



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77  
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated May 15, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 77 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Alexander W. Dromerick, Acting Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 17, 1996



ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Pages

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3/4 3-19A  
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3/4 3-25  
3/4 3-28

Insert Pages

v  
3/4 3-17  
3/4 3-19A  
3/4 3-19B\*  
3/4 3-25  
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\* New page



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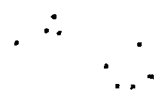




TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>Primary Containment Isolation Signals</u> (Continued)		
a. Reactor Vessel Water Level*		
1) Low, Low, Low, Level 1	$\geq 17.8$ in.	$\geq 10.8$ in.
2) Low, Low, Level 2	$\geq 108.8$ in.	$\geq 101.8$ in.
3) Low, Level 3	$\geq 159.3$ in.	$\geq 157.8$ in.
b. Drywell Pressure - High	$\leq 1.68$ psig	$\leq 1.88$ psig
c. Main Steam Line		
1) Radiation - High**	$\leq 3x$ Full Power Background	$\leq 3.6x$ Full Power Background
2) Pressure - Low	$\geq 766$ psig	$\geq 746$ psig
3) Flow - High	$\leq 121.5$ psid	$\leq 122.8$ psid
d. Main Steam Line Tunnel		
1) Temperature - High	$\leq 167.2^\circ\text{F}$	$\leq 170.6^\circ\text{F}$
2) $\Delta$ Temperature - High	$\leq 70.0^\circ\text{F}$	$\leq 71.7^\circ\text{F}$
3) Temperature - High MSL Lead Enclosure***	$\leq 148.2^\circ\text{F}$	$\leq 151.6^\circ\text{F}$
e. Condenser Vacuum Low	$\geq 8.5$ in Hg vacuum	$\geq 7.6$ in. Hg vacuum
f. RHR Equipment Area Temperature - High (HXs/A&B Pump Rooms)	$\leq 135^\circ\text{F}$	$\leq 144.5^\circ\text{F}$
g. Reactor Vessel Pressure - High (RHR Cut-in Permissive)	$\leq 128$ psig	$\leq 148$ psig
h. SGTS Exhaust - High Radiation	$\leq 5.7 \times 10^{-3}$ $\mu\text{Ci/cc}$	$\leq 1.0 \times 10^{-2}$ $\mu\text{Ci/cc}$

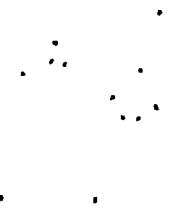


Table 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

- \*\*** Within 24 hours prior to the planned start of the hydrogen injection test and with the reactor power at greater than 20% rated power, the normal full-power radiation background level and associated trip and alarm setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip and alarm setpoints may be adjusted during the test program based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and associated trip and alarm setpoints shall be reset within 24 hours after completion of the hydrogen injection test. At reactor power levels below 20% rated power hydrogen injection shall be terminated, and control rod withdrawal is prohibited until the Main Steam Line Radiation Monitor trip setpoint is restored to its pre-test value.
- \*\*\*** The trip setpoint and allowable value for a channel may be established based on Figure 3.3.2-1, if:
- a. the actual ambient temperature reading for all operable channels in the Lead Enclosure area are equal to or greater than the ambient temperature used as the basis for the setpoint, and
  - b. the absence of steam leaks in the Main Steam Line Tunnel Lead Enclosure area is verified by visual inspection prior to increasing a channel setpoint, and
  - c. a surveillance is implemented in accordance with Note "d" of Table 4.3.2.1-1.



