

CATEGORY 1

REGULAR INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9707240089 DOC.DATE: 97/07/14 NOTARIZED: NO DOCKET #
 FACIL:50-220 Nine Mile Point Nuclear Station, Unit 1, Niagara Powe 05000220
 AUTH.NAME AUTHOR AFFILIATION
 DOTY,S.T. Niagara Mohawk Power Corp.
 ABBOTT,R.B. Niagara Mohawk Power Corp.
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 97-006-00:on 970613,NMP Unit 1 lost ability to monitor
 RCS leakage via DWL floor drain monitoring sys.Caused by
 inadequate foreign matl exclusion.DWL FDS was pumped out &
 cleaned.W/970714 ltr.

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NIAGARA MOHAWK

GENERATION
BUSINESS GROUP

NINE MILE POINT NUCLEAR STATION/LAKE ROAD, P.O. BOX 63, LYCOMING, NEW YORK 13093

July 14, 1997
NMP1L 1235

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

RE: LER 97-06
Docket No. 50-220

Gentlemen:

In accordance with 10CFR50.73 (a)(2)(i)(A), we are submitting LER 97-006, "Technical Specification Required Shutdown Because of Loss of Reactor Coolant System Leakage Monitoring Ability."

Very truly yours,

Richard B. Abbott
Plant Manager - NMP1

RBA/TRE/lmc
Enclosure

xc: Mr. H. J. Miller, Regional Administrator
Mr. B. S. Norris, Senior Resident Inspector
Records Management

IEDH

9707240089 970714
PDR ADDCK 05000220
S PDR



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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)

Nine Mile Point Unit 1

DOCKET NUMBER (2)

05000220

PAGE (3)

1 OF 4

TITLE (4)

Technical Specification Required Shutdown Because of Loss of Drywell Floor Drain Leakage Monitoring Ability

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)	
06	13	97	97	006	00	07	14	97	N/A	05000	
									N/A	05000	

OPERATING MODE (9)

N

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

POWER LEVEL (10) 100	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	<input type="checkbox"/> OTHER
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	(Specify in Abstract below and in Text, NRC Form 366A)
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
	<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
S. T. Doty - NMP1 Maintenance Manager	(315) 349-4594

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	IJ	ISV							

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

YES (If yes, complete EXPECTED SUBMISSION DATE)

