



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION I

475 ALLENDALE ROAD  
KING OF PRUSSIA, PENNSYLVANIA 19406

March 28, 1988

88 AUG -5 P4:08

Docket No. 50-220  
50-410

CAL No. 88-13  
EA No. 88-95

PUBLIC

Niagara Mohawk Power Corporation  
ATTN: Mr. C. V. Mangan  
Senior Vice President  
301 Plainfield Road  
Syracuse, New York 13212

Gentlemen:

Subject: CONFIRMATORY ACTION LETTER 88-13: MAINTENANCE OF OPERATOR LICENSES

On February 22-26, 1988, the NRC conducted an inspection of your Licensed Operator Retraining and Continuing Training Program. The results of this inspection concluded that documentation of continuing training, required reading assignments and required makeup training attendance was missing from individual training files for Unit 1 licensed Operators and Senior Operators.

On March 22-25, 1988, a special announced inspection was conducted to review the results of your internal investigation regarding the missing documentation. The results of your investigation concluded that portions of required training, for licensed Operators and Senior Operators, had not been completed prior to the end of your annual requalification period of February 22, 1988, in accordance with your program. In accordance with 10 CFR 55.53(h), a licensed operator or senior operator must be enrolled in the requalification program of the facility for which he holds a license, in order to meet the conditions of his/her license. In addition, an applicant for renewal of a license is required to have satisfactorily completed the requalification program for the facility for which the license renewal is sought in accordance with 10 CFR 55.57(a)(4).

10 CFR 50.54(i-1) (Conditions of Licenses) states in part that the licensee shall have in effect an operator requalification program which must as a minimum meet the requirements of 55.59(c) (Requalification Program).

On March 25, 1988, an exit meeting was conducted to discuss apparent violations associated with the implementation and documentation of your Licensed Operator Retraining Program, i.e., your NRC approved Requalification Program.

We are concerned with the potential adverse implications that licensed Operators and Senior Operators are not current and that license renewal applications received to date are not complete and accurate.

During a telephone call with you on March 28, 1988, you agreed that the following actions have been or will be taken:

1. Complete all requalification training, as required by your approved licensed Operator Training Program, prior to the restart of the Unit 1 reactor.

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Niagara Mohawk Power Corporation -2-

2. Verify that those individuals whose licenses have been renewed since May 26, 1987, have completed the approved requalification program requirements.
3. Verify that requalification training for Unit 2 is being maintained in accordance with the requalification program requirements.
4. Submit a report upon completion of the above items to the Administrator, Region 1.

Further, we understand that Unit 1 will not be taken critical until you receive authorization from the Regional Administrator.

If your understanding of the actions to be taken are different than those described above, please contact this office within 24 hours of receipt of this letter.

Thank you for your cooperation.

Sincerely,



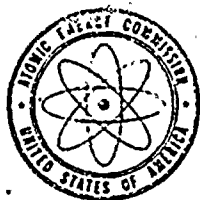
William V. Johnston, Acting Director  
Division of Reactor Safety  
Region I

cc:

T. E. Lempges, Vice President, Nuclear Generation  
J. A. Perry, Vice President, Quality Assurance  
T. Perkins, General Superintendent, Nuclear Generation  
W. Hansen, Manager Corporate Quality Assurance  
T. Roman, Unit 1 Station Superintendent  
J. Aldrich, Unit 1 Superintendent, Operations  
C. Beckham, Manager Nuclear Quality Assurance Operations  
W. Drews, Technical Superintendent  
Troy B. Conner, Jr. Esquire  
John W. Keib, Esquire  
Director, Power Division  
Public Document Room (PDR)  
Local Public Document Room (LPDR)  
Nuclear Safety Information Center (NSIC)  
NRC Resident Inspector  
State of New York



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UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

FACILITY FILE

OCT 3 1974

Docket No 50- 220

Niagara Mohawk Power Corporation  
ATTN: Mr. T. J. Perkins,  
Station Superintendent  
Nine Mile Point Nuclear Station  
P.O. Box 32  
Lycoming, New York 13093

Gentlemen:

Nuclear power plant simulators were introduced into operator training programs in 1969. In the time since their introduction the Operator Licensing Branch has carefully evaluated their role in the training programs. Based on this evaluation the Operator Licensing Branch has approved increased use of appropriate simulators in the administration of the operator licensing program.

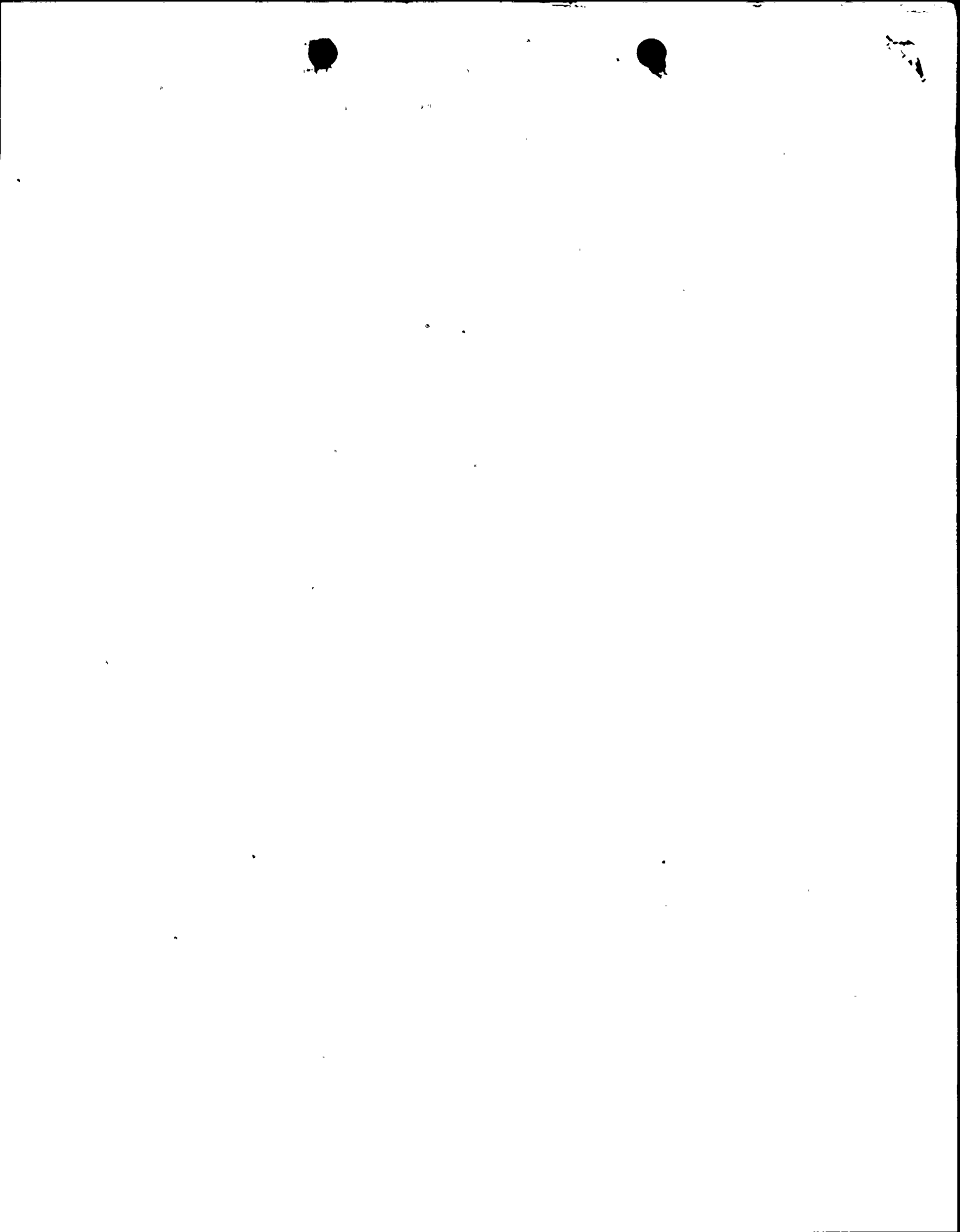
First, the applicants seeking licenses prior to criticality at a facility were given appropriate credit for participating in training programs that utilized acceptable simulators. Then, training programs were approved that utilized simulators for facilities after which the simulators were modeled. Further, the use of appropriate simulators was permitted in meeting certain requirements of the operator requalification programs.

We have now determined that it is acceptable to use nuclear power plant simulators in determining the qualifications of individuals who apply for licenses after initial criticality.

Presently, applicants for licenses, after the facility has achieved criticality must participate in training programs that include, at least, two reactor startups at the facility for which he seeks a license to meet the eligibility requirements to sit for an examination. In addition, applicants must perform a startup of the reactor as part of the operating test.

The Operator Licensing Branch will consider training programs that utilize appropriate nuclear power plant simulators for startup experience for meeting the startup eligibility requirements to sit for examination. In addition, a reactor startup will not be required as part of the operating test, providing that appropriate certification regarding an individual's ability to manipulate the controls is contained in his application.

LB



OCT 3 1974

In order for the applicant to be eligible for this alternate program, the following requirements must be met.

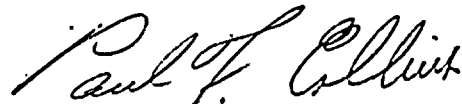
1. The applicant has manipulated the controls of his reactor facility during five significant reactivity changes, which may, or may not, include reactor startups.
2. The applicant has participated in an AEC-approved training program that includes training at a nuclear power plant simulator, and
3. The application contains a certification from the simulator training center attesting to the applicant's:
  - a. ability to manipulate the controls and keep the reactor under control during a reactor startup,
  - b. ability to predict instrument response and use the instrumentation during a reactor startup,
  - c. ability to follow the facility startup procedure and
  - d. ability to explain alarms and annunciators that may occur during this operation.

The simulators used in the programs must meet the present requirements for simulators enumerated in Paragraph 3. e., Appendix A of 10 CFR Part 55, namely, that the simulator reproduce the general operating characteristics of the facility involved, and the arrangement of instrumentation and controls of the simulator is similar to that of the facility involved.

The present procedure for training and examining will be continued for those facilities who do not desire to use the alternate procedure described above.

I hope this information will be of use to you. If you have any questions, please contact me.

Sincerely,



Paul F. Collins, Chief  
Operator Licensing Branch  
Directorate of Licensing



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MAR 13 1974

Docket No. 50-220

Niagara Mohawk Power Corporation  
ATTN: Mr. R. R. Schneider  
Vice President-Electric Operations  
300 Erie Boulevard West  
Syracuse, New York 13202

Gentlemen:

We have reviewed the revised Nine Mile Point Station requalification program for licensed operators and senior operators submitted on February 21, 1974. Based on our review of the material submitted, we have determined the program meets the requirements of Section 50.54(i-1) of 10 CFR Part 50 and Appendix A of 10 CFR Part 55.

The program adequately describes:

1. The lecture series to be administered, including subjects and duration.
2. The specific manipulations of controls to meet the requirements of Section 3a of Appendix A.
3. The methods to be employed to assure individual review of design, procedure and license changes.
4. The methods to be employed to assure individual review of abnormal and emergency procedures.
5. The specific evaluation criteria for determining attendance at a specific lecture, required participation in an accelerated requalification program and other additional training, as applicable.

APP1

MAR 1 3 1946

MAR 13 1974

Niagara Mohawk Power Corporation

- 2 -

6. The records to be maintained to document each individual's participation in the program.

Sincerely,

ORIGINAL SIGNED BY  
P. F. COLLINS

Paul F. Collins, Chief  
Operator Licensing Branch  
Directorate of Licensing

cc: Arvin E. Upton, Esquire  
LeBoeuf, Lamb, Leiby & MacRae  
1821 Jefferson Place, N.W.  
Washington, D. C. 20036

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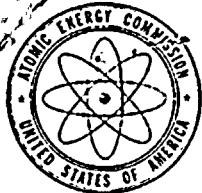
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- CJDeBevec
- RMDiggs
- PFCollins
- JJHolman

OFFICE ▶	OLB:L	ORB#2	OLB:L	ADOR:L		
SURNAME ▶	JJHolman:dlf	CJDeBevec	PFCollins	DJSKovholt		RG
DATE ▶	3/13/74	3/13/74	3/13/74	3/13/74		

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UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

