



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated January 6, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:



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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 69 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION...


Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 11, 1995



ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

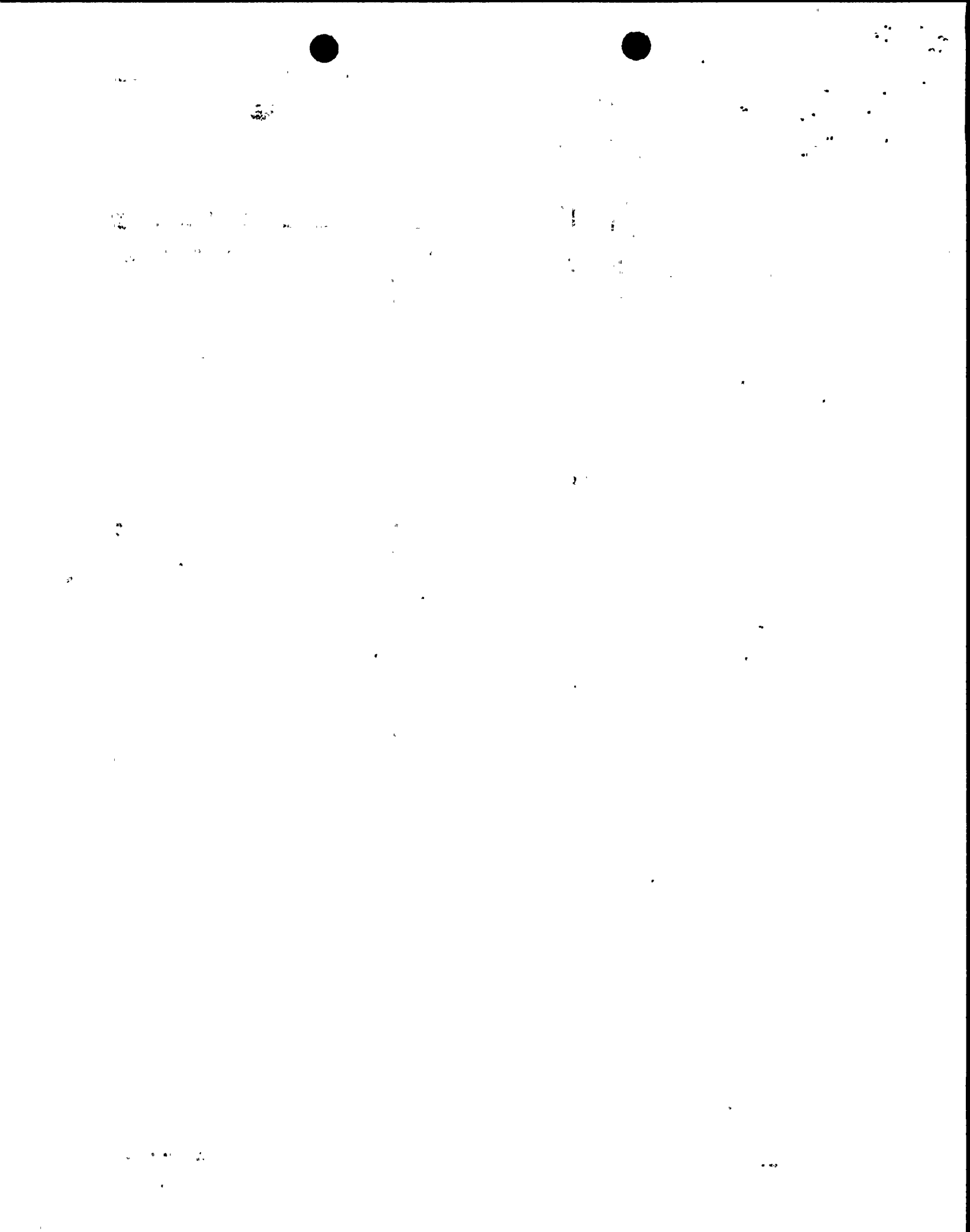
Revise Appendix A as follows:

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3/4 3-81
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3/4 3-87
3/4 4-10
3/4 4-11

Insert Pages

3/4 3-81
3/4 3-82
3/4 3-83
3/4 3-84
3/4 3-85
3/4 3-86
3/4 3-87
3/4 4-10
3/4 4-11



INSTRUMENTATION

MONITORING INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITIONS FOR OPERATION

3.3.7.5 The accident monitoring instrumentation channels shown in Table 3.3.7.5-1 shall be OPERABLE.

APPLICABILITY: Operational Conditions 1 and 2.

