

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9710240189 DOC. DATE: ~~97/09/30~~ NOTARIZED: NO DOCKET #
FACIL: 50-410 Nine Mile Point Nuclear Station, Unit 2, Niagara Moha 05000410
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DAHLBERG, K.A. Niagara Mohawk Power Corp.
RECIP: NAME RECIPIENT AFFILIATION

SUBJECT: Monthly operating rept for Sept 1997 for Nine Mile Point
Unit 2. W/971015 ltr.

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	OC/DBA/PAD		1	1	RGN1	1	1	
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October 15, 1997
NMP2L 1731

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

RE: Nine Mile Point Unit 2
Docket No. 50-410
NPF-69

Subject: Operating Statistics, Unit Shutdowns and Power Reductions for September 1997

Dear Sir:

Submitted herewith is the Report of Operating Statistics, the Unit Shutdown and Power Reductions Summary, and a Narrative Report of Operational Experience for September 1997.

Very truly yours,



K. A. Dahlberg
Plant Manager - Unit 2

/ct

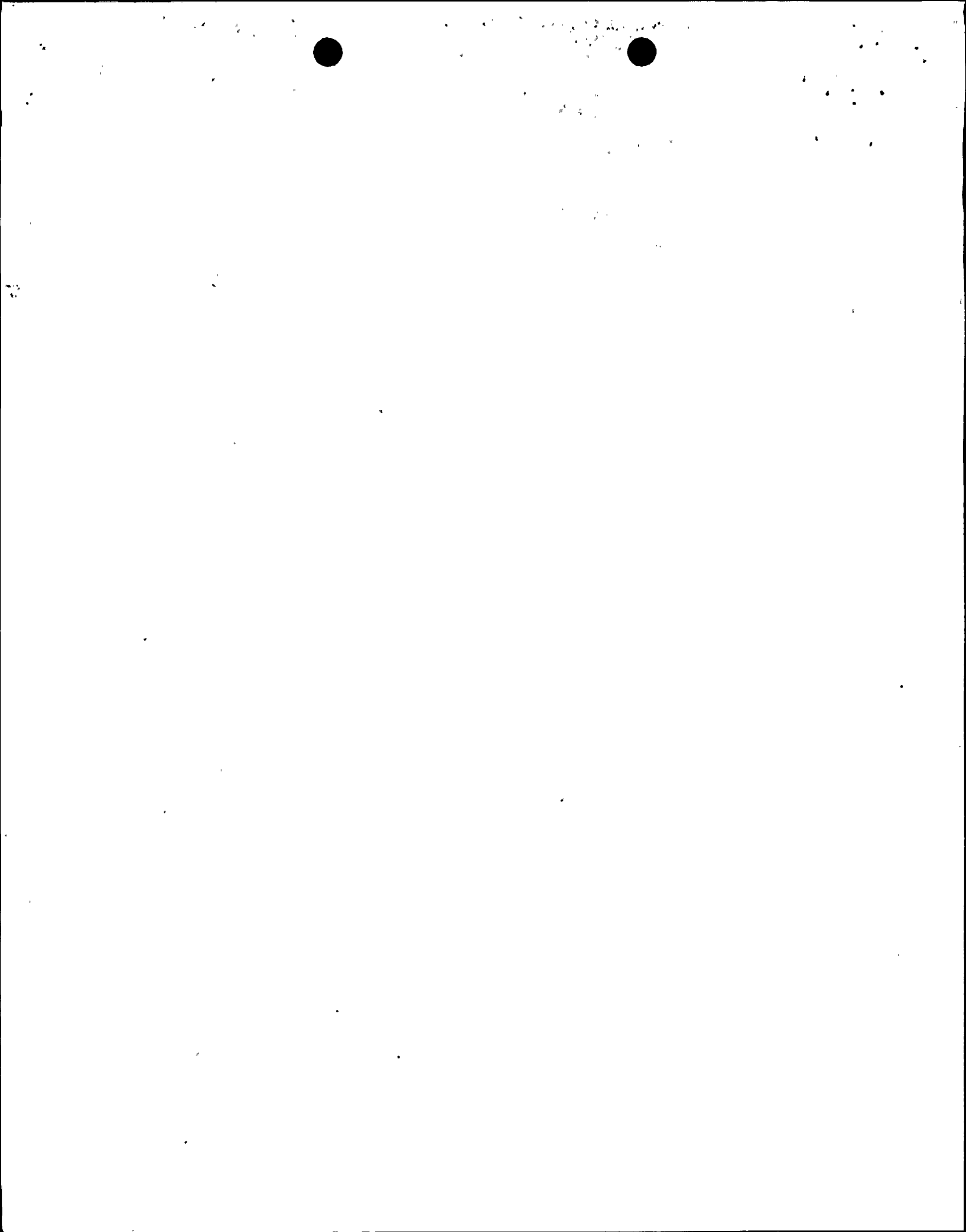
Enclosures

pc: H.J. Miller, Regional Administrator, Region 1
B.S. Norris, Senior Resident Inspector

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NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION UNIT #2
NARRATIVE OF OPERATING EXPERIENCE

Nine Mile Point Unit Two operated with a capacity factor of 93.38% MDC and an availability factor of 100.00% for the month of September 1997.