Docket No. 50-220

MAR 13 1974

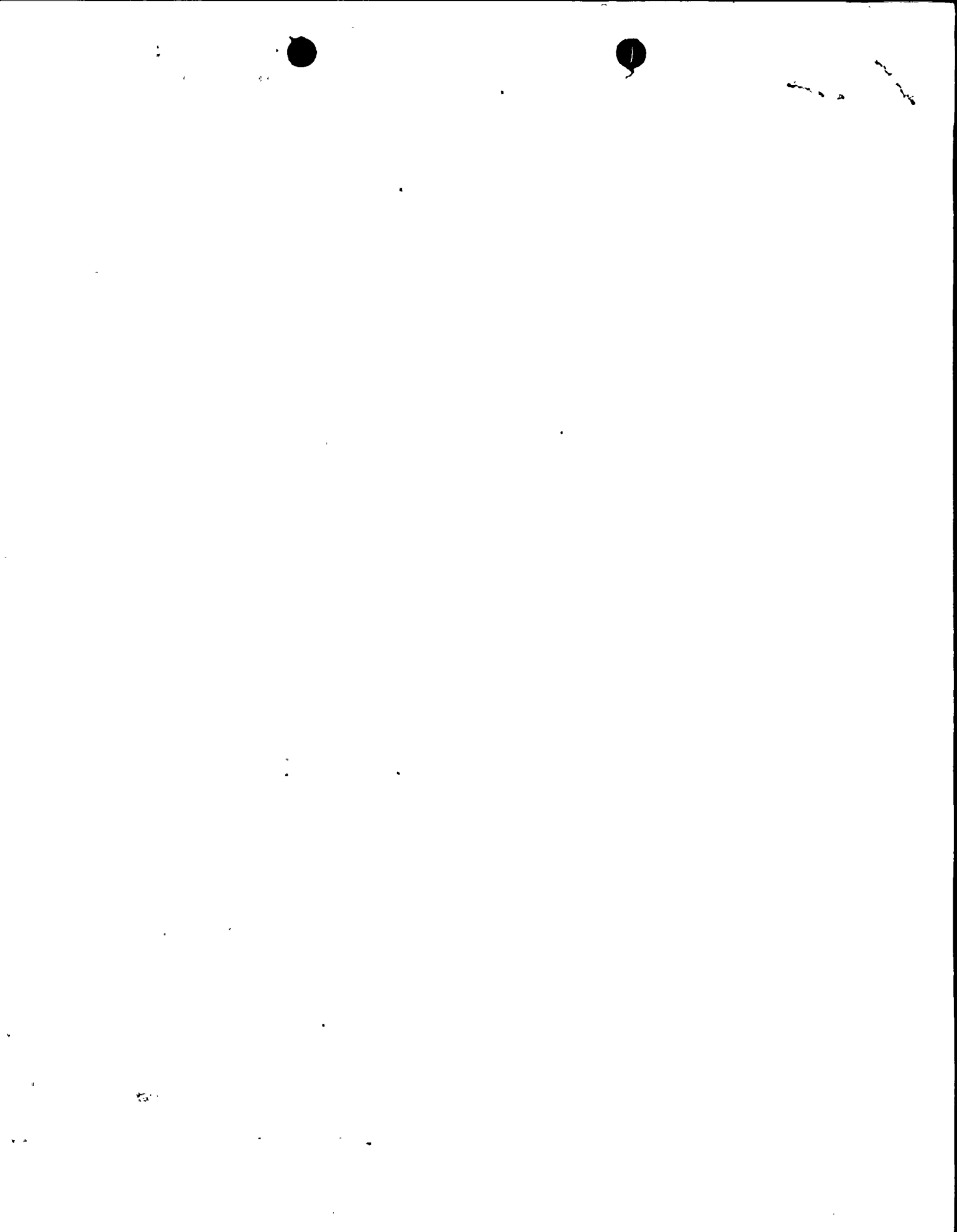
Niagara Mohawk Power Corporation  
ATTN: Mr. R. R. Schneider  
Vice President-Electric Operations  
300 Erie Boulevard West  
Syracuse, New York 13202

Gentlemen:

We have reviewed the revised Nine Mile Point Station requalification program for licensed operators and senior operators submitted on February 21, 1974. Based on our review of the material submitted, we have determined the program meets the requirements of Section 50.54(i-1) of 10 CFR Part 50 and Appendix A of 10 CFR Part 55.

The program adequately describes:

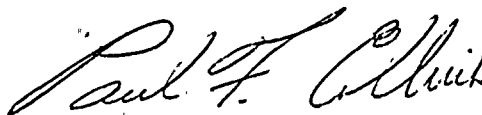
1. The lecture series to be administered, including subjects and duration.
2. The specific manipulations of controls to meet the requirements of Section 3a of Appendix A.
3. The methods to be employed to assure individual review of design, procedure and license changes.
4. The methods to be employed to assure individual review of abnormal and emergency procedures.
5. The specific evaluation criteria for determining attendance at a specific lecture, required participation in an accelerated requalification program and other additional training, as applicable.



MAR 13 1974

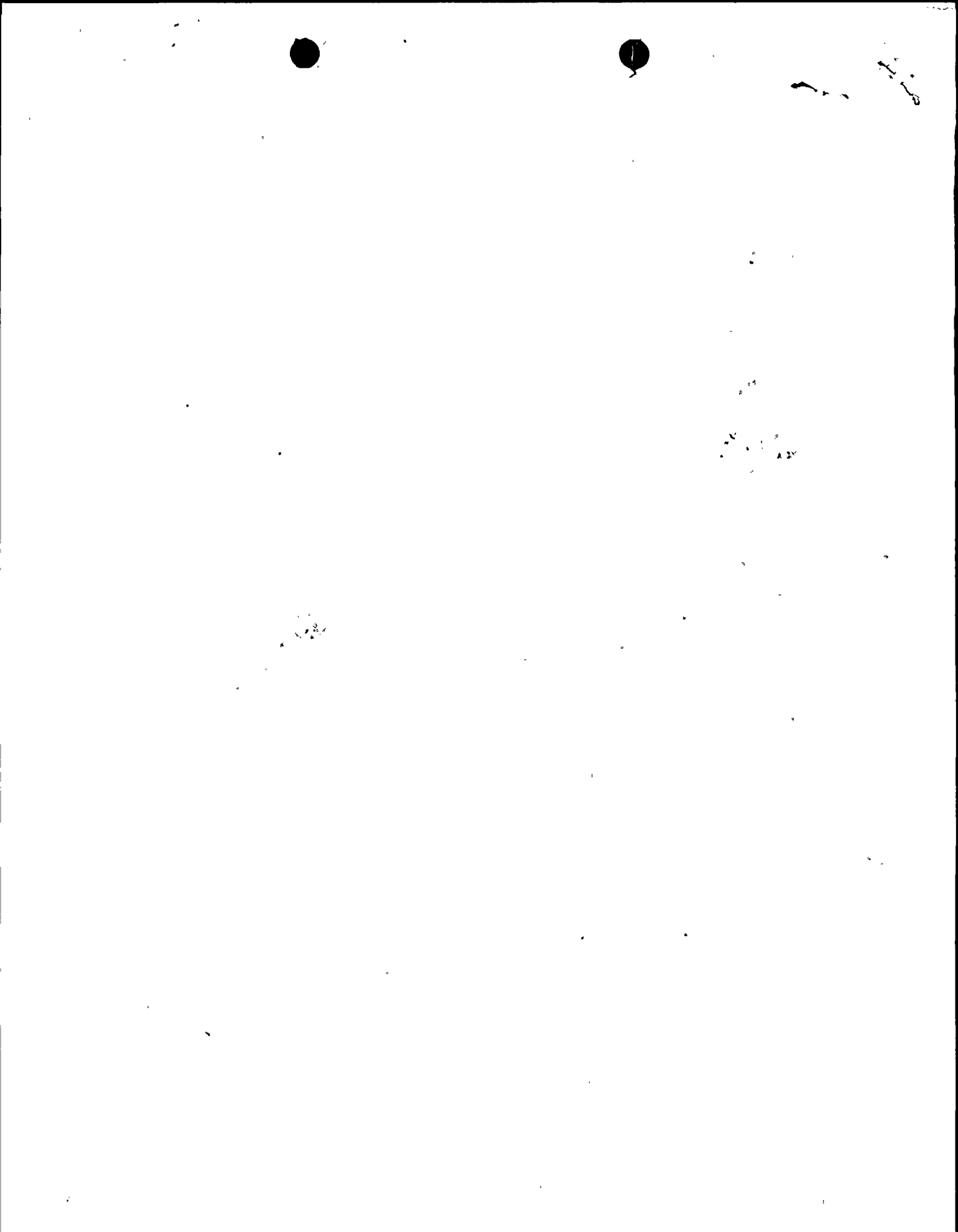
6. The records to be maintained to document each individual's participation in the program.

Sincerely,



Paul F. Collins, Chief  
Operator Licensing Branch  
Directorate of Licensing

cc: Arvin E. Upton, Esquire  
LeBoeuf, Lamb, Leiby & MacRae  
1821 Jefferson Place, N.W.  
Washington, D. C. 20036





*Fidelity  
File*

FEB 7 1974

Niagara Mohawk Corporation  
ATTN: Mr. P. Allister Burt,  
General Superintendent  
Nuclear Generation  
Nine Mile Point Station  
P. O. Box 32  
Lycoming, New York 13093

Gentlemen:

We have reviewed your letter, dated August 31, 1973. Your letter requested that we review your training program and fourteen individual resumes to determine if the individuals involved would be eligible to sit for cold license examinations for the FitzPatrick Station.

Based on our review of your training program, resumes and discussion with you, we have determined that additional information is necessary for us to make a determination regarding cold eligibility.

Your formal training program consisted of six phases for the senior operator applicants and five phases for the operator applicants. In addition, each individual was to have obtained operating experience at the Nine Mile Point Station and have participated in pre-startup activities at the FitzPatrick Station. The first five phases of the formal training program were completed in May 1973. These programs were to be followed by phase F, refresher course. It is our understanding that phase F training has not been completed.

Based on the time that has elapsed since the completion of phase E, we believe that phase F should be more extensive than that implied by "refresher course". We believe that immediately prior to the administration of phase F training each individual involved be examined to determine areas of weaknesses. The course content of phase F can be developed based on the above evaluations in addition to the overall review. In addition, we believe a thorough review of the FitzPatrick procedures is in order since the procedures were issued subsequent to the completion of phase E training. Please submit an outline of your phase F training for our review.



FEB 7 1974

Niagara Mohawk Corp.

- 2 -

Also, there was some question in your letter regarding who, Nine Mile Point, General Electric or General Physics personnel, would be teaching phase F and who would be certifying to the successful completion of the program. Please inform us as to who will teach the course and who will certify to successful completion of the course. In addition, please confirm our understanding that individuals who do not receive certification of successful completion will not be endorsed by Niagara Mohawk to take the cold examinations.

Upon receipt of this information, we will make a determination regarding cold eligibility of the individuals involved.

Based on information submitted by you, we had scheduled the written portion of the cold examinations for the week of March 3, 1974 and the cold operating tests for the week of March 23, 1974. In view of the above and other information we have regarding the status of your Technical Specifications, startup procedures and your program for quality assurance for operations, we believe these dates to be unrealistic. In addition, we informed you in a letter, dated June 10, 1973, that applications should be submitted eight weeks prior to the written examination date. To date, the applications have not been received. Consequently, we will not plan on administering the examinations during the month of March.

Upon resolution of the above items, Mr. Robert J. Campbell, EWR Group Leader, OLB, will make arrangements for administration of the examinations.

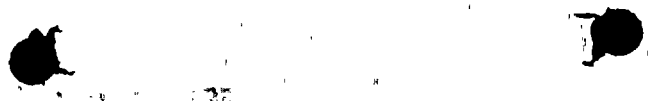
Sincerely,

ORIGINAL SIGNED BY  
P. F. COLLINS

Paul F. Collins, Chief  
Operator Licensing Branch  
Directorate of Licensing

See previous concurrence sheet.

OFFICE ▶	OLB:L	OLB:L	LWRB 2-1	ADOR:L		
SURNAME ▶	RJCampbell:dlf	PFCollins	JSnell	DJSkowholt		
DATE ▶	2/ /74	2/ /74	2/ /74	2/ /74		



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DATE	NO.	NAME	ADDRESS

FEB 7 1974

Also, there was some question in your letter regarding who, Nine Mile Point, General Electric or General Physics personnel, would be teaching phase F and who would be certifying to the successful completion of the program. Please inform us as to who will teach the course and who will certify to successful completion of the course. In addition, please inform us if our understanding is correct that individuals who do not certify will not sit for cold examinations.

Upon receipt of this information, we will make a determination regarding cold eligibility of the individuals involved.

Based on information submitted by you, we had scheduled the written portion of the cold examinations for the week of March 3, 1974 and the cold operating tests for the week of March 23, 1974. In view of the above and other information we have regarding the status of your Technical Specification, startup procedures and your program for quality assurance for operations, we believe these dates to be unrealistic. In addition, we informed you in a letter, dated June 10, 1973, that applications should be submitted eight weeks prior to the written examination date. To date, the applications have not been received. Consequently, we will not plan on administering the examinations during the month of March.

Upon resolution of the above items, Mr. Campbell, BWR Group Leader, OLB, will make arrangements for administration of the examinations.

Sincerely,

ORIGINAL SIGNED BY  
E. F. COLLINS

Paul F. Collins, Chief  
Operator Licensing Branch  
Directorate of Licensing

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D.J. Skovholt, L  
R. J. Campbell, L

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RO (3)  
OGC

Docket No. 50-220

Niagara Mohawk Power Corporation  
ATTN: Mr. Rudolph R. Schneider,  
Vice President-Electric  
Operations  
300 Erie Boulevard West  
Syracuse, New York 13202

Gentlemen:

By letter dated December 13, 1973, in accordance with the requirements of 10 CFR 50, you submitted a proposal for our approval for the Program for Requalification of AEC Licensed Personnel of the Nine Mile Point Station. We have reviewed your proposed program and we conclude that the additional information identified in Attachment A is required before we can act on your proposal.