ACTION:

- a. With one or more accident monitoring instrumentation channels inoperable, take the ACTION required by Table 3.3.7.5-1.
- b. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.7.5 Each of the above required accident monitoring instrumentation channels shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.7.5-1.

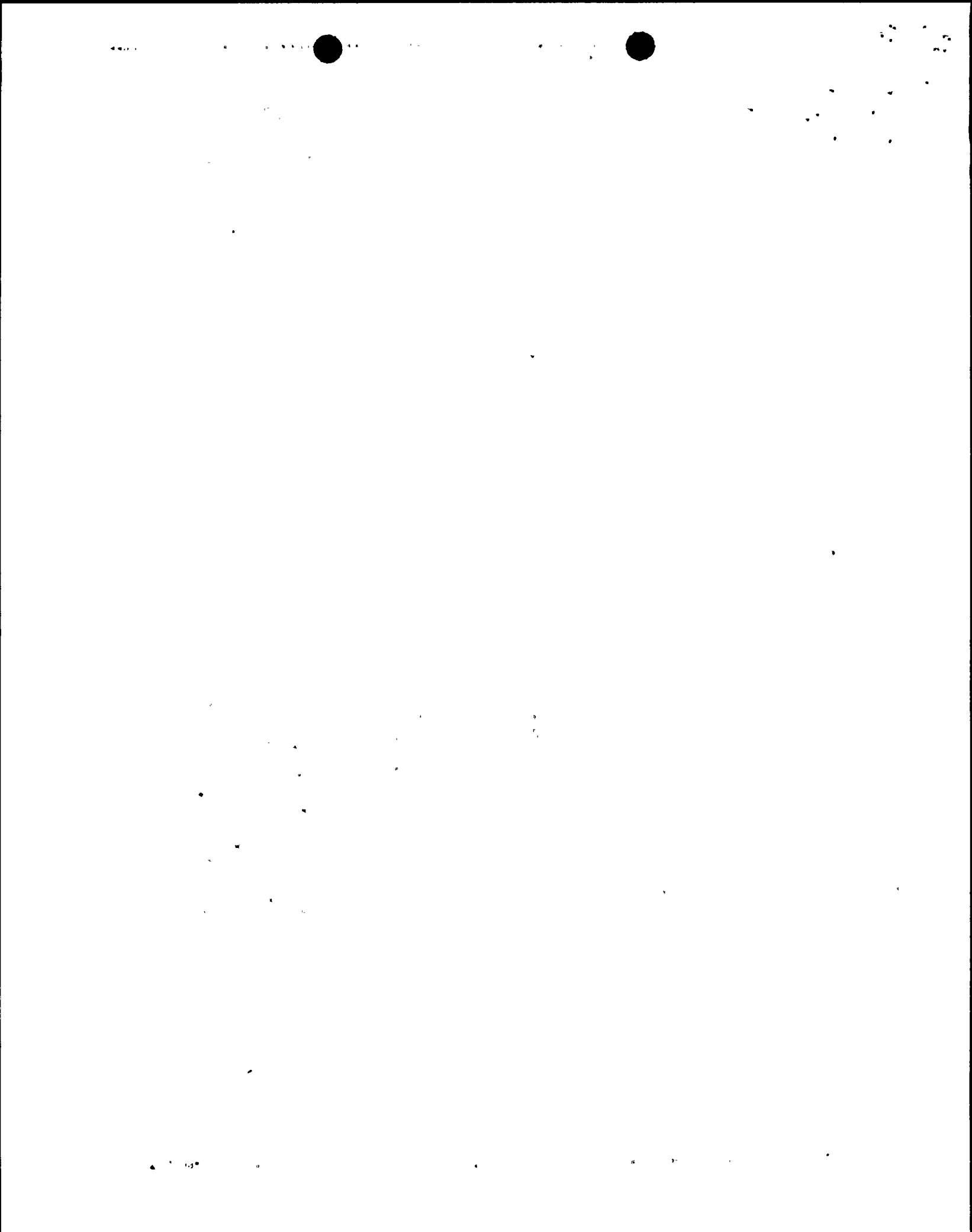


TABLE 3.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>ACTION</u>
1. Reactor Vessel Pressure	2	80
2. Reactor Vessel Water Level		
a. Fuel Zone	2	80
b. Wide Range	2	80
3. Suppression Pool Water Level		
a. Narrow Range	2	80
b. Wide Range	2	80
4. Suppression Pool Water Temperature	2/Quadrant	80
5. Suppression Chamber Pressure	2	80
6. Suppression Chamber Air Temperature	2	80
7. Drywell Pressure		
a. Narrow Range	2	80
b. Wide Range	2	80
8. Drywell Air Temperature	2	80
9. Drywell Oxygen Concentration	2	80
10. Drywell Hydrogen Concentration Analyzer and Monitor	2	80