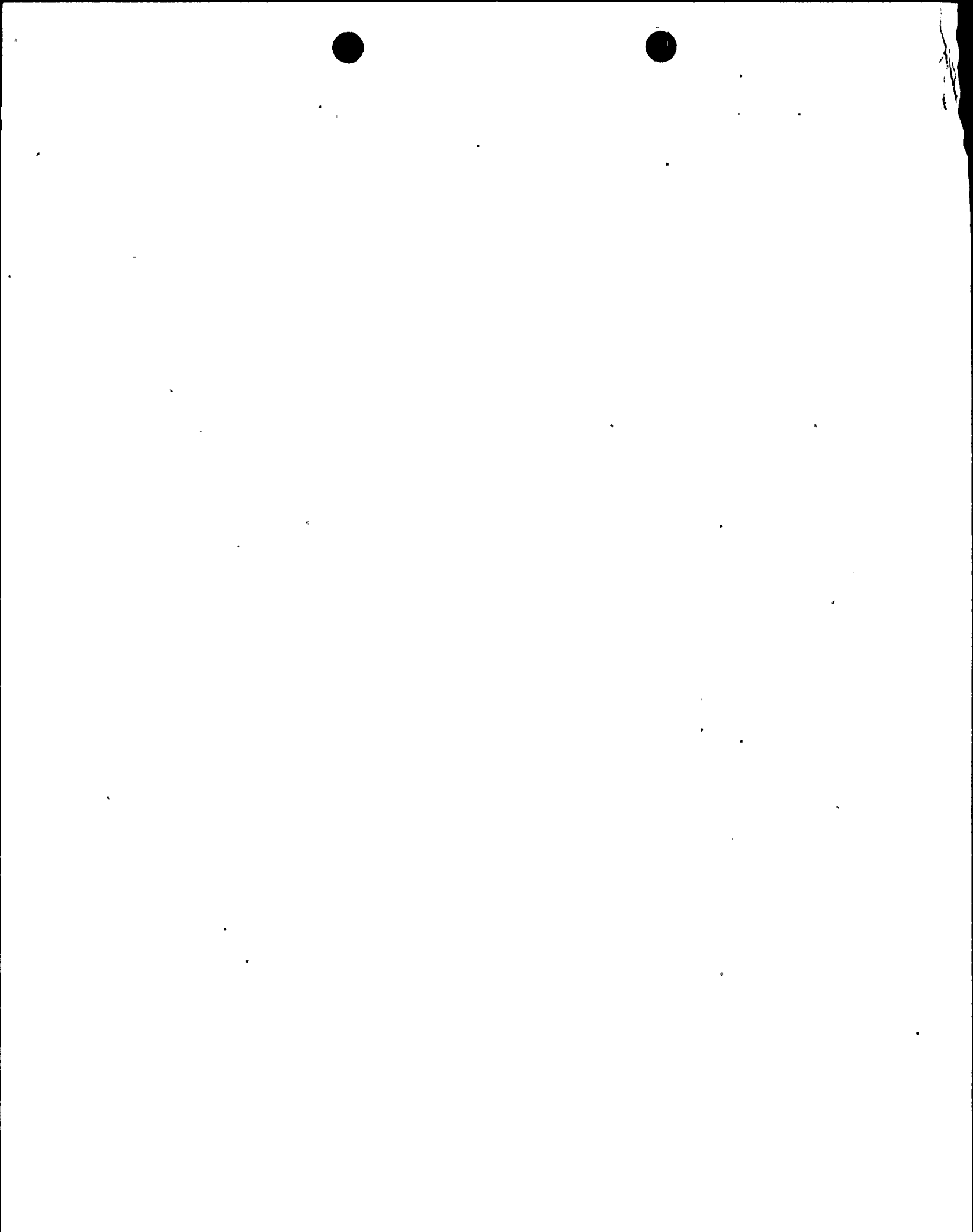
NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On June 13, 1997, at 0755 hours, with Nine Mile Point Unit 1 (NMP1) in the RUN mode and reactor thermal power at approximately 100 percent, Niagara Mohawk Power Corporation (NMPC) lost the ability to monitor the reactor coolant system leakage via the Drywell Floor Drain Monitoring System. This required entry into the action statement for Technical Specification (TS) 3.2.5, which required the reactor to be in cold shutdown within 24 hours. At 0718 hours on June 14, 1997, NMP1 entered the cold shutdown condition.

The immediate cause of the loss of the ability to monitor the reactor coolant system leakage via the Drywell Floor Drain Monitoring System was the securing of the drywell floor drain outboard isolation valve in accordance with TS 3.3.4b after the inboard isolation valve was declared inoperable.

NMPC repaired the inoperable inboard isolation valve and restarted NMP1 on June 20, 1997.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Nine Mile Point Unit 1	05000220	97	06	00	02 OF 04

TEXT (If more space is required, use additional NRC Form 366A's) (17)