Sincerely,  
ORIGINAL SIGNED BY  
P. F. COLLINS

Paul F. Collins, Chief  
Operator Licensing Branch  
Directorate of Licensing

Enclosure:  
Attachment A - Request  
for Additional Information

cc: w/enclosure  
Arvin E. Upton, Esquire  
LeBoeuf, Lamb, Leiby & Mac Rae  
1821 Jefferson Place, N.W.  
Washington, D. C. 20036

OFFICE ▶	OLB:L <i>[Signature]</i>	ORB #2 <i>[Signature]</i>	OLB:L <i>[Signature]</i>		
SURNAME ▶	JJHollman:dlf	CJDeBevec	PFCollins		
DATE ▶	1/14/74	1/14/74	1/14/74		

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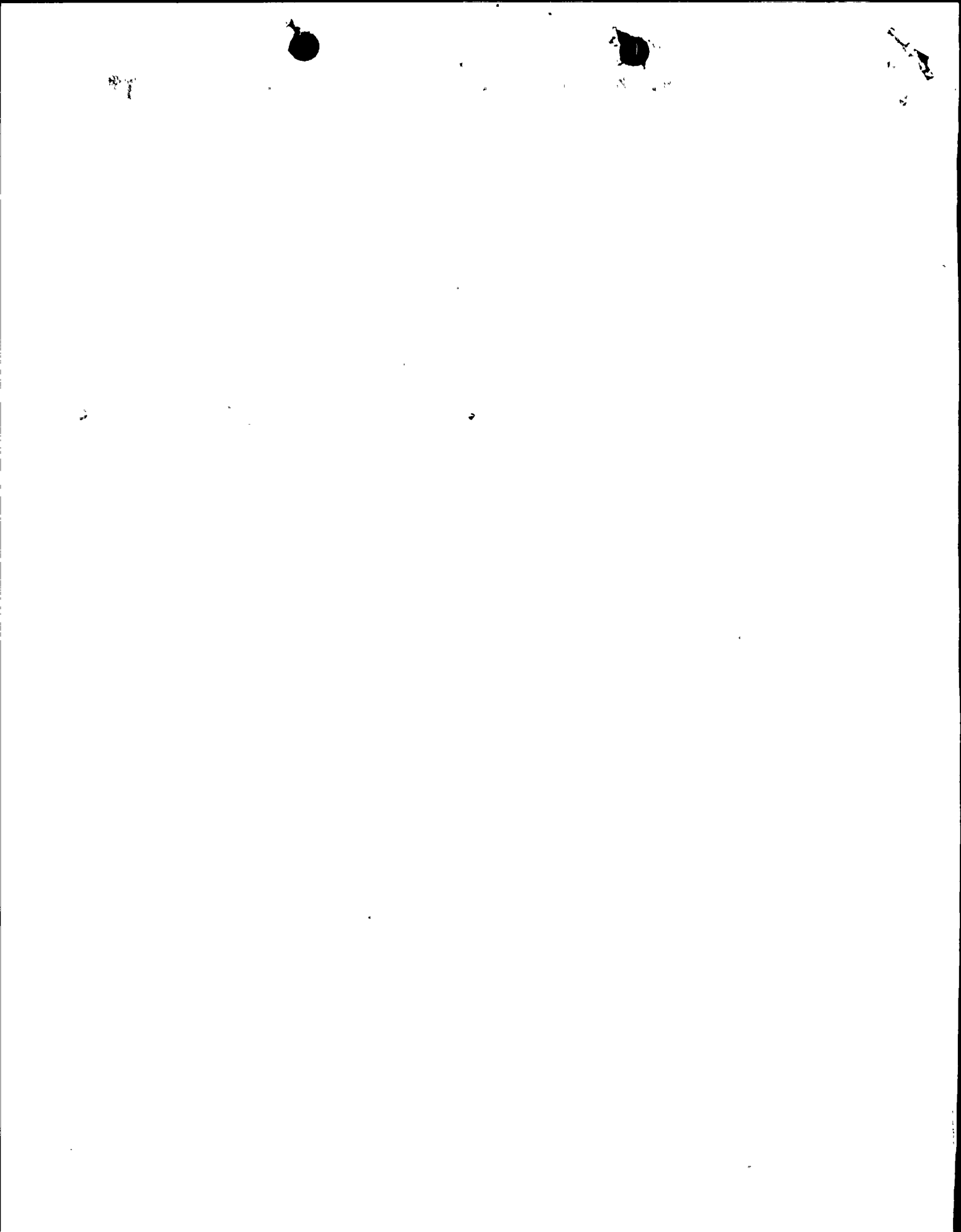
ORIGINAL SIGNED BY  
P. F. COLLIER

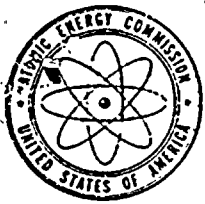
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ATTACHMENT A  
REQUEST FOR ADDITIONAL INFORMATION  
NIAGARA MOHAWK POWER CORPORATION  
DOCKET NO. 50-220

1. Is it the intention of Niagara Mohawk to include in the lecture series lectures on quality assurance for operation? We believe this subject should be covered. In addition, provision for contingency lectures should be made. These are lectures covering upcoming activities such as, major maintenance outages, refueling, etc.
2. Is it the intention of Niagara Mohawk to require a combination of reactivity manipulations to meet the required ten manipulations in two years, or would Niagara Mohawk expect credit for a single method of reactivity performed ten times? We believe a combination of manipulations more closely meets the intent of the regulation. Further, a startup should be to the point of adding heat, a shutdown should be a plant shutdown and any power changes should be in manual rod control and of at least ten percent. In addition, we do not consider control rod exercise to be an acceptable manipulation for credit.
3. Provide justification for not removing an employee from licensed duties when his examination results clearly indicate the need for an accelerated requalification program. We believe he should be removed from licensed duties; however, we don't believe the duration of the accelerated program needs to be specified. The scope and duration of the accelerated program can be determined by facility management on a case-by-case basis.
4. Provide the passing grade for the periodic written examinations. We believe this should be 80% as it is for the individual sections on the annual comprehensive examinations and that an individual scoring less than that should repeat the specific lecture or reading assignment covered on the examination.
5. Describe the participation of the licensed staff members in the requalification program.





UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

*Facility  
File*

August 22, 1973

Docket No. 50-220

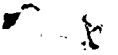
Niagara Mohawk Power Corporation  
ATTN: Mr. Philip D. Raymond,  
Vice President-Engineering  
300 Erie Boulevard West  
Syracuse, New York 13202

Gentlemen:

The Atomic Energy Commission has amended its regulations to require operators who manipulate the controls of nuclear power reactors and other licensed production and utilization facilities to complete a requalification program or be administered a reexamination before each license renewal and to require facility licensees to carry out adequate operator requalification programs.

An amendment to Part 55 of the AEC's regulations, which was published in the Federal Register on August 17, 1973 establishes minimum requirements for requalification programs including on-the-job training and lectures on basic theories and safety systems of the specific plant. It applies to both operators and senior operators.

An amendment to Part 50 published in the Federal Register on the same date requires that an application for a license to operate a facility must include a description and plans for implementation of an operator requalification program. Also, each facility licensee must have such a program in operation within three months after issuance of an operating license. Holders of operating licenses in effect on the effective date of these amendments, September 17, 1973, must submit an operator requalification program for Commission approval and concurrently implement that program within three months after the effective date of the amendments.




Niagara Mohawk Power Corp.

-2-

All power, test and reprocessing facility licensees are requested to submit the required information with one signed original and thirty-nine additional copies. If a single submittal covers more than one unit at a station, one signed original and fifty-nine additional copies should be provided. All other facility licensees are requested to submit one signed original and twenty-one additional copies.

Sincerely,



D. J. Skovholt, Assistant Director  
for Operating Reactors  
Directorate of Licensing

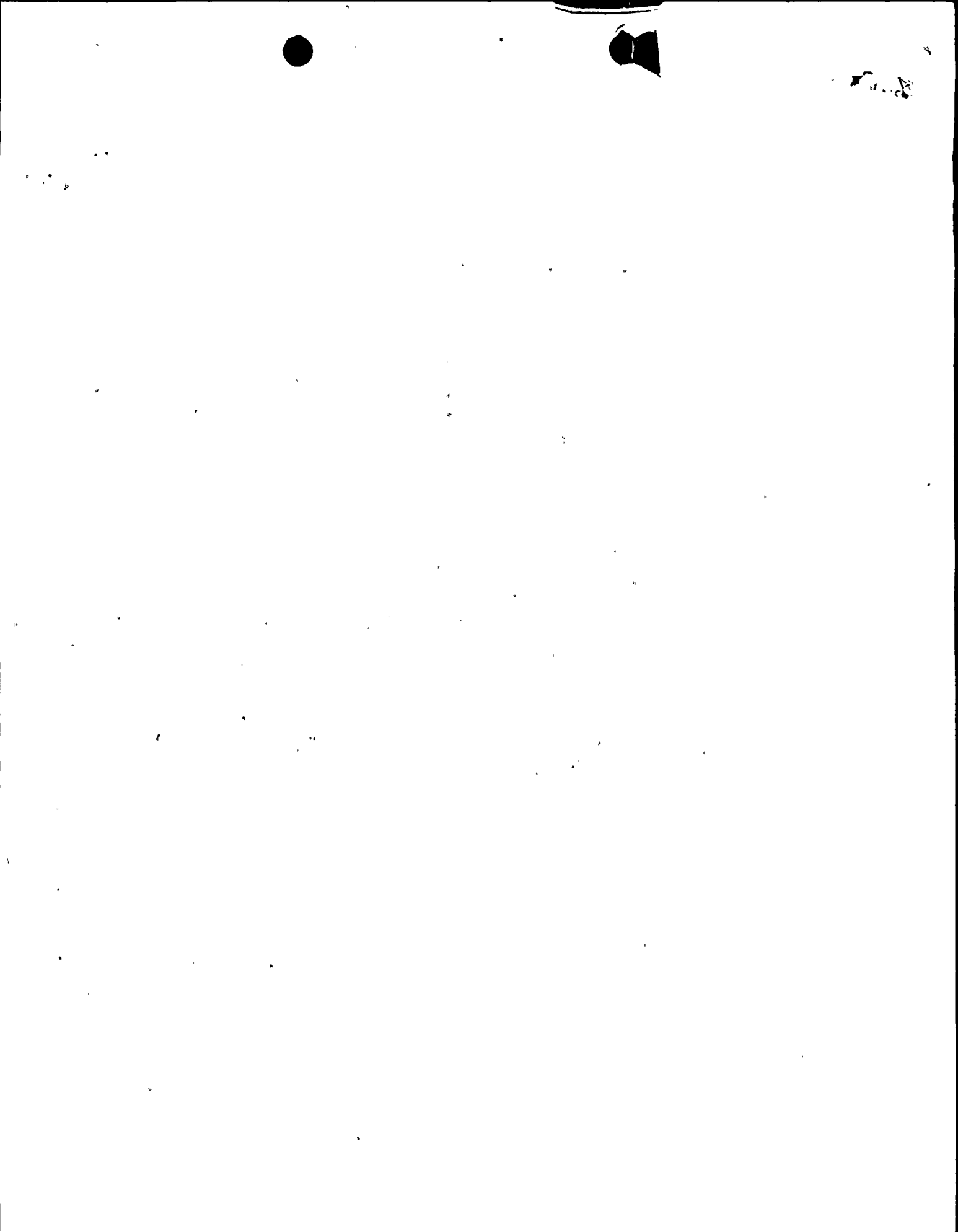
Enclosure:

1. Amendment to 10 CFR Part 50 and Part 55

cc: Arvin E. Upton, Esquire  
Attorney  
LeBoeuf, Lamb, Leiby & MacRae  
1821 Jefferson Place N.W.  
Washington, D.C. 20036

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D.L. Ziemann



MAR 1 0 1973

Dennis L. Ziemann, Chief, Operating Reactors Branch No. 2, L  
THRU: P. F. Collins, Chief, Operator Licensing Branch, L

**NINE MILE POINT-EMERGENCY DIESEL STARTING**

In the process of administering operator exams at the subject facility in February 1973, Mr. E. Conners, OLB, was advised that a switch located on the control panels at the diesels could be in the wrong position and that this wrong position would not be annunciated in the main control room. This appears to be unusual, since similar switch at other power plants are normally annunciated in the control room if they are not positioned properly.

The subject switch is called a "Selector" switch and has two positions called "Local" and "Remote". When the D/G are in their normal standby condition and ready to start on loss of 115 KV; the subject selector switch is to be in the "remote" position per Operating Procedure 45-4.

It is my understanding that if the switch should be in the wrong position (Local), the control room would not be alerted. At present, it is not known definitely whether this switch can negate automatic starting of the D/G but we understand it will negate manual D/G starting from the control room.

Mr. E. Conners is on leave and will return March 20, if any further information is desired.

*S/RJ.Campbell*

R. J. Campbell  
OLB-BWR Group Leader  
Directorate of Licensing

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PFCollins  
C. DeBevec  
Facility File ✓

OFFICE ▶	OLB:L Ext. 7476 <i>RJC</i>	OLB:L <i>PFC</i>				
SURNAME ▶	RJCampbell:hb	PFCollins				
DATE ▶	3/13/73	3/13/73				

THE UNITED STATES OF AMERICA  
DEPARTMENT OF JUSTICE

OFFICE OF THE ATTORNEY GENERAL

MEMORANDUM FOR THE ATTORNEY GENERAL  
SUBJECT: [Illegible]

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[Illegible]  
[Illegible]

[Illegible text]

[Illegible text]



FEB 7 1973

DISTRIBUTION:

OLB READING File  
PD File  
RP Reading File  
Docket File-Facility ✓

Niagara Mohawk Power Corporation  
ATTN: Mr. Thomas J. Perkins,  
Station Superintendent  
Nine Mile Point Nuclear Station  
Post Office Box 32  
Lycoming, New York 13093

Gentlemen:

We have reviewed the fifteen operator and one senior operator license applications for the Nine Mile Point Nuclear Station. We find the applications to be in order with the following exceptions.