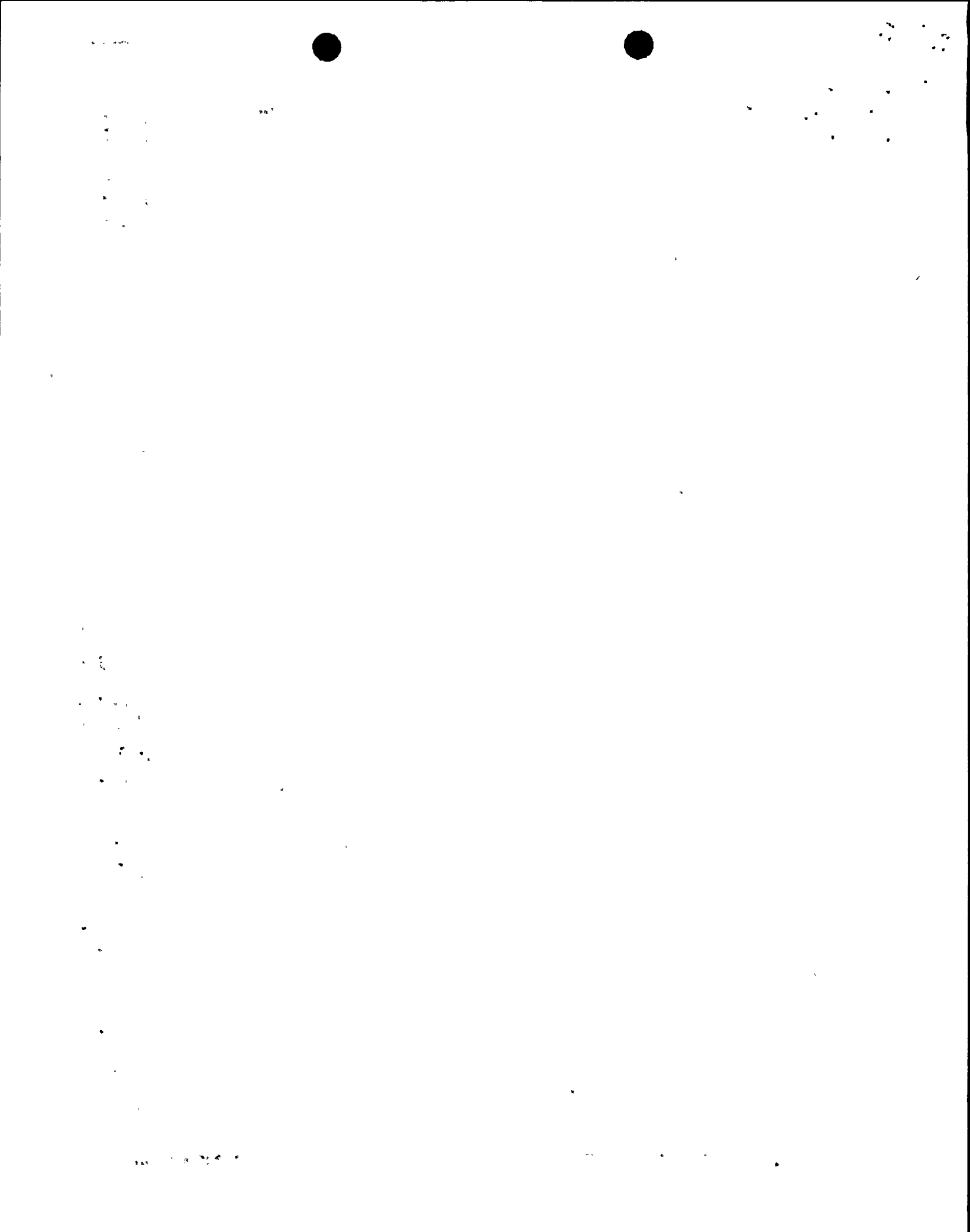


