

CATEGORY 1

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RECIP: NAME RECIPIENT AFFILIATION

SUBJECT: Monthly operating rept for May 1997 for Nine Mile Point Unit
2.8/970616 ltr.

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NIAGARA MOHAWK

GENERATION
BUSINESS GROUP

NINE MILE POINT NUCLEAR STATION/LAKE ROAD, P.O. BOX 63, LYCOMING, NEW YORK 13093

June 16, 1997
NMP2L 1712

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

RE: Nine Mile Point Unit 2
Docket No. 50-410
NPF-69

Subject: Operating Statistics, Unit Shutdowns and Power Reductions for May 1997

Dear Sir:

Submitted herewith is the Report of Operating Statistics, the Unit Shutdown and Power Reductions Summary, and a Narrative Report of Operational Experience for May 1997.

Very truly yours,

J. T. Conway
Plant Manager - Unit 2

/ct

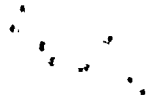
Enclosures

pc: H.J. Miller, Regional Administrator, Region 1
B.S. Norris, Senior Resident Inspector

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NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT #2

NARRATIVE OF OPERATING EXPERIENCE

Nine Mile Point Unit Two operated with a capacity factor of 99.58% MDC and an availability factor of 100% for the month of May 1997.

On May 3, 1997 at 0000 hours, reactor core thermal power was reduced to 55% where operations performed control rod pattern adjustments and swapped reactor feedwater pumps. On May 4, 1997 at 0430 hours, reactor core thermal power was returned to 100% of rated.

On May 26, 1997 at 0945 hours, while operating at 100% of rated core thermal power, Nine Mile Point Unit Two experienced level control problems in the "A" Reheater (DSR) drain tank. Drain tank level controller, 2DSR-LC65A, was causing Reheater drain tank low level alarms and high/high level alarms in the sixth and fifth point feedwater heaters. Operations stabilized the plant by placing level control valve 2DSR-LVX65A in manual and reducing the demand signal to the level control valve. During followup troubleshooting, the controllers for the "A" Reheater drain tank level control valves were placed in manual which resulted in level alarms in the feedwater heater strings. A high/high level in "A" Reheater drain tank occurred which should have isolated both 2MSS-AOV92A and B, (steam supply valves to the Moisture Separator Reheaters (MSRs)) but only isolated the "B" valve. As the 2MSS-AOV92B valve started to close, the Reheater drain tank high/high level alarm cleared. The SSS, after reviewing instrumentation, ordered the 2MSS-AOV92B open. Following this action, it was noticed that there was a mismatch in steam flow between the "A" and "B" MSRs. At 1200 hours, reactor core thermal power was reduced to 87%. Plans were made to remove both MSRs from service to investigate. While securing the MSRs from service, another Reheater Drain Tank Hi Hi Level occurred resulting in isolation of the MSRs. A work plan was developed to investigate the reheater problems while reactor core thermal power was returned to 95% of rated .

On May 31, 1997 at 0000 hours, Nine Mile Point Unit Two started to reduce reactor core thermal power to 55% to prepare for Moisture Separator Reheater boroscope inspections. At the end of this report period Nine Mile Point Unit Two had reduced reactor core thermal power to 55% due to ALARA considerations, where troubleshooting of the Moisture Separator Reheaters was to take place. A contingency Maintenance outage was planned to begin June 1, 1997, dependent upon the results of the evaluation of the MSR.

Other causes of minor power reductions during the month of May 1997 were a result of control rod adjustments. There were no challenges to the safety relief valves during this report period.



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OPERATING DATA REPORT

DOCKET NO.: 50-410

DATE: 06/05/97

PREPARED BY: C. Caroccio

TELEPHONE: (315) 349-4615

OPERATING STATUS

- 1. Unit Name: Nine Mile Point Unit #2
- 2. Reporting Period: MAY 1997
- 3. Licensed Thermal Power (MWt): 3467
- 4. Nameplate Rating (Gross MWe): 1259
- 5. Design Electrical Rating (Net MWe): 1143
- 6. Maximum Dependable Capacity (Gross MWe): 1169.67
- 7. Maximum Dependable Capacity (Net MWe): 1105.44
- 8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reason: None.
- 9. Power Level To Which Restricted, If Any (Net MWe): None.
- 10. Reasons For Restrictions, If Any: None.

Items 21 and 22 Cum. are weighted values.

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	744.00	3,648.00	80,281.00
12. Number of Hours Reactor was Critical	744.00	3,648.00	59,947.05
13. Reactor Reserve Shutdown Hours	0.00	0.00	0.00
14. Hours Generator On-Line	744.00	3,648.00	57,734.58
15. Unit Reserve Shutdown Hours	0.00	0.00	12.98
16. Gross Thermal Energy Generated (MWH)	2,499,548.40	12,393,044.88	184,808,730.44
17. Gross Electrical Energy Generated (MWH)	865,525.20	4,312,314.96	61,911,746.33
18. Net Electrical Energy Gen. (MWH)	819,024.43	4,081,348.00	58,311,856.90
19. Unit Service Factor	100.00%	100.00%	71.92%
20. Unit Availability Factor	100.00%	100.00%	71.93%
21. Unit Capacity Factor (Using MDC Net)	99.58%	101.21%	68.72%
22. Unit Capacity Factor (Using DER Net)	96.31%	97.88%	66.55%
23. Unit Forced Outage Rate	0.00%	0.00%	12.19%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):

Scheduled Maintenance Outage, June 1, 1997 to June 7, 1997.

25. If Shut Down At End of Report Period, Estimated Date of Startup:

26. Unit in Test Status (Prior to Commercial Operation):

	<u>Forecast</u>	<u>Achieved</u>
INITIAL CRITICALITY		05/23/87
INITIAL ELECTRICITY		08/08/87
COMMERCIAL OPERATION		04/05/88



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APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.: 50-410
UNIT: NMP2
DATE: 06/05/97
PREPARED BY: C. Caroccio
TELEPHONE: (315) 349-4615

MONTH MAY 1997

DAY	AVERAGE DAILY POWER LEVEL (Mwe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1136	17	1143
2	1137	18	1142
3	768	19	1137
4	1134	20	1141
5	1141	21	1141
6	1143	22	1141
7	1145	23	1142
8	1144	24	1140
9	1141	25	1140
10	1143	26	1088
11	1140	27	1071
12	1137	28	1066
13	1142	29	1061
14	1141	30	1062
15	1140	31	639
16	1142		



UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-410

UNIT NAME: NMP#2

DATE: 06/05/97

REPORT MONTH - MAY 1997

PREPARED BY: C. Carocco

TELEPHONE: (315) 349-4615

No.	Date	Type ¹	Duration (Hours)	Reasons ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
97-04	970503	S	0	B	4	NA	FWS	2FWS-P1C	Reactor Core Thermal Power reduced to 55% for feedwater pump swap.
97-05	970526	S	0	B	4	NA			Reactor Core Thermal Power reduced to 87% to investigate Moisture Separator Reheater Drain Tank Level control problems.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Exam
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

⁵
Exhibit I-Same Source