None of the applications include the details on course of instruction, including the number of course hours, administered by the facility licensee pursuant to Section 55.10(a)(6) of 10 CFR Part 55.

Please submit the required information for each applicant. If you prefer, you may submit one letter indicating the above information with a cover letter listing the applicants. Please submit three (3) copies of the cover letter for each individual. In addition, Mr. Paterson's application indicates that he has not manipulated the reactor controls during reactor startups. We require that applicants manipulate the reactor controls through at least two training startups in order to be eligible to sit for an examination.

Please provide us with Mr. Paterson's startup experience or your plans to have him obtain this experience.

Sincerely,

ORIGINAL FILED IN  
P. F. COLLINS

Paul F. Collins, Chief  
Operator Licensing Branch  
Directorate of Licensing

OFFICE ▶	OLB:L				
	Ext. 7486				
SURNAME ▶	PECollins:hb.				
DATE ▶	2/7/73				

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CONFIDENTIAL

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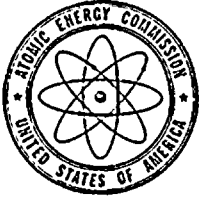
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CONFIDENTIAL



UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

December 12, 1972

~~Skovholt~~  
~~R. G. Foster~~  
~~D. Denton~~  
~~CAMPBELL~~  
FACILITY  
FILE

D. L. Ziemann, Chief, Operating Reactors Branch #2, L

NIAGARA MOHAWK POWER CORPORATION (NINE MILE POINT) - DOCKET NO. 50-220  
INSULATION FIRE IN FEEDWATER SYSTEM

Enclosed for your information is a copy of an Inquiry Report from our Region I (Newark) Office concerning an insulation fire that occurred in the turbine building at the Nine Mile Point reactor facility on September 13, 1972. The fire was limited to the insulation on the feedwater supply line beneath the feedwater pump and resulted from a leak in the oil supply line to the turbine driven feedwater pump.

The licensee made selected inspections of relays and contacts in the turbine building that might have been affected by the fire and confirmed that no damage resulted to these components. In addition, visual inspection of the feedwater piping during a subsequent shutdown revealed no indications of problems.

As discussed with you earlier, the licensee will report this occurrence in the next Semi-annual Report.

*J. G. Keppeler*  
J. G. Keppeler

Enclosure:

RO Inquiry Report  
No. 50-220/72-08

cc: L. R. Rogers, RS  
R. S. Boyd, L (2)  
R. C. DeYoung, L (2)  
~~D. J. Skovholt, L (3)~~  
D. R. Muller, L  
H. R. Denton, L (2)  
R. L. Tedesco, L  
R. H. Vollmer, L

J. M. Hendrie, L  
R. W. Houston, L  
P. A. Morris, RO, w/o encl.  
H. D. Thornburg, RO, w/o encl.  
R. H. Engelken, RO, w/o encl.  
J. P. O'Reilly, RO:I  
DR Central Files



RO INQUIRY REPORT NO. 50-220/72-08

Subject: Niagara Mohawk Power Corporation  
Nine Mile Point 1-BWR

License No.: DPR-17

Title: Fire Insulation on Supply Line to Feewater Pump

Prepared by: R. T. Cantrell  
F. S. Cantrell, Reactor Inspector

9/20/72  
Date

A. Date and Manner AEC was Informed

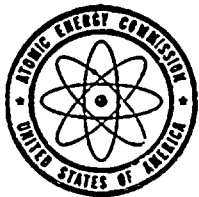
September 14, 1972 by telephone call from Mr. Roland Smith, Maintenance Supervisor.

B. Description of Particular Event or Circumstance

Mr. Smith stated that a smoldering insulation fire was found in the turbine building at 5:30 p.m. September 13, 1972. As a result of a leak in the oil supply line to the turbine driven feedwater pump, the insulation on the feedwater supply line beneath the pump became soaked with oil. The oil leak had been repaired during a shutdown, September 9-10, 1972. The fire was centered in the pipe sleeve in the concrete below the feedwater pump. Mr. Smith stated that the fire, the source of which was believed to be autoignition, did not pose a threat to the safe operation of the plant, or equipment in the vicinity of the fire. Operation of the plant continued throughout the occurrence.

C. Action by Licensee

1. A maintenance crew was assembled and, using respiratory protection, the insulation was dug out of the pipe sleeve from below. The oil soaked insulation burst into flames as it was exposed to the oxygen in the air; however, personnel were standing by with fire extinguishers to extinguish the fire as the insulation was removed. Approximately 5 feet of insulation was removed.

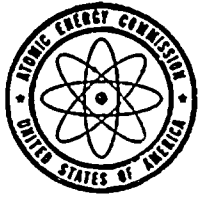


UNITED STATES  
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DIVISION OF COMPLIANCE  
REGION I  
970 BROAD STREET  
NEWARK, NEW JERSEY 07102

201 645-

*Mr. Smith for magnesium metal  
to be used in  
in piping material  
Probably surface oxidation  
J.C.*

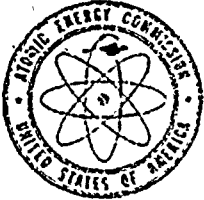
2. Mr. Smith stated that the only threat to equipment was a cable tray located approximately 6 feet west of the water pipe and that this was only a threat during the period while insulation was being removed.
3. A visual inspection of the carbon steel piping indicated no damage.
4. The oil on the insulation was determined to be Mobil DTE-797 turbine oil with a minimum flash point of 405°F. Mr. Smith stated that the vendor would be contacted to determine how autoignition could occur with temperatures in the vicinity of 350°F.
5. Mr. Smith stated that NMP did not plan to make a separate report of this event; however, the details would be covered in the semi annual report of operation.



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UNITED STATES  
ATOMIC ENERGY COMMISSION  
DIRECTORATE OF REGULATORY OPERATIONS  
REGION I  
970 BROAD STREET  
NEWARK, NEW JERSEY 07102

5 Karbol (2)  
~~DeWitt~~  
G. Curtis  
~~(a) Campbell~~  
~~11/11/72~~  
FAC FILE

J. G. Keppler, Chief, Reactor Testing & Operations Branch  
Directorate of Regulatory Operations, HQ

RO INQUIRY REPORT NO. 50-220/72-11Q  
NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT 1

The subject inquiry report is forwarded for your information.

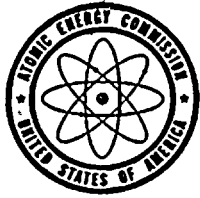
A special site inspection is in progress to review the details of this occurrence and the licensee's actions preparatory to resumption of operations. Our inspection findings will be documented in an inspection report to be submitted in the near future. The licensee will submit a written report (10 days) to Licensing. Distribution will be made by this office to the PDR, LPDR, NSIC, DTIE and State representatives after review by the licensee for proprietary information.

*R. T. Carlson*  
R. T. Carlson, Chief,  
Reactor Operations Branch

Enclosure:

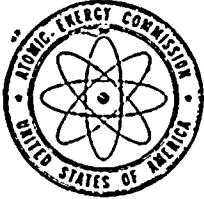
Subject Inquiry Report No. 50-220/72-11Q

- cc: RO Chief, Reactor Testing & Operations Branch, HQ
- RO:HQ (5)
- DR Central Files
- Regulatory Standards (3)
- Directorate of Licensing (13)
- PDR
- Local PDR
- NSIC
- DTIE
- State of New York



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DIRECTORATE OF REGULATORY OPERATIONS  
REGION I  
970 BROAD STREET  
NEWARK, NEW JERSEY 07102

RO Inquiry Report No. 50-220/72-110

Licensee: Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202

License No.: DPR-17

Facility: Nine Mile Point

Descriptive Title: Equipment Failure - Apparent Drift In Safety Valve Setpoint

Prepared by:

*T. Young, Jr.*  
T. Young, Jr., Reactor Inspector

11/31/72  
Date

A. Date and Manner AEC was Informed:

By telephone call from the licensee on November 19, 1972.

B. Description of Particular Event or Circumstance:

A primary system safety valve mounted on the reactor vessel head actuated following a turbine trip and reactor scram at the Nine Mile Point reactor on November 19, 1972. Actuation of the safety valve resulted in release of some primary steam to the containment drywell. The following preliminary information was provided by the licensee:

At 5:30 a.m. on November 19, 1972, while operating at full power (620 MWe) a turbine trip and reactor scram occurred while performing routine surveillance tests on the turbine thrust bearing wear detector. A faulty bypass switch on this detector caused the turbine trip. The main steam isolation valves remained open and the turbine bypass valves opened to control pressure; however, the reactor pressure increased to a maximum of 1083 psig, at which point one safety valve prematurely relieved for a period of nine seconds (valve setpoint - 1226 psig). The electromatic relief valves did not operate (lowest valve setpoint - 1085 psig). Drywell pressure increased from 0.7 to

1226  
- 1083  
-----  
143 psi < S.P.



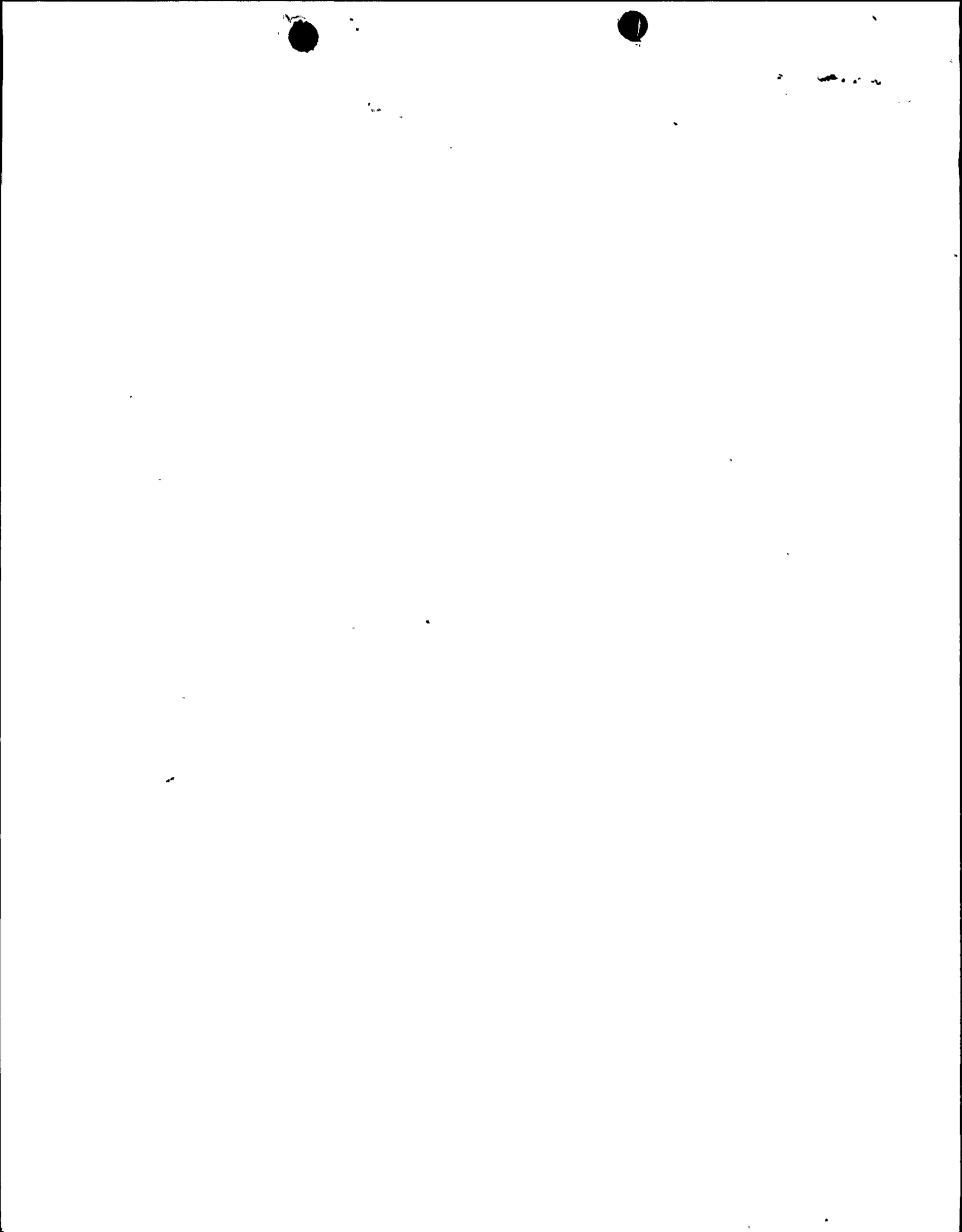
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2.6 psig (high drywell pressure alarm setpoint - 3.5 psig). Approximately 300 gallons of water were collected in the floor drain collector tank as a result of the steam release to the containment drywell.

Damage appears to be limited to insulation on the reactor vessel head, some paint on the drywell wall, and the reference chamber on the containment leak rate test equipment. No significant radioactivity releases to the environment or personnel exposures resulted from the occurrence. The plant was brought to cold shutdown conditions.

C. Action by Licensee:

1. The faulty safety valve is being replaced with a spare, and this valve will be disassembled to determine the cause of the apparent setpoint drift.
2. The insulation on the reactor vessel head and the reference chamber on the containment leak rate test equipment will be repaired.
3. The licensee will submit a written report within 10 days, to the Directorate of Licensing pursuant to paragraph 6.7.1 of the Technical Specifications.