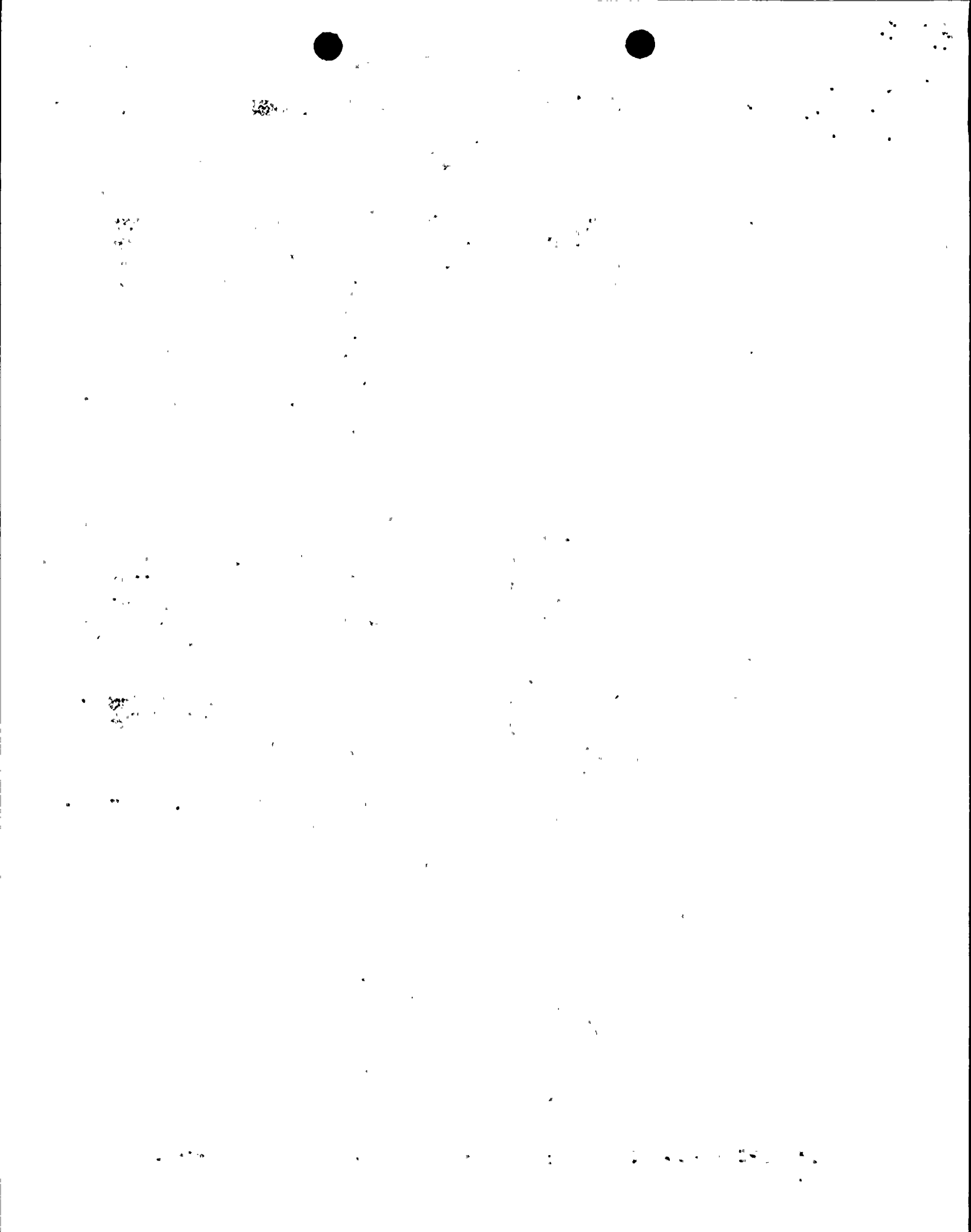
TABLE 3.3.7.5-1 (Continued)