Nine Mile Point Unit Two remains restricted to 95% core thermal power due to removing both "A" and "B" MSRs from service in June, 1997. (See June 1997 narrative)

On September 7, 1997 at 0700 hours, reactor core thermal power was reduced to 55% in preparation for a feedwater pump swap. The pump swap was completed and power ascension back to 95% core thermal power began at 1503 hours. At 2200 hours with core thermal power at 92%, the control room received a Hi/Hi level alarm on the 5th point "C" feedwater heater which caused a loss of extraction heating steam to the "C" 5th point heater. Core thermal power was reduced to 56% while Instrument and Control personnel were dispatched to investigate. It was determined that the air relay on the valve positioner for 2ESS-LV5C had failed. The relay was replaced and the heater was restored to service. On September 8, 1997 at 1434 hours preparations to return to 95% core thermal power were being made. During the briefing for reactivity changes, the rod drive control system alarmed. An operator was dispatched to monitor the control rod hydraulic control units while Instrument and Control personnel investigated. It was determined that the cause of the alarm was due to a faulty transponder card, which was replaced, and the rod drive control system was reset. Operations was again briefed on reactivity changes for the rod pattern adjustments. Control rod pattern adjustments and power ascension began at 2223 hours. On September 9, 1997 at 1800 hours, while Operations was preparing to make the final recirculation flow adjustment from 94% core thermal power to 95% power, the chart recorder for the reactor recirculation control system on valve 2RCS-FCV "A" malfunctioned. Operations was unable to determine from visual observations if the chart recorder or the valve was malfunctioning. Instrument and Control personnel were dispatched to the valve to troubleshoot. The investigation determined that the chart recorder was stuck and the valve was operating properly. Power maneuvers were halted until the chart recorder was placed back in service at 2155 hours. At this time, Reactor Engineering determined that the original rod pattern that they were coming up on was no longer valid. This was due to the magnitude and duration of the power reduction for the 5th point feedwater heater transient. The reactor core was experiencing a normal xenon transient, and a new rod pattern had to be established to reach the 95% target rod pattern. To accomplish this, core thermal power was reduced to 80%, where the new rod pattern was implemented to achieve 95% power. On September 10, 1997 at 0726 hours, reactor core thermal power was restored to the maximum allowable power of 95% of rated core thermal power.

There were no challenges to the safety relief valves during this report period.



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OPERATING DATA REPORT

DOCKET NO.: 50-410

DATE: 10/02/97

PREPARED BY: C. Caroccio

TELEPHONE: (315) 349-4615

OPERATING STATUS

- 1. Unit Name: Nine Mile Point Unit #2
- 2. Reporting Period: SEPTEMBER 1997
- 3. Licensed Thermal Power (MWt): 3467
- 4. Nameplate Rating (Gross MWe): 1259
- 5. Design Electrical Rating (Net MWe): 1143
- 6. Maximum Dependable Capacity (Gross MWe): 1169.67
- 7. Maximum Dependable Capacity (Net MWe): 1105.44
- 8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reason: None.
- 9. Power Level To Which Restricted, If Any (Net Mwe): 95% (Approx. 1086)
- 10. Reasons For Restrictions, If Any: Unit running with both "A" and "B" Moisture Separator Reheaters out-of-service.

Items 21 and 22 Cum. are weighted values.

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	720.00	6,551.00	83,184.00
12. Number of Hours Reactor was Critical	720.00	6,299.18	62,598.23
13. Reactor Reserve Shutdown Hours	0.00	0.00	0.00
14. Hours Generator On-Line	720.00	6,225.02	60,311.60
15. Unit Reserve Shutdown Hours	0.00	0.00	12.98
16. Gross Thermal Energy Generated (MWH)	2,322,559.78	20,691,899.23	193,107,584.79
17. Gross Electrical Energy Generated (MWH)	788,139.89	7,115,372.80	64,714,804.17
18. Net Electrical Energy Gen. (MWH)	743,235.03	6,724,115.79	60,954,624.17
19. Unit Service Factor	100.00%	95.02%	72.50%
20. Unit Availability Factor	100.00%	95.02%	72.52%
21. Unit Capacity Factor (Using MDC Net)	93.38%	92.85%	69.20%
22. Unit Capacity Factor (Using DER Net)	90.31%	89.80%	67.01%
23. Unit Forced Outage Rate	0.00%	2.28%	12.10%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):
None.

25. If Shut Down At End of Report Period, Estimated Date of Startup:

26. Unit in Test Status (Prior to Commercial Operation):

	<u>Forecast</u>	<u>Achieved</u>
INITIAL CRITICALITY		05/23/87
INITIAL ELECTRICITY		08/08/87
COMMERCIAL OPERATION		04/05/88



APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.: 50-410
UNIT: NMP2
DATE: 10/02/97
PREPARED BY: C. Caroccio
TELEPHONE: (315) 349-4615

MONTH SEPTEMBER 1997

DAY	AVERAGE DAILY POWER LEVEL (Mwe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1050	17	1052
2	1047	18	1052
3	1059	19	1049
4	1059	20	1051
5	1055	21	1061
6	1051	22	1061
7	883	23	1058
8	673	24	1063
9	946	25	1059
10	1033	26	1061
11	1050	27	1062
12	1052	28	1057
13	1051	29	1055
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16	1056		



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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-410

UNIT NAME: NMP#2

DATE: 10/02/97

REPORT MONTH - SEPTEMBER 1997

PREPARED BY: C. Caroccio

TELEPHONE: (315) 349-4615

No.	Date	Type ¹	Duration (Hours)	Reasons ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
97-11	970907	S	0	B	4	N/A	FWS	2FWS-P1C	Reactor Core Thermal Power reduced to 55% for feedwater pump swap.
97-12	970907	F	0	A	4	N/A	ESS	2ESS-LVSC	Air relay on valve positioner failed. Relay replaced, valve returned to service.
97-13	970908	F	0	A	4	N/A	RCS	2RMC-TCB-R308	Valve transponder card failed, replaced card, reset rod control system.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Exam
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

⁵
Exhibit I-Same Source



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