



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

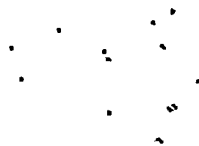
NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 76
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated March 20, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 76 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION


Jocelyn A. Mitchell, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 28, 1996



ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 76 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Pages

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Insert Pages

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TABLE 3.3.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION

NINE MILE POINT - UNIT 2

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| | <u>FUNCTIONAL UNIT</u> | <u>APPLICABLE OPERATIONAL CONDITIONS</u> | <u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (a)</u> | <u>ACTION</u> |
|----|---|--|--|---------------|
| 1. | Intermediate Range Monitors: | | | |
| a. | Neutron Flux - High | 2 | 3 | 1 |
| | | 3, 4 | 3 | 2 |
| | | 5(b) | 3 | 3 |
| b. | Inoperative | 2 | 3 | 1 |
| | | 3, 4 | 3 | 2 |
| | | 5 | 3 | 3 |
| 2. | Average Power Range Monitor(c): | | | |
| a. | Neutron Flux - Upscale, Setdown | 2 | 2 | 1 |
| | | 5(b)(k) | 2 | 3 |
| b. | Flow Biased Simulated Thermal Power - Upscale | 1 | 2 | 4 |
| c. | Fixed Neutron Flux - Upscale | 1 | 2 | 4 |
| d. | Inoperative | 1, 2 | 2 | 1 |
| | | 5(k) | 2 | 3 |
| 3. | Reactor Vessel Steam Dome Pressure - High | 1, 2(d) | 2 | 1 |
| 4. | Reactor Vessel Water Level - Low, Level 3 | 1, 2 | 2 | 1 |



TABLE 3.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION

TABLE NOTATIONS

- (a) A channel may be placed in an inoperable status for up to 6 hours for required surveillance without placing the Trip System in the tripped condition provided at least one OPERABLE channel in the same Trip System is monitoring that parameter.
- (b) Unless adequate shutdown margin has been demonstrated per Specification 3.1.1, and the Refuel position one-rod-out interlock is OPERABLE per Specification 3.9.1, the shorting links shall be removed from the RPS circuitry prior to and during the time any control rod is withdrawn.*
- (c) An APRM channel is inoperable if there are less than 2 LPRM inputs per level or less than 14 LPRM inputs to an APRM channel.
- (d) This function is not required to be OPERABLE when the reactor pressure vessel head is removed per Specification 3.10.1.
- (e) This function shall be automatically bypassed when the reactor mode switch is not in the Run position.
- (f) This function is not required to be OPERABLE when PRIMARY CONTAINMENT INTEGRITY is not required.
- (g) Also actuates the standby gas treatment system.
- (h) With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.
- (i) This function shall be automatically bypassed when turbine first stage pressure is less than or equal to 136.4** psig, equivalent to THERMAL POWER less than 30% of RATED THERMAL POWER.
- (j) Also actuates the EOC-RPT system.
- (k) Required to be OPERABLE only during shutdown margin demonstrations performed per Specification 3.10.3.

* Not required for control rods removed per Specification 3.9.10.1 or 3.9.10.2.

** To allow for instrument accuracy, calibration and drift, a setpoint of less than or equal to 125.8 psig turbine first stage pressure shall be used.



TABLE 4.3.1.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

| <u>FUNCTIONAL UNIT</u> | <u>CHANNEL CHECK</u> | <u>CHANNEL FUNCTIONAL TEST</u> | <u>CHANNEL CALIBRATION(a)</u> | <u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u> |
|--|----------------------|--------------------------------|-------------------------------|---|
| 1. Intermediate Range Monitors: | | | | |
| a. Neutron Flux - High | S/U, S,(b) S | S/U(c), W, R(d) W | R R | 2 3, 4, 5 |
| b. Inoperative | NA | W | NA | 2, 3, 4, 5 |
| 2. Average Power Range Monitor(e): | | | | |
| a. Neutron Flux - Upscale, Setdown | S/U, S,(b) S | S/U(c), W W | SA SA | 2 5(n) |
| b. Flow-Biased Simulated Thermal Power - Upscale | S, D(f) | S/U(c), Q | W(g)(h), SA, R(i) | 1 |
| c. Fixed Neutron Flux - Upscale | S | S/U(c), Q | W(g), SA | 1 |
| d. Inoperative | NA | Q | NA | 1, 2, 5(n) |
| 3. Reactor Vessel Steam Dome Pressure - High | S | Q | R(k) | 1, 2 |
| 4. Reactor Vessel Water Level - Low, Level 3 | S | Q | R(k) | 1, 2 |
| 5. Main Steam Line Isolation Valve - Closure | NA | Q | R | 1 |
| 6. Main Steam Line Radiation - High | S | Q | R | 1, 2(j) |
| 7. Drywell Pressure - High | S | Q | R(k) | 1, 2(l) |

NINE MILE POINT - UNIT 2

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TABLE 4.3.1.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TABLE NOTATIONS

- (a) Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (b) The IRM and SRM channels shall be determined to overlap for at least 1/2 decade during each startup after entering OPERATIONAL CONDITION 2, and the IRM and APRM channels shall be determined to overlap for at least 1/2 decade during each controlled shutdown, if not performed within the previous 7 days.
- (c) Within 24 hours before startup, if not performed within the previous 7 days.
- (d) Perform a CHANNEL FUNCTIONAL TEST with the mode switch in Startup/Hot Standby and the plant in the COLD SHUTDOWN or REFUEL Condition.
- (e) The LPRMs shall be calibrated at least once per 1000 effective full-power hours (EFPH) using the TIP system.
- (f) Verify measured core flow (total core flow) to be in the range of established core flow at the existing loop flow (APRM%).
- (g) This calibration shall consist of the adjustment of the APRM channel to conform to the power values calculated by a heat balance during OPERATIONAL CONDITION 1 when THERMAL POWER \geq 25% of RATED THERMAL POWER. Adjust the APRM channel if the absolute difference is greater than 2% of RATED THERMAL POWER. Any APRM channel gain adjustment made in compliance with Specification 3.2.2 shall not be included in determining the absolute difference.
- (h) This calibration shall consist of the adjustment of the APRM flow-biased channel to conform to a calibrated flow signal.
- (i) This calibration shall consist of verifying the 6 ± 0.6 seconds simulated thermal power time constant.
- (j) This function is not required to be OPERABLE when the reactor pressure vessel head is removed per Specification 3.10.1.
- (k) Perform the calibration procedure for the trip unit setpoint at least once per 92 days.
- (l) This function is not required to be OPERABLE when PRIMARY CONTAINMENT INTEGRITY is not required to be OPERABLE per Special Test Exception 3.10.1.
- (m) With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.
- (n) Required to be OPERABLE only during shutdown margin demonstrations performed per Specification 3.10.3.

