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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

March 3, 1997

Mr. B. Ralph Sylvia Executive Vice President Generation Business Group and Chief Nuclear Officer Niagara Mohawk Power Corporation Nuclear Learning Center 450 Lake Road Oswego, NY 13126

SUBJECT: MODIFICATIONS TO CORRECT CORE SHROUD REPAIR DEVIATIONS. NINE MILE POINT NUCLEAR STATION, UNIT NO. 1 (TAC NO. M90102)

Dear Mr. Sylvia:

During the 1995 refueling outage for Nine Mile Point Nuclear Station, Unit No. 1 (NMP-1), Niagara Mohawk Power Corporation installed four core shroud stabilizer assemblies consisting of the rods, brackets, springs, and other parts. This repair measure was implemented to ensure the structural integrity of the core shroud with respect to the function of the circumferential welds that experienced cracks in some boiling water reactors (BWRs) as discussed in NRC Generic Letter 94-03, "Intergranular Stress Corrosion Cracking of Core Shrouds in Boiling Water Reactors," and other communications. The 1995 shroud repair was designed as an alternative to the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code pursuant to 10 CFR 50.55a(a)(3)(i), and was approved by the NRC staff by letter dated March 31, 1995.

As discussed in your letters dated March 23, 1995, and April 30, 1996, the post-installation inspection of the shroud repair revealed conditions that differed from the intended design. Therefore, in May 30 and August 14, 1996, letters, you described plans for two modifications to the stabilizer assemblies during the March 1997 outage. One modification will replace the tie rod and spring assembly with one having the spring on the opposite side of the tie rod because the lower spring currently bears, in part, on the blend radius of the recirculation nozzle. The other modification will add a Ushaped extension piece such that the lower spring contact for the 90°, 166° and 350° stabilizer assemblies will extend beyond the H6A weld, as originally intended. The replacement assembly at 270° will also extend beyond the H6A weld.

As discussed in the enclosed safety evaluation (SE), the NRC staff has reviewed the proposed modifications and finds that they adequately restore the stabilizer assemblies to the intended function previously approved by the NRC staff, and the loads and displacements at critical locations will be within allowable values specified in the design specifications. The modifications are, therefore, acceptable.

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B. Ralph Sylvia

March 3, 1997

This completes the NRC staff's efforts under TAC No. M90102. If you have questions, contact me by phone at (301) 415-3049 or by electronic mail at dsh@nrc.gov.

Sincerely,

/S/

Darl Hood, Senior Project Manager Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosure: SE

Docket No. 50-220

cc w/encl: See next page

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Darl Hord

Darl Hood, Senior Project Manager Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

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