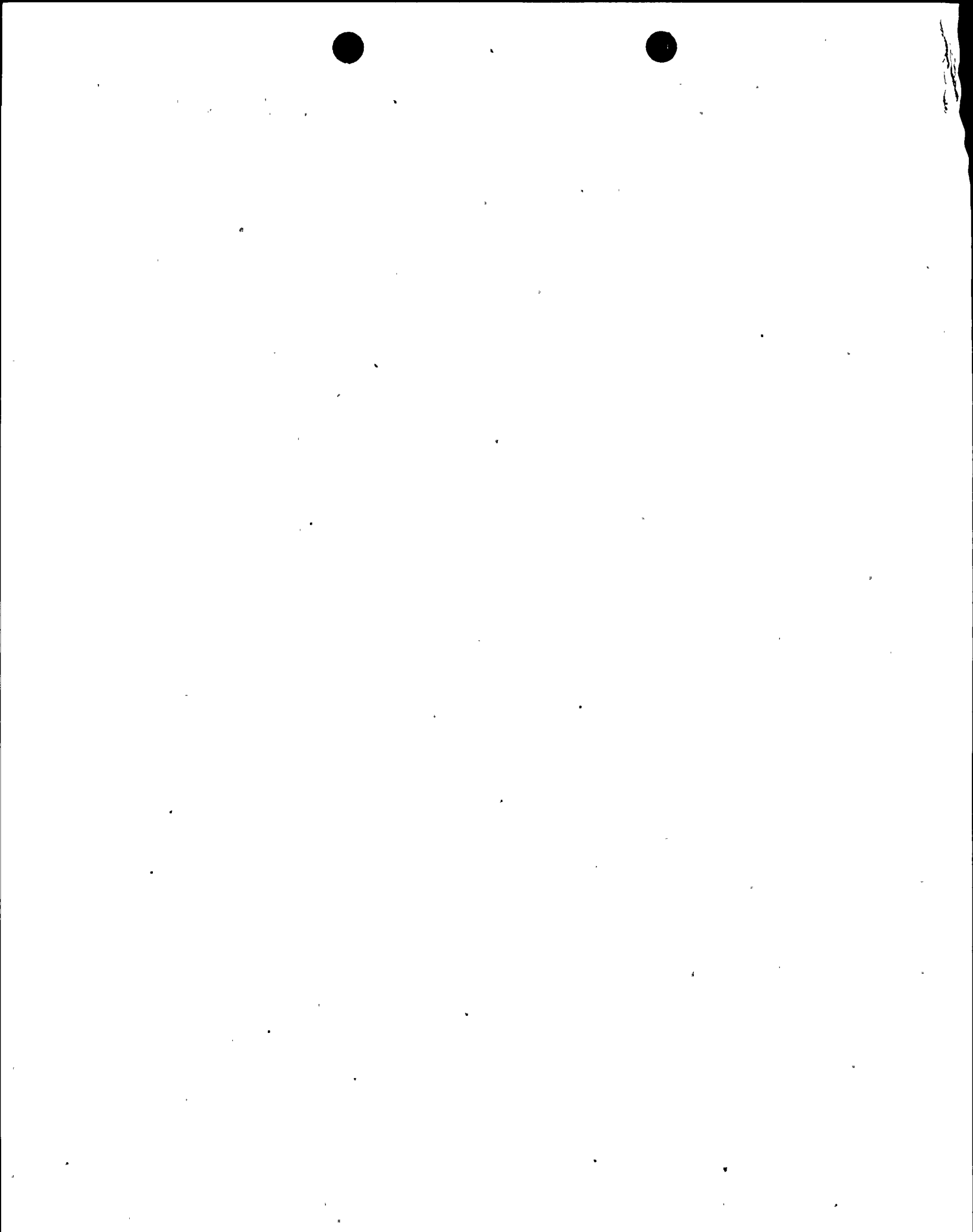
I. DESCRIPTION OF EVENT

On June 13, 1997, at 0755 hours, with Nine Mile Point Unit 1 (NMP1) in the RUN mode and reactor thermal power at approximately 100 percent, Niagara Mohawk Power Corporation (NMPC) lost the ability to monitor the reactor coolant system leakage via the Drywell Floor Drain Monitoring System. This required entry into the action statement for Technical Specification (TS) 3.2.5, which required the reactor to be in cold shutdown within 24 hours. At 0718 hours on June 14, 1997, NMP1 entered the cold shutdown condition.