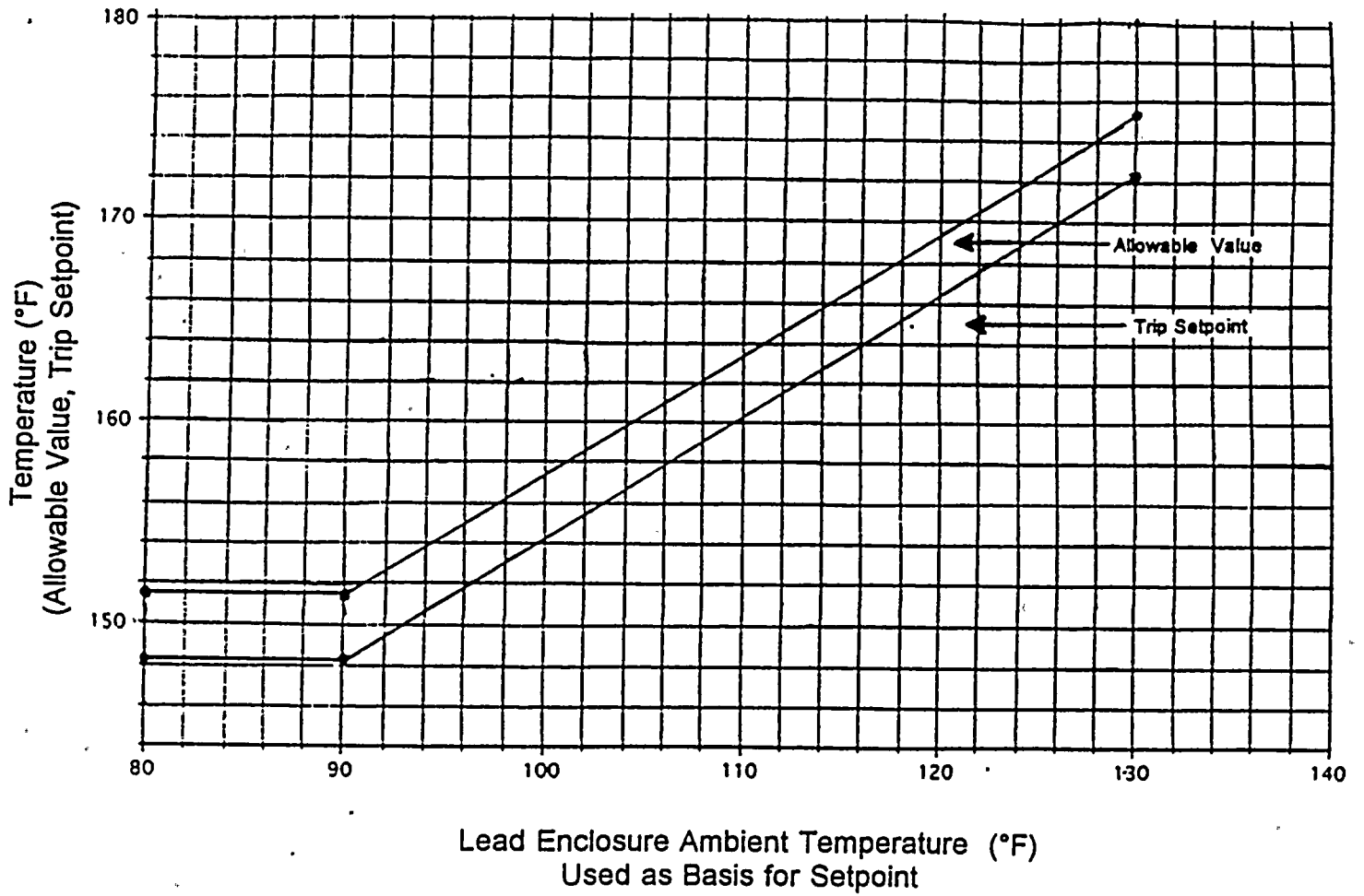


FIGURE 3.3.2-1 Allowable and Trip Setpoint Values for the Main Steam Line Tunnel Lead Enclosure



**TABLE 4.3.2.1-1**

**ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS**

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTION TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATION CONDITIONS FOR WHICH SURVEILLANCE IS REQUIRED</u>
1. <u>Primary Containment Isolation Signals</u>				
a. Reactor Vessel Water Level				
1) Low, Low, Low, Level 1	S	Q	R(a)	1, 2, 3
2) Low, Low, Level 2	S	Q	R(a)	1, 2, 3 and *
3) Low, Level 3	S	Q	R(a)	1, 2, 3
b. Drywell Pressure - High	S	Q	R(a)	1, 2, 3
c. Main Steam Line				
1) Radiation - High	S	Q	R	1, 2, 3
2) Pressure - Low	S	Q	R(a)	1
3) Flow - High	S	Q	R(a)	1, 2, 3
d. Main Steam Line Tunnel				
1) Temperature - High	S	Q	R(b)	1, 2, 3
2) ΔTemperature - High	S	Q	R(b)	1, 2, 3
3) Temperature - High MSL Lead Enclosure	S(d)	Q	R(b)	1, 2, 3
e. Condenser Vacuum - Low	S	Q	R(a)	1, 2**, 3**
f. RHR Equipment Area Temperature - High (HXs/A&B Pump Rooms)	S	Q	R(b)	1, 2, 3
g. Reactor Vessel Pressure High (RHR Cut-in Permissive)	S	Q	R(a)	1, 2, 3

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ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TABLE NOTATIONS

- \* During CORE ALTERATIONS and operations with a potential for draining the reactor vessel. This only applies to secondary containment isolation and automatic start of SGTS.
- \*\* When any turbine stop valve is greater than 90% open and/or when the key-locked condenser low vacuum bypass switch is open (in Normal position).
- † When handling irradiated fuel in the reactor building and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
  - (a) Perform the calibration procedure for the trip unit setpoint at least once per 92 days.
  - (b) Calibration excludes sensors; sensor response and comparison shall be done in lieu of.
  - (c) Manual isolation pushbuttons are tested at least once per operating cycle during shutdown. All other circuitry associated with manual isolation shall receive a CHANNEL FUNCTIONAL TEST at least once per 92 days as part of the circuitry required to be tested for the automatic system isolation.
  - (d) In addition to the normal shift channel check, if a channel setpoint has been established using Figure 3.3.2-1, then once per shift the actual ambient temperature reading for all operable channels in the Lead Enclosure area shall be verified to be equal to or greater than the ambient temperature used as the basis for the setpoint.

