

## UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 17, 1999

Mr. John H. Mueller Chief Nuclear Officer Niagara Mohawk Power Corporation Nine Mile Point Nuclear Station Operations Building, Second Floor Lycoming, NY 13093

SUBJECT: GENERAL VALVE RELIEF REQUEST-01 FOR THE SECOND INTERVAL

INSERVICE PUMP AND VALVE TESTING PROGRAM PLAN, NINE MILE POINT

**NUCLEAR STATION, UNIT 2 (TAC NO. MA2148)** 

Dear Mr. Mueller:

By letter dated June 16, 1998, Niagara Mohawk Power Corporation (NMPC) submitted its second ten-year Inservice Testing (IST) Program Plan for pumps and valves at Nine Mile Point Nuclear Station, Unit 2 (NMP2). Section III of the IST Program Plan (pages 36-39) includes a general relief request. GVRR-01, for series check valves without intermediate test connections for which you find the requirements of Part 10 of the American National Standards Institute/American Society of Mechanical Engineers (ASME) "Operations and Maintenance Standards" (OM-10) to be impractical. The series check valves in this configuration are system pressure pump discharge check valves (2 High Pressure Core Spray (CSH)\*P2 to CSH Header 2CSH\*V17, \*V55; 2 Low Pressure Core Spray (CSL) \*P2 to CSL Header 2CSL\*V14, \*V21; 2 Residual Heat Removal (RHS) \*V47, \*V48; 2RHS\*P2 to valves 2RHS\*V60, \*V61, and \*V17, \*V18). The function of these valves is to prevent diversion of emergency core cooling system (ECCS) flow by preventing reverse flow from the ECCS discharge path into the associated pressure pump discharge piping. NMPC tests these valves in accordance with the guidance in NUREG-1482, "Guidelines for Inservice Testing at Nuclear Power Plants," paragraph 4.1.1, "Closure Verification For Series Check Valves Without Intermediate Test Connections," and you request relief consistent with the NRC Recommendation in that paragraph. NMPC proposes to test each pair of series check valves quarterly as a unit for reverse flow closure and to apply the test results of each set to both valves of the set.

The NRC staff has reviewed the proposed GVRR-01 against the requirements of the 1989 Edition of Section XI of the ASME Boiler and Pressure Vessel Code (ASME Code) pursuant to 10 CFR 50.55a. Enclosed is the NRC staff's safety evaluation which concludes that imposition of the ASME Code requirements would result in hardship without a compensating increase in the level of quality and safety, and authorizes GVRR-01 pursuant to 10 CFR 50.55a(a)(3)(ii) on the basis that NMPC's proposed alternatives provide reasonable assurance of operational readiness. This NRC staff approval applies to the second ten-year IST interval for the above-specified NMP2 valves.

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If you have any questions regarding this matter, please contact Darl Hood by phone on (301) 415-3049 or by electronic mail at dsh@nrc.gov.

Sincerely,

S. Singh Bajwa, Director Project Directorate I-1

Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-410

Enclosure: Safety Evaluation

cc w/encl: See next page



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Sincerely,

Original signed by:

S. Singh Bajwa, Director
Project Directorate I-1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

**Docket No. 50-410** 

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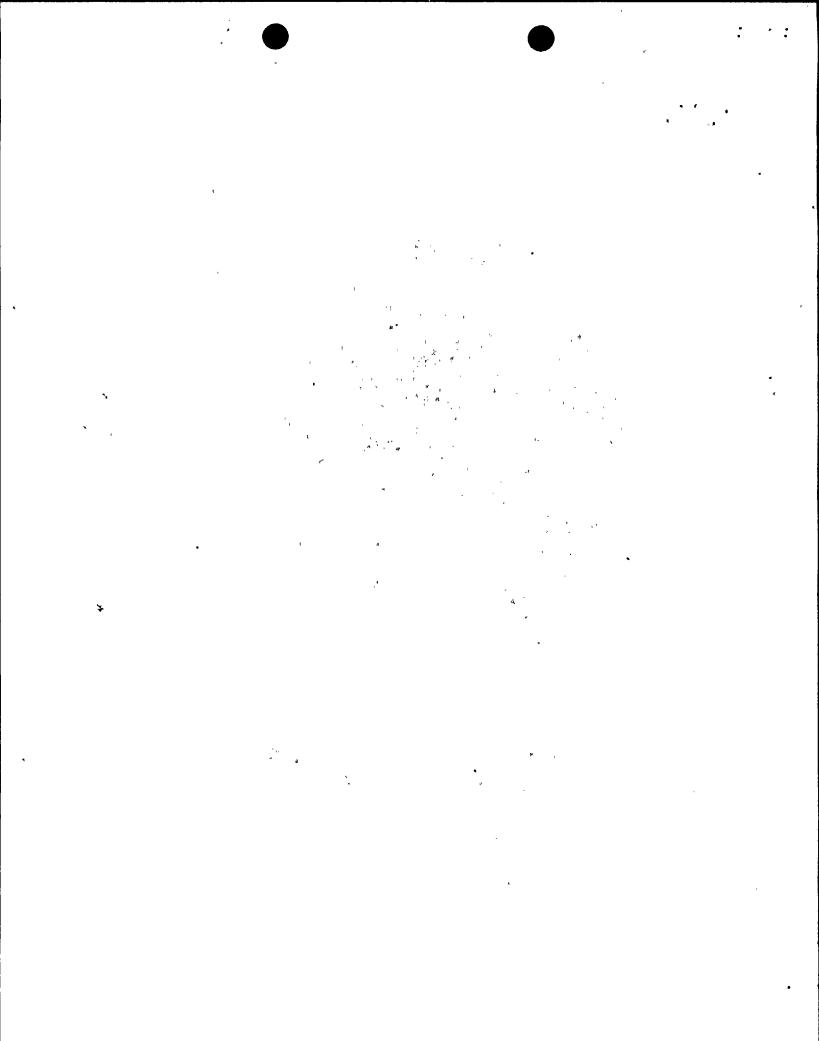
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John H. Mueller ⊮agara Mohawk Power Corporation

CC:

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