The loss of the ability to monitor the reactor coolant system leakage via the Drywell Floor Drain Monitoring System resulted from the closure of the drywell floor drain outboard isolation valve after the inboard isolation valve was declared inoperable. At 2300 hours on June 12, 1997, during performance of procedure N1-ST-Q5, drywell floor drain inboard isolation valve 83.1-11 failed its closure time since the valve exhibited dual indication in the control room when the valve was cycled closed. The operators declared isolation valve 83.1-11 inoperable and entered a 4 hour action statement per TS 3.3.4b. At 0240 hours on June 13, 1997, drywell floor drain outboard isolation valve 83.1-12, which had passed its stroke test on June 12, 1997, was closed and a yellow hold-out tag was placed on its control switch, thus satisfying the 4 hour action statement per 3.3.4b. Closure of valve 83.1-12 prevented draining of the drywell floor drain sump, and resulted in high level in the sump, and consequently, loss of leak rate measurement capability. The leak rate measurement is determined by the rate of level increase in the sump. Once the sump filled, the instrumentation could no longer measure this rate of increase. This loss of leak rate measurement capability at 0755 hours on June 13, 1997, prompted entry into the action statement of TS 3.2.5, which requires the reactor to be shutdown within 24 hours. Reactor power was reduced to approximately 19 percent power and at 0231 hours on June 14, a manual scram was inserted. All control rods fully inserted and plant systems responded as expected.

II. CAUSE OF EVENT

The root cause of the failure of drywell floor drain inboard isolation valve 83.1-11 is inadequate foreign material exclusion caused by a failure to consider the impact on the floor drain system when flushing the Drywell elevation 225'. Sludge and metal filings from the sump were found in the valve when it was disassembled and cleaned. This material prevented the valve from closing and resulted in the valve being declared inoperable. There were no records found that indicate this sump has ever been cleaned.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATIONESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION
REQUEST: 30.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE
RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY
COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT
(3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Nine Mile Point Unit 1	05000220	97	06	00	03 OF 04

TEXT (If more space is required, use additional NRC Form 366A's) (17)

