

## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

50-220

March 5, 1997

Mr. B. Ralph Sylvia Executive Vice President, Generation Business Group and Chief Nuclear Officer Niagara Mohawk Power Corporation Nuclear Learning Center 450 Lake Road Oswego, NY 13126

SUBJECT: FOUR RELIEF REQUESTS FOR VALVE INSERVICE TESTING PROGRAM -NINE MILE POINT UNIT 1 (TAC NO. M96337)

Dear Mr. Sylvia:

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PDR

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PDR

By letter dated August 9, 1996, as supplemented February 28, 1997, Niagara Mohawk Power Corporation proposed revisions to four relief requests for Nine Mile Point Nuclear Station, Unit 1 (NMP-1), with respect to the second tenyear interval program for inservice testing of valves. The four requests are identified as: VG-2, Revision 1; CS-RR-6, Revision 1; LP-RR-2, Revision 1; and CRD-RR-3, Revision 1. The existing version of these relief requests were previously by the NRC staff in safety evaluations dated March 7, 1991 (Relief Requests VG-2, LP-RR-2, and CRD-RR-3), and July 26, 1995 (Relief Request CS-RR-6).

As described in the enclosed safety evaluation (SE), the NRC staff has completed its review of the your submittals and concludes that:

- 1. The Code requirement for Relief Request CS-RR-6, Revision 1, is impractical to perform and the relief is granted as requested pursuant to 10 CFR 50.55a(f)(6)(i).
- 2. For Relief Requests LP-RR-2, Revision 1, and CRD-RR-3, Revision 1, your proposed alternative to defer verification of reverse flow closure to refueling outages is approved pursuant to 10 CFR 50.55a(f)(4)(iv) based upon your letter dated February 28, 1997, which proposes to meet the requirements of Paragraphs 4.3.2, and 6.2 of OM-10 and reference Section 3.1.1.3 of NUREG-1482 in the IST program.
- 3. For Relief Request VG-2, Revision 1, your proposed alternative to use Option B to 10 CFR 50, Appendix J for leakage-rate testing of LJ containment isolation valves is approved pursuant to the provisions in 10 CFR 50.55a(f)(4)(iv) based upon your letter dated February 28, 1997, which proposes to meet OM-10 paragraphs 4.2.2.2, "Containment Isolation Valves;" 4.2.2.3(e), "Analysis of Leakage-Rates;" and 4.2.2.3(f), "Corrective Action." The proposed alternative in relief request VG-2, Revision 1, to not perform the trending required by IWV-3427(b) for LA containment isolation

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valves is approved because you will meet the NRC staff position as stated in GL 89-04, Position 10. The proposal to leak test the pressure isolation valves in accordance with the technical specifications and the guidelines in NUREG-1482, Section 4.4.7, "Pressure Isolation Valves," rather than IWV-3420 is approved pursuant to 10 CFR 50.55a(a)(3)(i), since it provides an acceptable level of guality and safety.

You should complete the actions to comply with the provisions in the approval of the alternative testing proposed for relief requests LP-RR-2, Revision 1, CRD-RR-3, Revision 1, and VG-2, Revision 1, within 180 days from the date of this letter.

If you have any questions regarding the enclosure, contact Mr. Darl Hood by phone at (301) 415-3049, or by electronic mail at dsh@nrc.com.

Sincerely,

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S. Singh Bajwa, Acting Director Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-220

Enclosure: Safety Evaluation

cc w/encl: See next page

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Sincerely,

ORIGINAL SIGNED BY A. DROMERICK FOR:

S. Singh Bajwa, Acting Director Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-220

Enclosure: Safety Evaluation

cc w/encl: See next page

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