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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72 License No. NPF-69

- The Nuclear Regulatory Commission (the Commission) has found that: 1.
 - The application for amendment by Niagara Mohawk Power Corporation Α. (the licensee) dated January 25, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1:
 - Β. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
 - There is reasonable assurance (i) that the activities authorized С. by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - The issuance of this amendment will not be inimical to the common D. defense and security or to the health and safety of the public; and
 - Ε. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical 2. Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:



(2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 72 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Jocelyn a mitchell

Jocelyn A. Mitchell, Acting Director Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: June 10, 1996

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ATTACHMENT TO LICENSE AMENDMENT AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. NPF-69

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DOCKET NO. 50-410

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>		
3/4 3-33 3/4 3-43	3/4 3-33 3/4 3-43*		
3/4 3-44	3/4 3-44		

*Overleaf page

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TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

TABLE NOTATIONS

- * When the system is required to be OPERABLE per Specification 3.5.2 or 3.5.3.
- ** Required when the associated diesel generator is required to be OPERABLE.
- (a) When a channel is placed in an inoperable status solely for performance of required surveillances, entry into associated Conditions and required ACTIONS may be delayed for up to 6 hours provided the associated function or the redundant function maintains ECCS initiation capability.
- (b) Also actuates the associated division diesel generator.
- (c) Not required to be OPERABLE when reactor steam dome pressure is less than or equal to 100 psig.
- (d) The injection function of Drywell Pressure High and Manual Initiation is not required to be OPERABLE with indicated reactor vessel water level on the wide range instrument greater than level 8 setpoint coincident with the vessel pressure less than 600 psig because of hot calibration/cold operation level error.
- (e) Provides signal to close HPCS pump injection valve only.
- (f) Provides signal to HPCS pump suction valves only.

ACTION

- ACTION 30 With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement:
 - a. With one channel inoperable, place the inoperable channel in the tripped condition within 24 hours* or declare the associated system inoperable.
 - b. With more than one channel inoperable, declare the associated system inoperable.
- ACTION 31 With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, place the inoperable channel in the tripped condition within 24 hours; restore the inoperable channel to OPERABLE status within 7 days or declare the associated system inoperable.
- * The provisions of Specification 3.0.4 are not applicable.

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TABLE 4.3.3.1-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP	FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE IS REQUIRED		
D.	Loss of Power (Divisions I & II)						
	 4.16-kV Emergency Bus Undervoltage - Loss of Voltage 	S	м	R	1, 2, 3, 4†, 5†		
	 4.16-kV Emergency Bus Undervoltage - Degraded Voltage 	S	м	R	1, 2, 3, 4†, 5†		
Ε.	. Loss of Power (Division III)						
	 4.16-kV Emergency Bus Undervoltage - Loss of Voltage 	S	м	R	1, 2, 3, 4†, 5†		
	 4.16-kV Emergency Bus Undervoltage - Degraded Voltage 	S	м	R	1, 2, 3, 4†, [∴] 5†		

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TABLE 4.3.3.1-1 (Continued)

EMERGENCY CORE COOLING SYSTEM

ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

: TABLE NOTATIONS

- * When the system is required to be OPERABLE per Specification 3.5.2.
- ** Not required to be OPERABLE when reactor steam dome pressure is less than or equal to 100 psig.
- + Required when the associated diesel generator is required to be OPERABLE.
- (a) Manual initiation switches shall be tested at least once per 18 months during shutdown. All other circuitry associated with manual initiation shall receive a CHANNEL FUNCTIONAL TEST at least once per 92 days as part of the circuitry required to be tested for automatic system actuation.
- (b) The injection function of Drywell Pressure High and Manual Initiation is not required to be OPERABLE with indicated reactor vessel water level on the wide range instrument greater than Level 8 setpoint coincident with the vessel pressure less than 600 psig due to the hot calibration/cold operation level error.
- (c) Perform the calibration procedure for the Trip Setpoint at least once per 92 days.

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