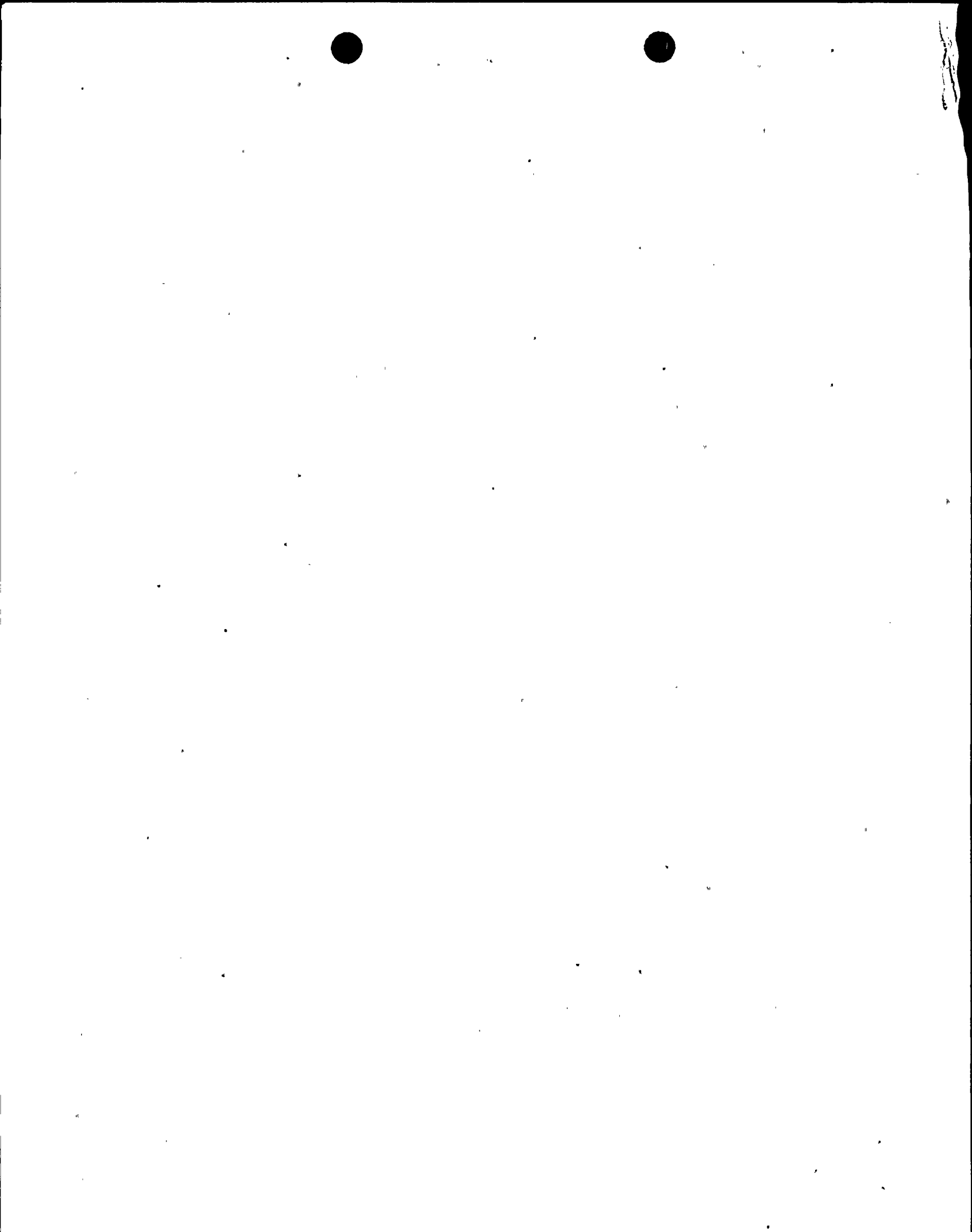
III. ANALYSIS OF EVENT

This event is reportable in accordance with 10CFR50.73 (a)(2)(i)(A), which requires licensees to report "the completion of any nuclear plant shutdown required by the plant's Technical Specifications."

There were no safety consequences as a result of this event. The outboard isolation valve 83.1-12 was closed in accordance with TS (satisfying containment isolation requirements) and the reactor shutdown posed no threat to the health and safety of the general public or plant personnel. The loss of the ability to monitor the reactor coolant system leakage via the Drywell Floor Drain Monitoring System does not affect the operability of other safety-related equipment.

IV. CORRECTIVE ACTIONS

1. The Drywell Floor Drain Sump was pumped out and cleaned; the Drywell Floor-Drain discharge piping, from the outboard isolation valve to the sump, was backflushed.
2. Maintenance disassembled, cleaned, reassembled and tested isolation valves 83.1-11 and 83.1-12.
3. Plant procedures will be revised to require the installation and removal of temporary screens around the drywell floor drains during refueling or other outage periods commensurate with work activities. This will be completed by August 29, 1997.
4. A task has been established in the Preventive Maintenance Surveillance Testing (PM/ST) database to require inspections, and cleaning as appropriate, of the Drywell Floor Drain Sump every other refueling outage. The frequency of these inspections will be evaluated based on inspection results.
5. A review of this event will be incorporated in the General Employee Training (GET) Program and the GET Requalification Program. The emphasis will be on maintaining an awareness of system interactions and the impact of foreign material exclusion on plant operations. This will be completed by December 31, 1997.
6. Plant procedures will be revised to address foreign material exclusion with regards to open systems in the plant, and the need to consider the potential impact of foreign material on these systems. This will be completed by September 30, 1997.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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FACILITY NAME (1) Nine Mile Point Unit 1	DOCKET NUMBER (2) 05000220	LER NUMBER (6)			PAGE (3) 04 OF 04
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
		97	- 06	- 00	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

V. ADDITIONAL INFORMATION

- A. Failed components: IV 83.1-11
- B. Previous similar events: none.
- C. Identification of components referred to in this LER:

COMPONENT	IEEE 803 FUNCTION	IEEE 805 SYSTEM ID
83.1-11	ISV	IJ
83.1-12	ISV	IJ