No. 75

Date 11/24/72

DIRECTORATE OF REGULATORY OPERATIONS  
NOTIFICATION OF AN INCIDENT OR OCCURRENCE

CAMPBELL  
**FACILITY FILE**

Facility: NIAGARA MOHAWK POWER CORPORATION (NINE MILE POINT 1)

Problem:

RO Region I (Newark) was informed by the licensee by telephone on November 19, 1972, that a primary system safety valve had actuated earlier that day following a reactor scram at the Nine Mile Point reactor. The actuation of the valve resulted in release of primary steam to the containment drywell. The following preliminary information was provided by the licensee:

At 5:30 a.m. on November 19, 1972, with the reactor operating at full power (620 Mwe), a turbine trip occurred resulting in a reactor scram. The turbine trip resulted from a faulty bypass switch in the turbine protective circuitry. During the ensuing transient, reactor pressure increased to a maximum of 1083 psig, at which point one of the safety valves installed on the reactor vessel head (setpoint 1226 psig), opened prematurely for a period of 9 seconds. The drywell pressure increased to 2.6 psig and the maximum drywell temperature was 120°F. There was no significant change in reactor water level and the occurrence did not initiate actuation of any of the engineered safeguards. Approximately 300 gallons of primary water were collected in the drywell floor drain collector tank. Containment integrity was maintained until the atmosphere in the drywell had been sampled to determine that radioactivity levels were sufficiently low to permit release. The effect of the steam release was limited to minor insulation damage on the reactor vessel head and some of the paint on the drywell wall. No significant radioactivity releases to the environs or personnel exposures resulted from the occurrence.

The plant is presently in the cold shutdown condition. The cause of the premature actuation of the safety valve is being investigated by the licensee. The licensee plans to return the reactor to power following replacement of the malfunctioning safety valve and completion of the investigation of the occurrence.

Action:

1. An RO inspector is at the site to obtain detailed information on the occurrence. Further action by RO will be based on the results of the inspection.
2. The state of New York and the Northeast Office of the Division of Public Information have been informed by telephone.
3. Commissioner Ramey's Technical Assistant, Commissioner Doub's Technical Assistant and the Staff of the Joint Committee on Atomic Energy are being informed by copy of this notification.





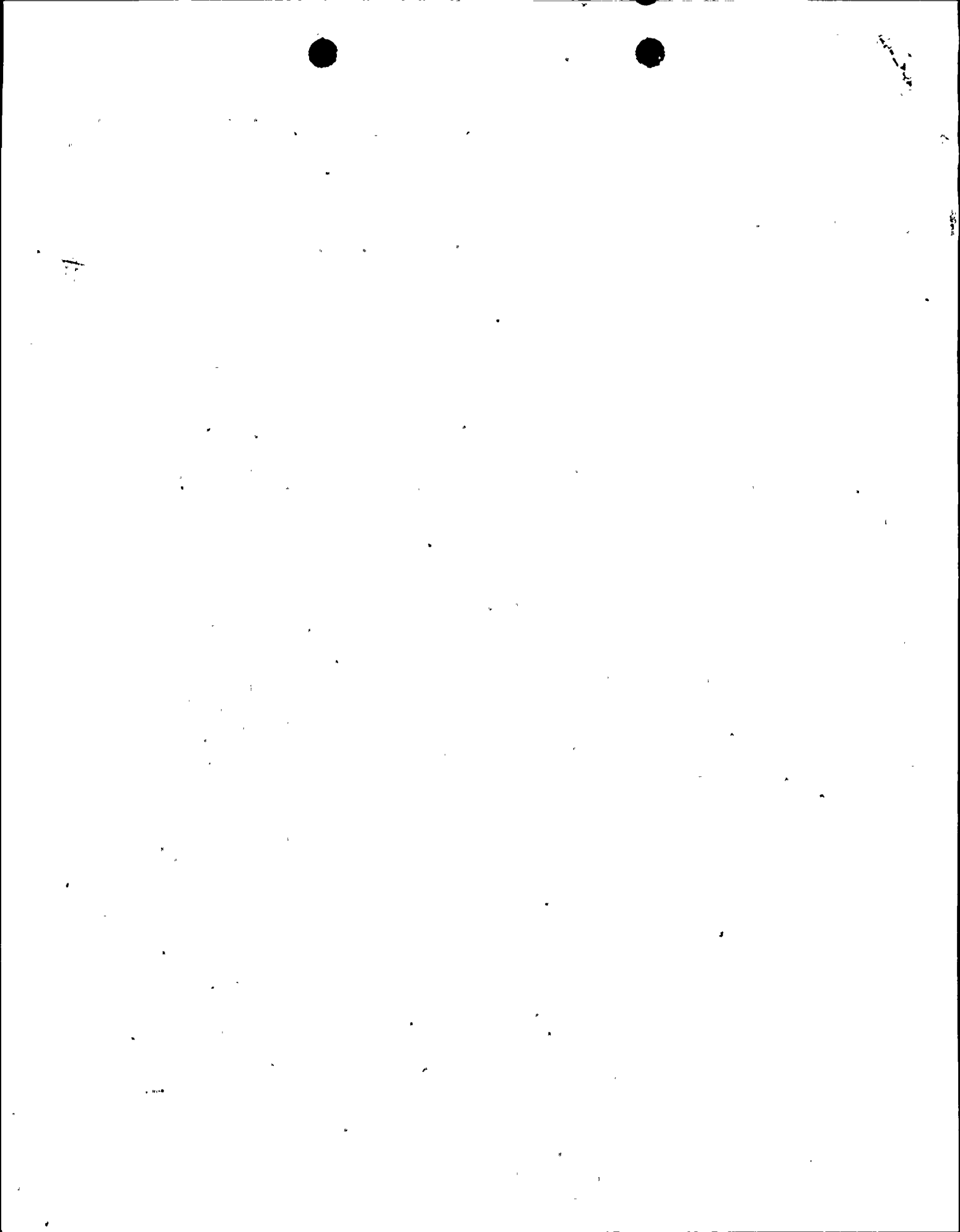
Contact:

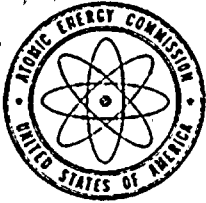
Further information on this problem can be obtained from:

D. C. Kirkpatrick - X7421  
J. G. Keppler - X7421  
R. H. Engelken - X7356

Distribution:

Chairman Schlesinger	M. Biles, OS	D. A. Nussbaumer, L
Commissioner Ramey	R. F. Fraley, ACRS (3)	R. E. Cunningham, L
Commissioner Larson	L. R. Rogers, RS	F. E. Kruesi, RO
Commissioner Doub	J. B. Minogue, RS	P. A. Morris, RO
Commissioner Ray	J. J. Davis, RS	H. D. Thornburg, RO
L. M. Muntzing, DR	J. F. O'Leary, L	T. R. Wilson, RO
E. J. Bloch, DDR	E. G. Case, L	R. H. Engelken, RO
C. K. Beck, DRGL	J. M. Hendrie, L	D. Thompson, RO
S. H. Hanauer, DRTA	A. Giambusso, L	R. D. O'Neill, OCR
D. J. Donoghue, DRA	F. Schroeder, L	J. R. Totter, DBER
W. MacDonald, OPS	S. H. Smiley, L	J. D. Goldstein, DBER
General Manager (2)	R. S. Boyd, L	G. A. Arlotto, RS
Secretary (2)	D. J. Skovholt, L	S. Levine, OEA
J. Fouchard, IS	R. C. DeYoung, L	D. L. Ziemann, L
M. Shaw, RDT	D. R. Muller, L	RO Regional Offices
J. A. Harris, IS	D. F. Knuth, L	DR Reading File
E. J. Bauser, JCAE	R. R. Maccary, L	REG Central File
J. H. Rubin, AGM	R. L. Tedesco, L	PDR
J. D. Anderson, INS	H. R. Denton, L	Local PDR
F. Ingram, IS		





UNITED STATES  
ATOMIC ENERGY COMMISSION  
DIRECTORATE OF REGULATORY OPERATIONS  
REGION I  
970 BROAD STREET  
NEWARK, NEW JERSEY 07102

*P. Collins*  
*CAMPBELL*  
*McA*  
FACILITY FILE

SEP 29 1972

J. G. Keppler, Chief, Reactor Testing & Operations Branch  
Directorate of Regulatory Operations

RO INQUIRY REPORT NO. 50-220/72-09  
NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT 1 - BWR  
EQUIPMENT FAILURE - SHUTDOWN DUE TO SEAL LEAKAGE

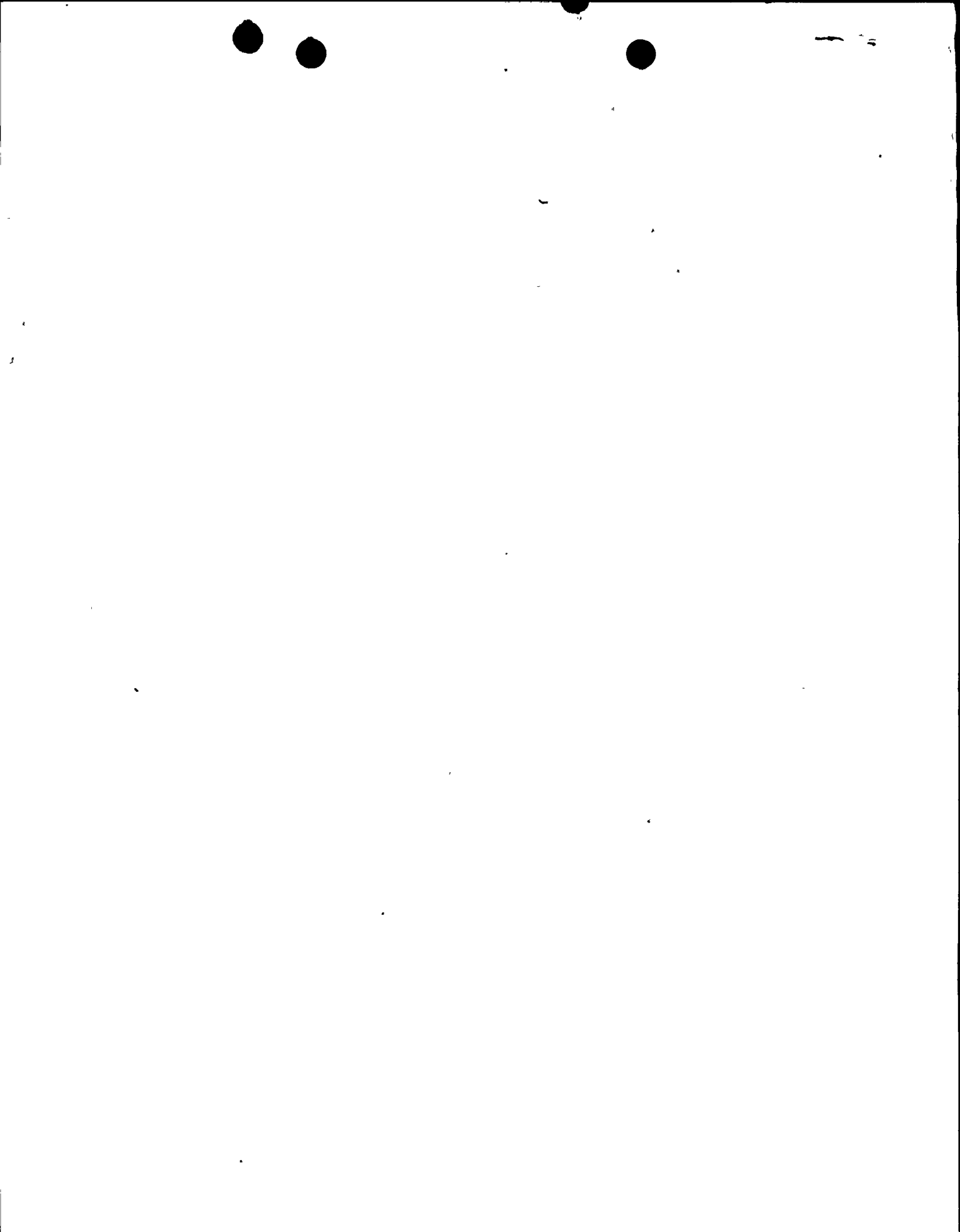
The subject inquiry report is forwarded for your information.

Technical Specifications require the licensee to monitor the reactor coolant system leakage once per day; however, due to the approach to limits, the licensee was monitoring leakage once per hour. The licensee indicated that when the reading showed 25.0 gpm a reactor shutdown was initiated. During the next inspection, we intend to pursue the question of adequacy of conservatism in the licensee's actions.

*R. T. Carlson*  
R. T. Carlson  
Acting Sr. Reactor Inspector  
Reactor Operations Branch

Enclosure:  
Subject Inquiry Report

cc: R. S. Minogue, RS (3)  
R. S. Boyd, L (2)  
R. C. DeYoung, L (2)  
✓ D. J. Skovholt, L (3)  
H. R. Denton, L (2)  
P. A. Morris, RO  
H. D. Thornburg, RO  
R. H. Engelken, RO  
RO Files  
DR Central Files



RO Inquiry Report No. 50-220/72-09

Subject: Niagara Mohawk Power Corporation

Facility: Nine Mile Point 1 - BWR

License No.: DPR-17

Title: Equipment Failure - Shutdown Due to Seal Leakage

Prepared by: *F. S. Cantrell*  
F. S. Cantrell, Reactor Inspector

9/29/72  
Date

A. Date & Manner AEC was Informed:

September 21, 1972 by telephone call from Mr. P. A. Burt, General Superintendent.

