



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 11, 2017

Mr. Daniel Stoddard
President and Chief Nuclear Officer
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Blvd.
Glenn Allen, VA 23060

SUBJECT: SURRY POWER STATION - CORRECTIONS FOR SURRY POWER STATION, UNIT NO 2 – REQUESTS FOR RELIEF LMT-R01, LMT-SS01, LMT-CS01, LMT-P01, LMT-C01, LMT-C02, LMT-C03, AND LMT-C04 – FOR LIMITED COVERAGE EXAMINATIONS PERFORMED IN THE FOURTH 10-YEAR INSERVICE INSPECTION INTERVAL (CAC NOS. MF7718, MF7719, MF7720, MF7721, MF7722, MF7723, MF7724 AND MF7725)

Dear Mr. Stoddard:

By letter dated February, 17, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16365A118), the U.S. Nuclear Regulatory Commission (NRC) issued relief requests (RRs) Nos. LMT-R01, LMT-SS01, LMT-CS01, LMT-P01, LMT-C01, LMT-C02, LMT-C03, and LMT-C04 to Surry Power Station (Surry), Unit 2.

The Relief Requests were in response to your application dated May 5, 2016 (ADAMS Accession No. ML16131A635), as supplemented by letter dated October 27, 2016 (ADAMS Accession No. ML16309A036). Dominion was seeking relief from certain American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) requirements specifically related to ASME Code Case N-460, "Alternative Examination Coverage for Class 1 and Class 2 Welds, Section XI, Division 1," for the fourth 10-year inservice inspection (ISI) interval, which commenced on May 10, 2004 and ended, as extended, on May 9, 2015. The examinations of certain components conducted during the fourth interval received less than the required examination coverage.

By e-mail dated February 28, 2017, the licensee noted that Section 4.0, Conclusion, of the safety evaluation had errors in the Category identifiers C-F-1, C-F-2 for LMT-SS01, LMT-CS01, and B-B and B-D for LMT-C01 and LMT-C02 requests. In particular, the licensee noted that C-F-1 and C-F-2 was stated as class 1 pipes and should have been stated as class 2 pipes and for LMT-SS01 and LMT-CS01 "B" is incorrectly identified as "8" in the summary table for LMT-SS01 and LMT-CS01 requests. These errors on the part of the NRC are purely administration in nature and do not affect the NRC staff's overall conclusion associated with the relief request.

Enclosed is the corrected safety evaluation page 32 to replace the conclusion page issued with the relief requests. The areas of change are marked with revision bars in the margin. We regret any inconvenience this may have caused.

D. Stoddard

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If you have any questions regarding this matter, I may be reached at (301) 415-1438 or via e-mail at karen.cotton@nrc.gov.

Sincerely,

Handwritten signature of Karen Cotton Gross in cursive script.

Karen Cotton Gross, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-281

Enclosure:
As stated

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as limited in their historical inservice inspection databases and a linear indication resulted in the only required repair which was completed in 1988.

The NRC staff determined that the licensee adequately demonstrated that obtaining essentially 100 percent of the ASME Code-required examination is impractical for the main steam integral attachment welds. Furthermore, based on the above, the NRC staff concludes that there is reasonable assurance that the structural integrity and leak tightness of the welds will be maintained by the examination coverage achieved.

4.0 CONCLUSION

As set forth above, the staff determined that granting relief pursuant to 10 CFR 50.55a(g)(6)(i) is authorized by law and will not endanger life or property, or the common defense and security, and is otherwise in the public interest given due consideration to the burden upon the licensee that could result if the requirements were imposed on the facility. Furthermore, the staff concludes that the licensee's examinations were performed to the extent possible and provide reasonable assurance of the structural integrity of the subject pressurizer welds and nozzle inside radius, and piping integral welded attachments at Surry, Unit 2. Accordingly, the staff concludes that the licensee has adequately addressed all of the regulatory requirements set forth in 10 CFR 50.55a(g)(6)(i). Therefore, the NRC staff grants the following relief requests for the fourth ten-year ISI interval of Surry, Unit 2, which commenced May 10, 2004, and ended, as extended, on May 9, 2015.

LMT-R01	Category R-A, Class 1 Stainless Steel Pipe Risk Informed Welds
LMT-SS01	Category C-F-1 , Class 2 Stainless Steel Pipe Welds
LMT-CS01	Category C-F-2 , Class 2 Carbon Steel Pipe Welds
LMT-P01	Category C-F-1 and R-A Preservice Pipe Welds
LMT-C01	Category B-B, Pressurizer Shell to Head Section
LMT-C02	Category B-D, Pressurizer Nozzle Inner Radius Section
LMT-C03	Category C-C, Main Steam Integral Attachment H001-1
LMT-C04	Category C-C, Main Steam Integral Attachment H001-2

All other requirements of Section XI of the ASME Code for which relief was not specifically requested and approved in the subject RRs remain applicable, including third party review by the Authorized Nuclear Inservice Inspector.

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