



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated March 15, 1996, as supplemented July 18, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 75 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Jocelyn A. Mitchell for

Jocelyn A. Mitchell, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 27, 1996



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ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Pages

3/4 6-17

3/4 6-18

Insert Pages

3/4 6-17*

3/4 6-18

* Overleaf page (for TS with two-side copies--text not changed)



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CONTAINMENT SYSTEMS

DEPRESSURIZATION SYSTEMS

SUPPRESSION POOL

SURVEILLANCE REQUIREMENTS

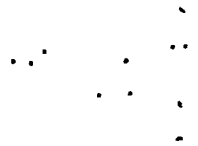
4.6.2.1 The suppression pool shall be demonstrated OPERABLE:

- a. By verifying the suppression pool water volume to be within the limits at least once per 24 hours.
- b. At least once per 24 hours in OPERATIONAL CONDITION 1 or 2 by verifying the suppression pool average water temperature to be less than or equal to 90°F, except:
 1. During testing that adds heat to the suppression pool verify the suppression pool average water temperature to be less than or equal to 105°F at least once per 5 minutes.
 2. When suppression pool average water temperature is greater than or equal to 90°F, verify at least once per hour that:
 - a) Suppression pool average water temperature is less than or equal to 110°F, and
 - b) THERMAL POWER is less than or equal to 1% of RATED THERMAL POWER after suppression pool average water temperature has exceeded 90°F for more than 24 hours.
 3. Following a scram with suppression pool average water temperature greater than 90°F, verify suppression pool average water temperature to be less than or equal to 120°F at least once per 30 minutes.
- c. By verifying at least 20 suppression pool water temperature instrumentation channels* OPERABLE by performance of a:
 1. CHANNEL CHECK at least once per 24 hours,
 2. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
 3. CHANNEL CALIBRATION** at least once per 18 months,

with the water high temperature alarm setpoints $\leq 90^{\circ}\text{F}$ for 10 of the temperature instruments and $\leq 110^{\circ}\text{F}$ for 10 of the temperature instruments.

* At least one pair in each of 10 suppression pool sectors with the alarm setpoint alternating between adjacent sectors.

** Calibration excludes sensors; sensors comparisons shall be made in lieu of calibration.



CONTAINMENT SYSTEMS

DEPRESSURIZATION SYSTEMS

SUPPRESSION POOL

SURVEILLANCE REQUIREMENTS

4.6.2.1 (Continued)

- d. At least once per 18 months by conducting a visual inspection of the exposed accessible interior and exterior surfaces of the suppression chamber.*
- e. At least every outage requiring the performance of a Containment Integrated Leak Rate Test, as scheduled in conformance with the criteria specified in the 10CFR50 Appendix J Testing Program Plan described in Section 6.8.4.f, by conducting a drywell-to-suppression chamber bypass leak test at an initial differential pressure of 3 psi and verifying that the A/\sqrt{K} calculated from the measured leakage is within the specified limit of 0.0054 square feet.
 - 1. If any drywell-to-suppression chamber bypass leak test fails to meet the specified limit, the test schedule for subsequent tests shall be reviewed and approved by the Commission.
 - 2. If two consecutive tests fail to meet the specified limit, a test shall be performed at least each refueling outage until two consecutive tests meet the specified limit, at which time the original test schedule may be resumed.
 - 3. The provisions of Specification 4.0.2 do not apply.
- f. During each refueling outage for which the drywell-to-suppression chamber bypass leak test in Specification 4.6.2.1.e is not conducted, by conducting a test of the four drywell-to-suppression chamber bypass leak paths containing the suppression chamber vacuum breakers at a differential pressure of at least 3 psi and
 - 1. verifying that the total leakage area A/\sqrt{K} contributed by all four bypass leak paths is less than or equal to 24% of the specified limit, and
 - 2. the leakage area for any one of the four bypass leak paths is less than or equal to 12% of the specified limit.

* Includes each vacuum relief valve and associated piping.



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