B. Description of Particular Event or Circumstance:

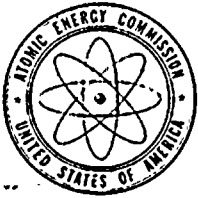
Mr. Burt stated that the total\* leakage increased from 12 gpm on September 18 to 25.0 gpm at 5:00 p.m. on September 20 (Technical Specification limit - 25.0 gpm). When the limiting leak rate was reached, a controlled reactor shutdown was initiated. Power was leveled at a reactor pressure of 300 psi in order to be able to identify the source of the leakage, which was determined to be the No. 11 recirculation pump seal. Seal leaks were also identified on two recirculation pump isolation valves. Mr. Burt stated that leakage decreased as reactor power and pressure were lowered, and that 25.0 gpm was the maximum total leak rate observed.

C. Action by Licensee:

Valves and pump packings were replaced and/or tightened. The valves were checked for operability and the limitorque settings were adjusted as necessary. During the check out, the shaft in the limitorque operator of the No. 11 pump suction valve failed. (See RO Inquiry Report No. 50-220/72-10Q.) The licensee plans to submit one written 10 day report to cover both events.

\*Identified and unidentified.





UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

*P. Collins*  
~~CAMPBELL~~

August 24, 1972

FILE

D. L. Ziemann, Chief, Operating Reactors Branch #2, L

NIAGARA MOHAWK POWER CORPORATION (NINE MILE POINT) - DOCKET NO. 50-220  
FAILURE OF REFUELING GRAPPLER

Enclosed for your information is a copy of an Inquiry Report from our Region I (Newark) Office concerning a failure of the refueling grapples on August 14, 1972, at the Nine Mile Point reactor facility.

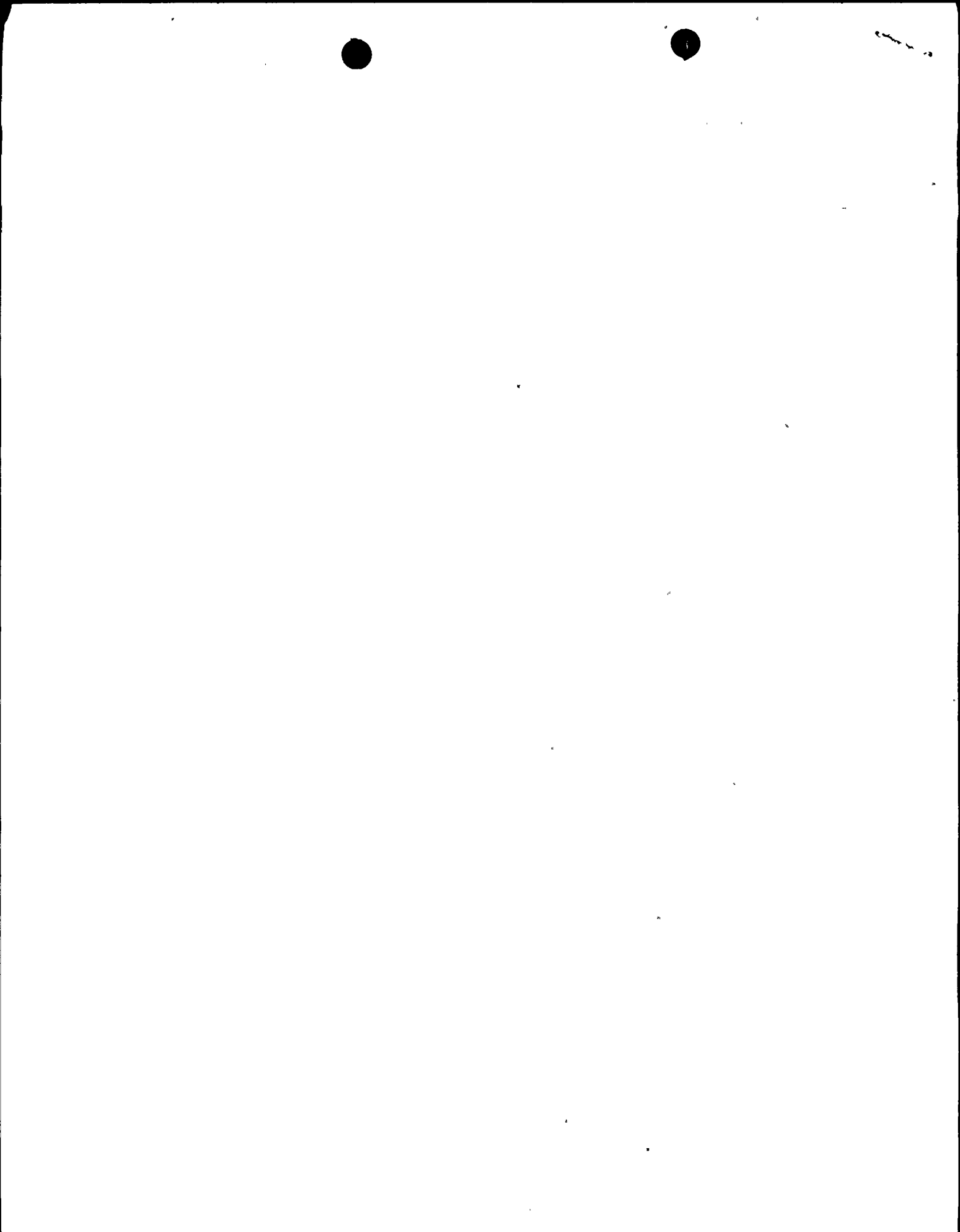
We are currently considering issuing a Reactor Operating Experience report to all licensees regarding this failure.

*J. G. Keppler*  
J. G. Keppler, Chief,  
Reactor Testing and Operations  
Branch  
Directorate of Regulatory Operations

Enclosure:

RO Inquiry Report  
No. 50-220/72-07

cc: L. R. Rogers, RS (3)  
R. S. Boyd, L (2)  
R. C. DeYoung, L (2)  
~~D. J. Skovholt, L (3)~~  
H. R. Denton, L (2)  
P. A. Morris, RO, w/o encl.  
H. D. Thornburg, RO, w/o encl.  
R. H. Engelken, RO, w/o encl.  
J. P. O'Reilly, RO:I  
J. G. Davis, RO:II  
B. H. Grier, RO:III  
J. W. Flora, RO:IV  
R. W. Smith, RO:V  
R. T. Carlson, RO:I, w/o encl.  
DR Central Files





RO Inquiry Report No. 50-220/72-07

Subject: Niagara Mohawk Power Corporation

Facility: Nine Mile Point 1 - BWR

License No.: DPR-17

Title: Equipment Failure - Refueling Grappler

Prepared by: *F. S. Cantrell, Jr.*  
F. S. Cantrell, Reactor Inspector

8/15/72  
Date

A. Date and Manner AEC was Informed:

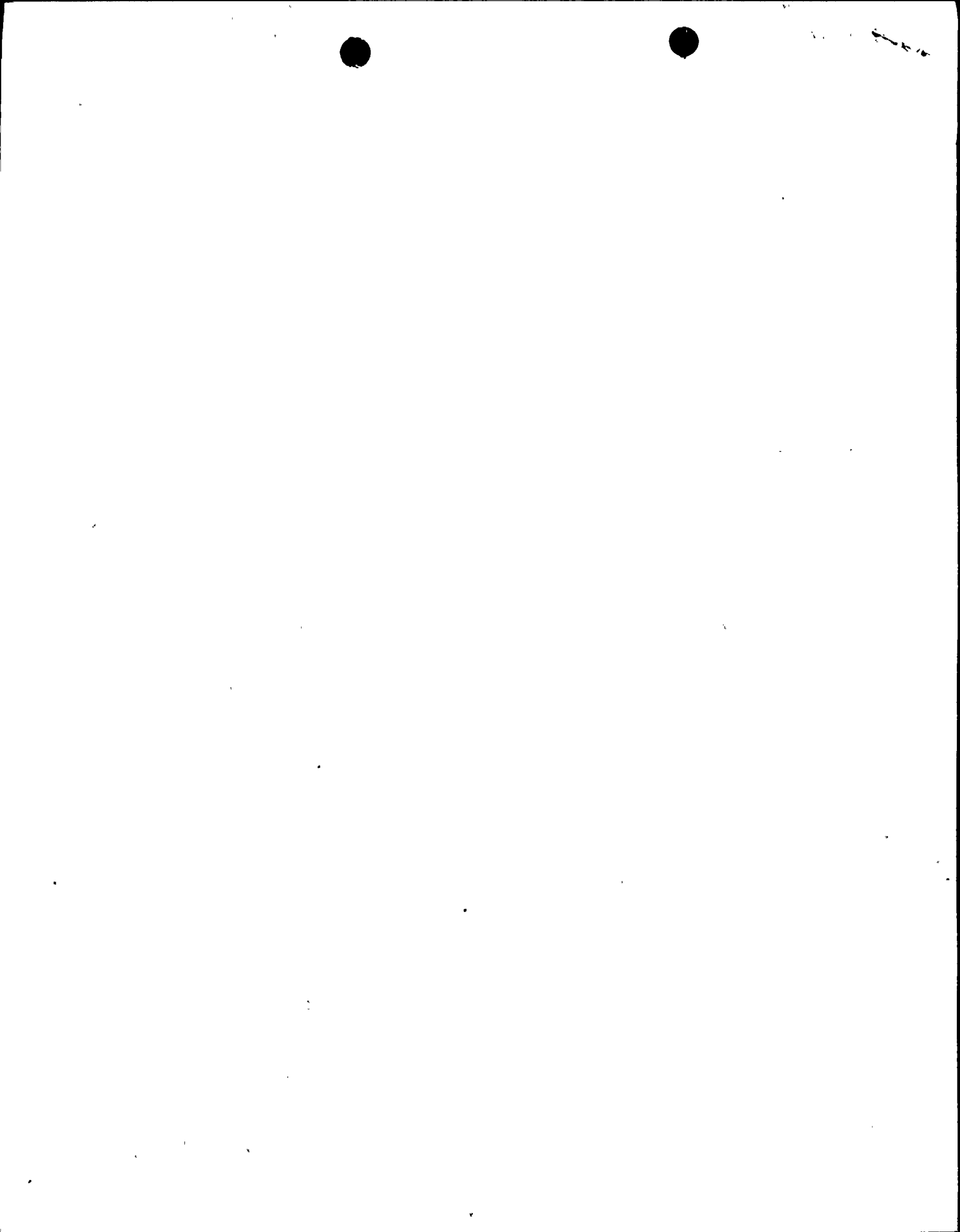
August 14, 1972, by telephone call from Mr. T. Perkins, Station Superintendent.

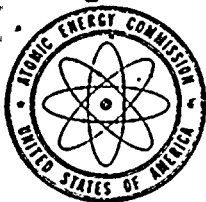
B. Description of Particular Event or Circumstance:

A piece of wood that was attached to a viewing float in the spent fuel pool to provide buoyance became water logged and sank. The refueling grappler was used in an attempt to retrieve the wood from the spent fuel pool. During retrieval, the "up drive" limit failed. The drive cable broke, allowing the grappler to fall on an empty spent fuel rack.

C. Action by Licensee:

The licensee stated that he planned to submit an information report to the Commission on this failure after it had been investigated by NMPC and General Electric. This investigation is scheduled for August 15, 1972.





