations during the current 10-year inservice inspection (ISI) interval, and whether any weld(s) is in the risk-informed inservice inspection program and has been or will be examined in the current 10-year ISI interval.

Enclosure

- b. Discuss whether any pressure boundary leakage was identified during the current 10-year ISI interval in each pipe segment under consideration regardless of how the leakage was identified (e.g., from the ASME Code, Section XI, required pressure testing, boric acid corrosion control program walkdowns, reactor restart walkdowns, etc.).

RAI No. 4

In Section titled "Reason for Request," the NRC staff notes that the licensee stated, in part, "*The system pressure requirements of IWC-5221 are an alternative to the system pressure requirements of CC N-800, which are.*" The NRC notes that code cases provide alternatives to the ASME Code, not the reverse. Please clarify or justify this statement.

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1 AND UNIT 2- REQUEST FOR
ADDITIONAL INFORMATION (CAC NOS. MF8910 AND MF8911)

DATED: MARCH 16, 2017.

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***by e-mail**

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