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>ACTION</u>
11. Drywell High Range Radiation Monitors	2	81
12. Penetration Flow Path Primary Containment Isolation Valve Position Indication*	2**	80**

* Not required for isolation valves whose associated flow path is isolated by at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

** Only one position indication channel is required for penetration flow paths with only one control room indication channel. If this one channel becomes inoperable, entry into ACTION 80b is required.



ACCIDENT MONITORING INSTRUMENTATION

ACTION

- ACTION 80 - a.** With the number of OPERABLE accident monitoring instrumentation channels for one or more functions one less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 30 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the following 14 days outlining the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channel(s) of the function(s) to OPERABLE status.
- b. With the number of OPERABLE accident monitoring instrumentation channels for one or more functions two less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.
- ACTION 81 - a.** With the number of OPERABLE Drywell High Range Radiation Monitors one less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel to OPERABLE status within 30 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the following 14 days outlining the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channel of the function to OPERABLE status.
- b. With the number of OPERABLE Drywell High Range Radiation Monitors two less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the following 14 days outlining the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channel(s) of the function to OPERABLE status.



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TABLE 4.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure	M	R
2. Reactor Vessel Water Level!		
a. Fuel Zone	M	R
b. Wide Range	M	R
3. Suppression Pool Water Level		
a. Narrow Range	M	R
b. Wide Range	M	R
4. Suppression Pool Water Temperature	M	R*
5. Suppression Chamber Pressure	M	R
6. Suppression Chamber Air Temperature	M	R*
7. Drywell Pressure		
a. Narrow Range	M	R
b. Wide Range	M	R
8. Drywell Air Temperature	M	R*
9. Drywell Oxygen Concentration	M	R
10. Drywell Hydrogen Concentration Analyzer and Monitor	M	Q**
11. Drywell High Range Radiation Monitors	M	R*
12. Penetration Flow Path Primary Containment Isolation Valve Position Indication	M††	R***



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ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TABLE NOTATIONS

- * Excludes sensors; sensor comparison shall be done in lieu of sensor calibration.
- ** Using sample gas containing:
 - a. One volume percent hydrogen, balance nitrogen.
 - b. Four volume percent hydrogen, balance nitrogen.
- *** The CHANNEL CALIBRATION shall consist of position indication verification using ASME Section XI IWV-3300 test criteria.
- † The CHANNEL CALIBRATION shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/hr and a one point calibration check of the detector below 10 R/hr with an installed or portable gamma source.
- †† Red, Green or other indication shall be verified as indicating valve position.



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REACTOR COOLANT SYSTEM

3/4.4.2 SAFETY/RELIEF VALVES

LIMITING CONDITIONS FOR OPERATION

3.4.2 The safety valve function of at least 16 of the following reactor coolant system safety/relief valves shall be OPERABLE with the specified code safety valve function lift settings*:

- 2 safety/relief valves @ 1148 psig $\pm 1\%$
- 4 safety/relief valves @ 1175 psig $\pm 1\%$
- 4 safety/relief valves @ 1185 psig $\pm 1\%$
- 4 safety/relief valves @ 1195 psig $\pm 1\%$
- 4 safety/relief valves @ 1205 psig $\pm 1\%$

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With the safety valve function of one or more of the above required 16 safety/relief valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- b. With one or more safety/relief valves stuck open, provided that the average water temperature in the suppression pool is less than 110°F, close the stuck-open safety/relief valve(s); if unable to close the open valve(s) within 5 minutes or if the average water temperature in the suppression pool is 110°F or more, place the reactor mode switch in the Shutdown position.

* The lift setting pressure shall correspond to ambient conditions of the valves at nominal operating temperatures and pressures.



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REACTOR COOLANT SYSTEM

3/4.4.2 SAFETY/RELIEF VALVES

SURVEILLANCE REQUIREMENTS

4.4.2 No requirements other than Specification 4.0.5.