UNITED STATES  
ATOMIC ENERGY COMMISSION  
DIRECTORATE OF REGULATORY OPERATIONS  
REGION I  
970 BROAD STREET  
NEWARK, NEW JERSEY 07102

*Detachable*  
*Cantrell*  
*D. Collins*  
*Campbell*  
*MSA*  
**FACILITY FILE**

RO Inquiry Report No. 50-220/72-100

Licensee: Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202

License No.: DPR-17

Facility: Nine Mile Point 1 - BWR  
Scriba, New York

Descriptive Title: Equipment Failure - Broken Shaft in Operator for  
Recirculation Pump Suction Valve

Prepared by: *F. S. Cantrell*  
F. S. Cantrell, Reactor/Inspector

9/29/72  
Date

A. Date and Manner AEC was Informed:

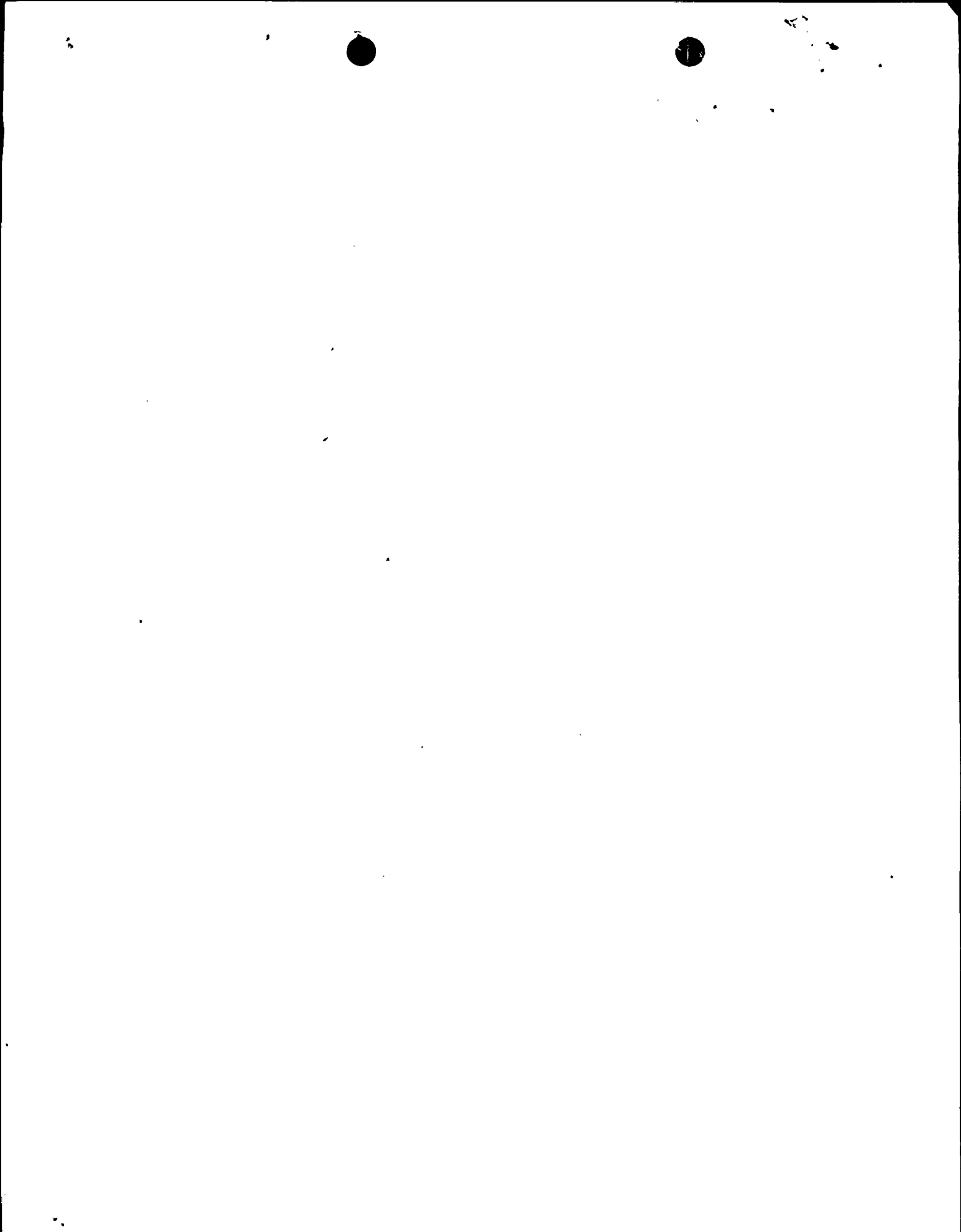
September 25, 1972, by telephone call from the licensee.

B. Description of Particular Event or Circumstance:

The shaft in the motor operator for the No. 11 recirculation pump suction valve failed during verification of operability following replacement of the valve packing and a re-adjustment of the limit switches in the operator. The plant was shut down at the time of the failure to repair reactor coolant system leaks in the drywell. The failed valve operator is a Limitorque operator, Model SMA-51.

C. Action by Licensee:

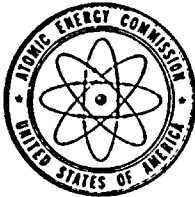
1. The valve operator was removed and the valve manually closed. With both the suction and discharge valves in the line in the closed position, reactor startup was initiated on September 24, 1972 to approximately 86% of authorized power.
2. Assuming the availability of replacement parts for this valve operator, the plant will be shut down to effect repair



during the weekend of October 6, 1972.

3. A vendor's representative examined the failed shaft and other components of the subject operator. The preliminary evaluation indicated that the shaft metal had crystalized prior to failure.
4. The cause of failure will be included in the licensee's report (10 day) to the Directorate of Licensing.





UNITED STATES  
ATOMIC ENERGY COMMISSION  
DIRECTORATE OF REGULATORY OPERATIONS  
REGION 1  
970 BROAD STREET  
NEWARK, NEW JERSEY 07102

*Skovholt*  
*Carter*  
*Collins*  
~~CAMPBELL~~  
~~McM~~  
FACILITY FILE

AUG 8 1972

J. G. Keppler, Chief, Reactor Testing & Operations Branch  
Directorate of Regulatory Operations, HQ

RO INQUIRY REPORT NO. 50-220/72-06  
NIAGARA MOHAWK POWER CORPORATION  
NINE MILE POINT 1 - BWR  
TECHNICAL SPECIFICATION VIOLATION

The subject inquiry report is forwarded for your information.

A routine telephone contact was made with Mr. Burt at 8:45 a.m. on July 31, 1972. At that time, the licensee did not know the precise time that the 24 hour period would expire. In a followup telephone call by the inspector at 10:15 a.m., he stated that the period expired at 10:00 a.m. The 10:00 a.m. time was also given in a phone conversation between Mr. Burt and myself at 11:00 a.m. In a subsequent telephone call from Mr. Burt at 2:45 informing us that the O<sub>2</sub> concentration was 3.9%, the 7:00 a.m. figure for the end of the 24 hour period was provided. The licensee was advised during all of the phone conversations that in order to get relief from the less than 5% O<sub>2</sub> after 24 hours requirement he would have to request a Technical Specification change from Licensing. He did not contact Licensing until after 10:15 a.m. He stated he was advised that it would be safer to remain at a constant power than to shut down.

We plan to discuss this matter further with NMP management in the near future.

The licensee plans to submit a 10 day written report to Licensing.

*R. T. Carlson*  
R. T. Carlson, Chief  
Reactor Operations Branch

Enclosure:  
Subject Inquiry Report

cc: R. Minogue, RS (3)  
R. S. Boyd, L (2)  
R. C. DeYoung, L (2)  
✓ B. J. Skovholt, L (3)  
H. R. Denton, L (2)  
P. A. Morris, RO  
H. D. Thornburg, RO  
R. H. Engelken, RO  
RO Files  
DR Central Files



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RO Inquiry Report No. 50-220/72-06

Subject: Niagara Mohawk Power Corporation

Facility: Nine Mile Point - BWR

License No.: DPR-17

Title: Technical Specification Violation - High Oxygen Concentration -  
Drywell

Prepared by: *F. S. Cantrell*  
F. S. Cantrell, Reactor Inspector

8/7/72  
Date

A. Date and Manner AEC was Informed:

July 31, 1972, by several telephone conversations with Mr. P. A. Burt, General Superintendent.

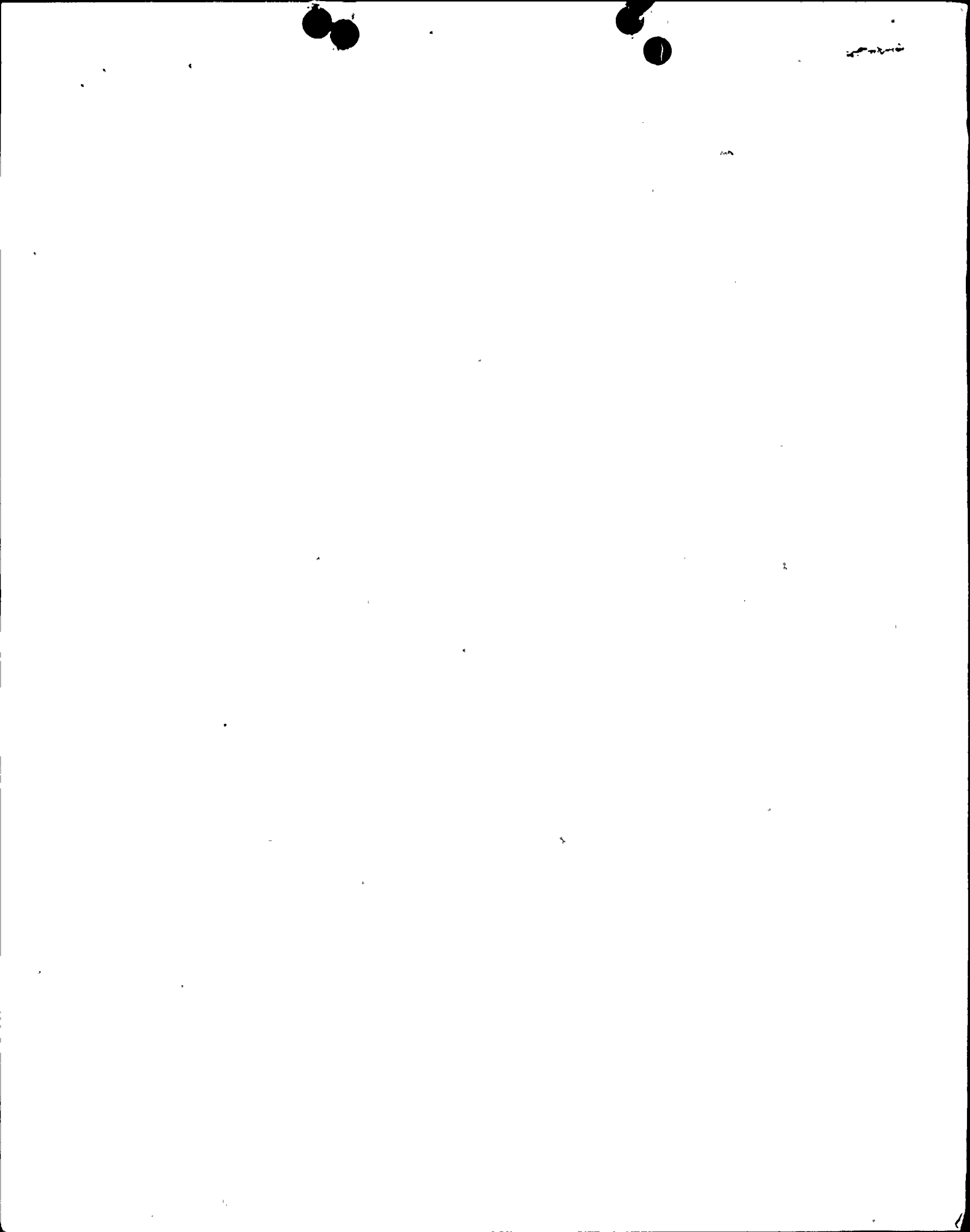
B. Description of Particular Event or Circumstance:

Mr. Burt stated that NMP-1 had ordered nitrogen (from Ohio) to inert the drywell; however, it did not appear that the trucks would arrive in time to reduce the O<sub>2</sub> content of the drywell to less than 5% within the 24 hour period after startup as required by Technical Specifications. Power level was 345 MWe (approximately 55%). Subsequently, it was determined that the "24 hour period" elapsed at 7:00 a.m. on July 31, 1972. The nitrogen supply arrived prior to 11:00 a.m. and purging the drywell with nitrogen started immediately. Analysis showed that the O<sub>2</sub> content had been reduced to 3.9% at 1:45 p.m.

C. Action by Licensee:

Reactor power level was held at 345 MWe until the O<sub>2</sub> content in the drywell was reduced to less than 5%.

The licensee plans to submit a 10 day written report as required by the Technical Specifications.



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 Branch R/F  
 ✓ Fac. File  
 DJSkovholt, DRL

JUN 4 1971

Dennis L. Ziemann, Chief, ORB #2  
 THRU: P. F. Collins, Chief, OLB

**ROD WORTH MINIMIZER**

During the demonstration examination at Niagara Mohawk Power Corporation's Nine Mile Point Reactor on May 15, 1971, I observed improper operating procedures and a possible Technical Specification violation in regard to operation with the RWM inoperative.

I was informed in the briefing that the RWM was out of service and this function would be performed by a licensed operator. When the startup demonstration began, the "RWM" operator held the only copy of the rod withdrawal sequence and called out the rod movements to the examinee who never checked the printed sequence.

Under Surveillance Requirement on page 23, the Technical Specification requires, "If the rod worth minimizer is not operable, a second licensed operator or qualified technical station employee shall verify that the operator at the reactor console is following the rod program." The same wording is used in the Operating Procedures on page 37-5. While looking at the procedure, I asked the examinee if the RWM function had been performed correctly when he was operating the reactor. He said, "This is the way we have always done it."

It was demonstrated that the procedure being used did not provide RWM protection. The RWM operator called for rod 26-07 to be moved from notch 4 to 8. When the rod was selected, it was still at notch 0; having been skipped on the previous pull. The "RWM" operator said it was his fault.

My concern about the RWM program was expressed at the exit interview with Tom Lempges, Operating Supervisor. Mr. Lempges indicated that the RWM supplied by GE had never functioned satisfactorily. He said that part of the problem was poor maintenance in that not enough time was allotted during outages to get the RWM running. I informed him that other GE plants, referring specifically to Monticello, had made their RWM work. Mr. Lempges seemed to agree with my concern on this subject but made no indication of positive correction action.

*Original signed by  
 E. L. Conner*

Eben L. Conner, Jr.

OFFICE ▶		OLB:DRL	Operator Licensing Branch	OLB:DRL
cc: DJSkovholt		<i>[Signature]</i>	Division of Reactor Licensing	<i>[Signature]</i>
SURNAME ▶		PFCollins:pf		ELConner:pf
DATE ▶		6/3/71		6/3/71

DEPARTMENT OF  
COMMERCE  
WASHINGTON, D.C.

