

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001 March 31, 1995

Docket File

50-220

MAY

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Mr. B. Ralph Sylvia Executive Vice President, Nuclear Niagara Mohawk Power Corporation Nine Mile Point Nuclear Station P.O. Box 63 Lycoming, NY 13093

## SUBJECT: NINE MILE POINT NUCLEAR STATION UNIT NO. 1 (NMP1), EVALUATION OF CORE SHROUD STABILIZER DESIGN (TAC NO. M91273)

Dear Mr. Sylvia:

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The purpose of this letter is to transmit our safety evaluation (SE) of the subject matter. Based on our review, we find that your proposed core shroud stabilizer design is acceptable as documented in the enclosed safety evaluation.

By letter dated January 6, 1995, as supplemented by letters dated January 23, and 26, February 14, 24, and 28, March 7 and 9, two on March 13, two on March 14, March 23, 27, 28, and 30, 1995, Niagara Mohawk Power Corporation (NMPC) submitted the details of the planned repair of the circumferential welds for the Nine Mile Point Unit 1 (NMP1) reactor core shroud. Information was also provided to the NRC staff during conference calls held on March 1, 3, 23, 24, and 27, 1995. The March 28, 1995, letter confirmed that the information provided during the conference calls would be formally submitted on the NMP1 docket no later than March 31, 1995.

Initially, NMPC's planned permanent repair involved installation of four tierod assemblies combined with core plate wedges to replace welds H1 through H7 and six brackets to replace the downward vertical load capability of the H8 weld. It was NMPC's intention to examine the H1 through H8 shroud welds in accordance with the Boiling Water Reactor Vessel and Internals Project Inspection Criteria and install the tie-rod assemblies and/or the H8 weld brackets only if cracking was found to be unacceptable for continued plant operation. Based on the results of the ultrasonic examination of the H8 weld, NMPC decided to install the four tie-rod assemblies and not the brackets.

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B. Sylvia

The NRC staff has reviewed the above submittals. Our evaluation is provided in the enclosed Safety Evaluation. The proposed core shroud repair has been designed as an alternative to the requirements of ASME Boiler and Pressure Vessel Code pursuant to Title 10, Code of Federal Regulations, Part 50.55a(a)(3)(i). This alternative is acceptable. This completes our action with respect to TAC No. M91273.

Sincerely,

/S/

Ledyard B. Marsh, Director Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-220

Enclosure: Safety Evaluation

cc w/encl: See next page

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LB March

Ledyard B. Marsh, Director Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

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B. Ralph Sylvia Niagara Mohawk Power Corporation

cc:

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