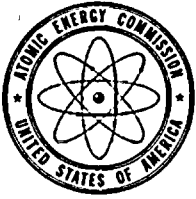
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*Handwritten signature or initials*

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UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

February 12, 1970

TO LICENSEES OF OPERATING NUCLEAR POWER STATIONS

We are currently reevaluating the methods used by facility licensees to ensure that licensed personnel at operating nuclear power stations are retaining competence in all areas of reactor operation, including emergency situations. In addition, we are assessing the effects of the different designs and expected operating schedules of current generation plants on the maintenance of operator competence.

Many of the nuclear power stations that are presently under construction and/or planned are designed to operate base-loaded almost continuously for two years between fuel loadings. In such cases, the members of the operating staff will have little opportunity to participate in transient and infrequent operations such as startups, shutdowns, and power changes. Hence, the value of operating experience, per se, in maintaining competence will be diminished and the importance of retraining will be emphasized to a greater extent than at present.

The facility licensee is responsible for ensuring the continued competence of the operating staff. Our regulations provide that the Commission audit the initial determinations of competence of operators and senior operators by means of examination and licensing and periodically audit the licensee's determinations of continued competence every two years in the process of operator license renewal. The eligibility of a licensed operator for renewal of his license is dependent, in part, upon the Commission's determination that he is capable of continuing to discharge his responsibilities competently and safely. The amount and type of retraining that is provided bears on the probability of continued competence.

Although the applications for renewal of operator and senior operator licenses often contain some information on retraining of the individuals involved, they do not state the prescribed retraining requirements of the facility licensee nor do they indicate the methods employed by managements to evaluate the programs and their results. We request that you provide a comprehensive description of your facility's



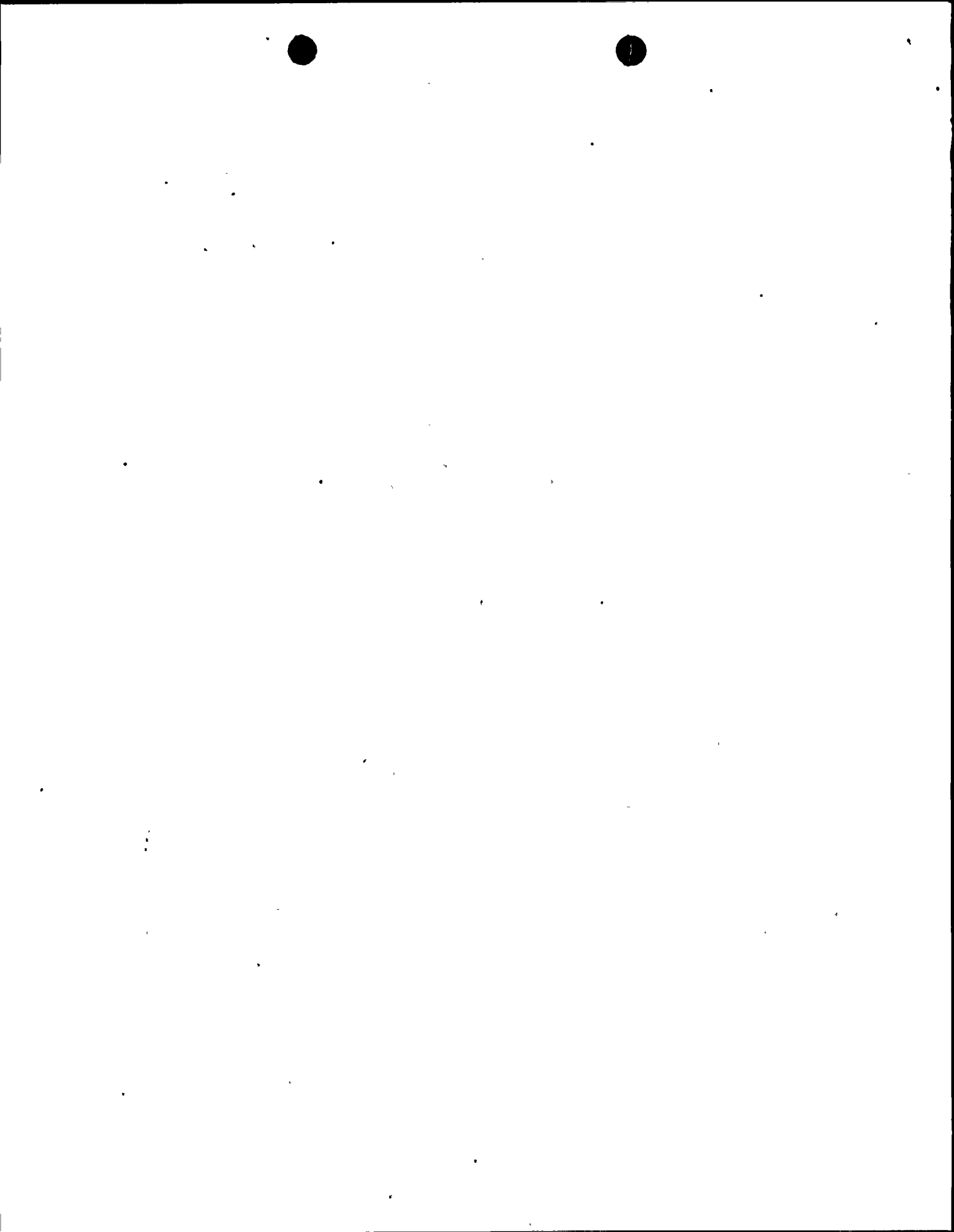
TO LICENSEES OF OPERATING NUCLEAR  
POWER STATIONS

-2-

February 12, 1970

retraining program, including the methods employed to evaluate the performance of individuals. The information should include, but not necessarily be limited to, the following:

1. Do you conduct a formal retraining program consisting of scheduled classroom lectures? If so, what subjects are covered? What is the length and frequency of the lectures? Are examinations administered and results recorded?
2. Who is responsible for developing and for administering the retraining program? Who conducts each of the various phases of the program?
3. What mechanism is used to assure that all members of the operating staff are knowledgeable of new procedures, procedure changes, license changes, unusual occurrences and equipment malfunctions?
4. Do you have a requirement that operators perform certain tasks, such as reactor startups and shutdowns, periodically as operations permit? If so, are records maintained? Do you evaluate their performance?
5. Do you require periodic simulation of emergency procedures by the shift crews? If so, do you evaluate their performance? Are records maintained? What other specific aspects of the retraining program serve to maintain operator competence to respond properly in the unlikely event of a major accident?
6. Do you conduct periodic fire drills and/or evacuation drills? Is performance evaluated?
7. Do you receive any assistance from organizations other than your own in conducting the retraining program? What is their function?
8. What training aids, e.g. simulators, films, video tapes, programmed learning machines or texts, models, are utilized in the program? How?





TO LICENSEES OF OPERATING NUCLEAR  
POWER STATIONS

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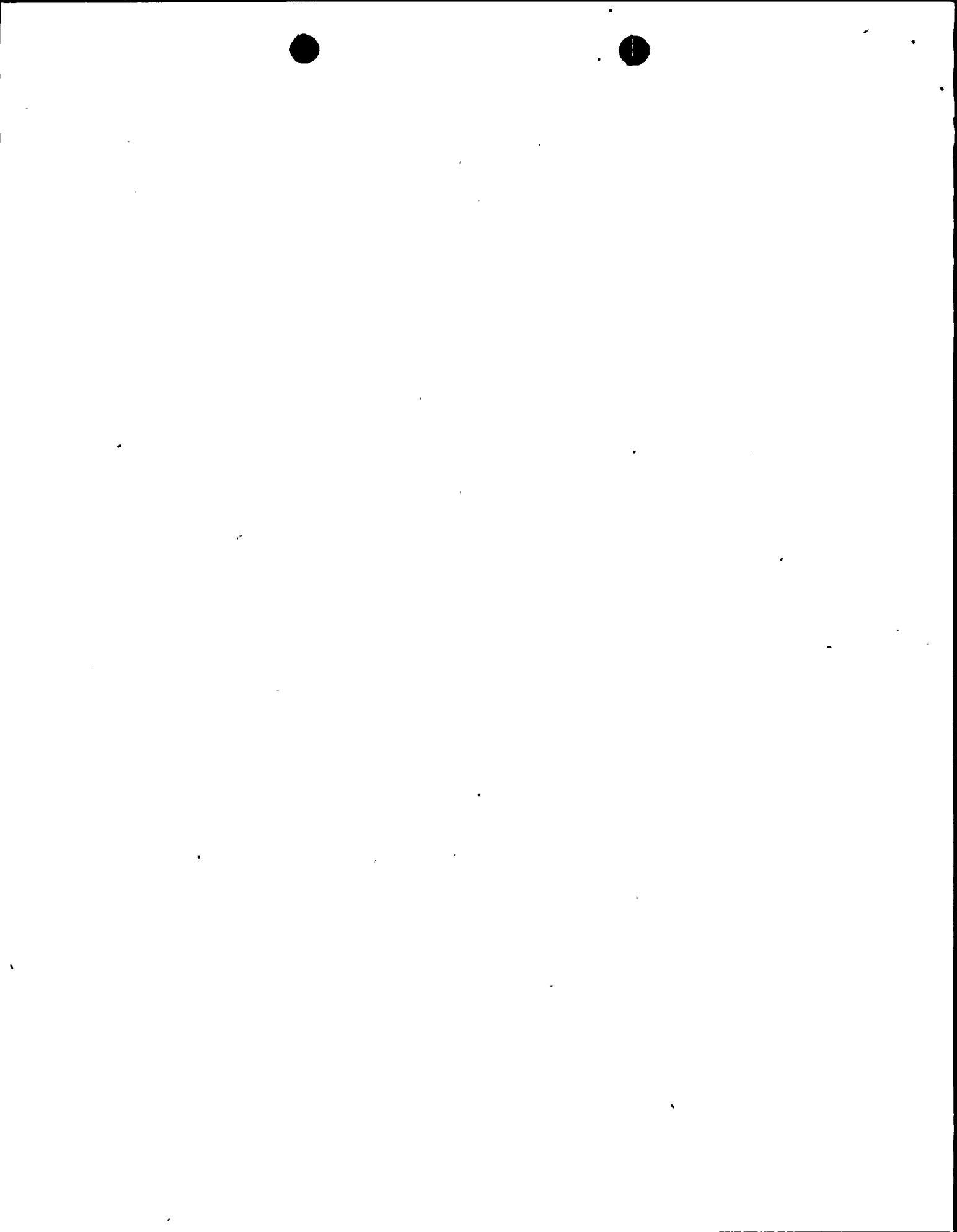
February 12, 1970

9. Distinguish between the minimum retraining requirements for operators and senior operators.
10. How do you evaluate individual competence in support of the certifications that you submit in applications for operator and senior operator licenses?

We would appreciate receiving the retraining program descriptions within sixty days of the date of this letter.

*Peter A. Morris*

Peter A. Morris, Director  
Division of Reactor Licensing



ADDRESSEE LIST FOR LETTER TO  
LICENSEES OF OPERATING NUCLEAR POWER STATIONS

Docket No. 50-3

Consolidated Edison Company of New York, Inc.  
4 Irving Place  
New York, New York 10003  
Attention: Mr. William J. Cahill, Jr.  
Vice President

Docket No. 50-238

First Atomic Ship Transport Inc.  
River and First Streets  
Hoboken, New Jersey 07030  
Attention: Mr. R. O. Mchann  
Executive Vice President

Docket No. 50-220

Niagara Mohawk Power Corporation  
300 Erie Boulevard West  
Syracuse, New York 13202  
Attention: Mr. Minot H. Pratt  
Vice President & Executive Engineer

Docket Nos. 50-10 (Unit #1) & 50-237 (Unit #2)

Commonwealth Edison Company  
One First National Plaza  
Chicago, Illinois 60690  
Attention: Mr. Henry E. Bliss  
Nuclear Licensing Administrator

Docket No. 115-5

Dairyland Power Cooperative  
2615 East Avenue South  
La Crosse, Wisconsin 54601  
Attention: Mr. John P. Madgett  
General Manager



Docket No. 50-155

Consumers Power Company  
212 West Michigan Avenue  
Jackson, Michigan 49201

Attention: Mr. Robert L. Hauter  
Electric Production  
Superintendent - Nuclear

Docket No. 50-133

Pacific Gas & Electric Company  
245 Market Street  
San Francisco, California 94106

Attention: Mr. Richard H. Peterson  
Senior Vice President & General Counsel

Docket No. 50-219

Jersey Central Power & Light Company  
Madison Avenue at Punch Bowl Road  
Morristown, New Jersey 07960

Attention: Mr. George H. Ritter  
Vice President

Docket No. 50-146

Saxton Nuclear Experimental Corporation  
Post Office Box 542  
Reading, Pennsylvania 19603

Attention: Mr. C. R. Montgomery  
General Manager

Docket No. 50-213

Connecticut Yankee Atomic Power Company  
Post Office Box 270  
Hartford, Connecticut 06101

Attention: Mr. Donald C. Switzer  
Vice President

Docket No. 50-171

Philadelphia Electric Company  
1000 Chestnut Street  
Philadelphia, Pennsylvania 19105

Attention: Mr. Vincent P. McDevitt  
Vice President & General Counsel



Docket No. 50-244

Rochester Gas & Electric Corporation  
89 East Avenue  
Rochester, New York 14604  
Attention: Mr. E. J. Nelson  
Vice President

Docket No. 50-29

Yankee Atomic Electric Company  
441 Stuart Street  
Boston, Massachusetts 02116  
Attention: Mr. L. E. Minnick  
Vice President

Docket No. 50-231

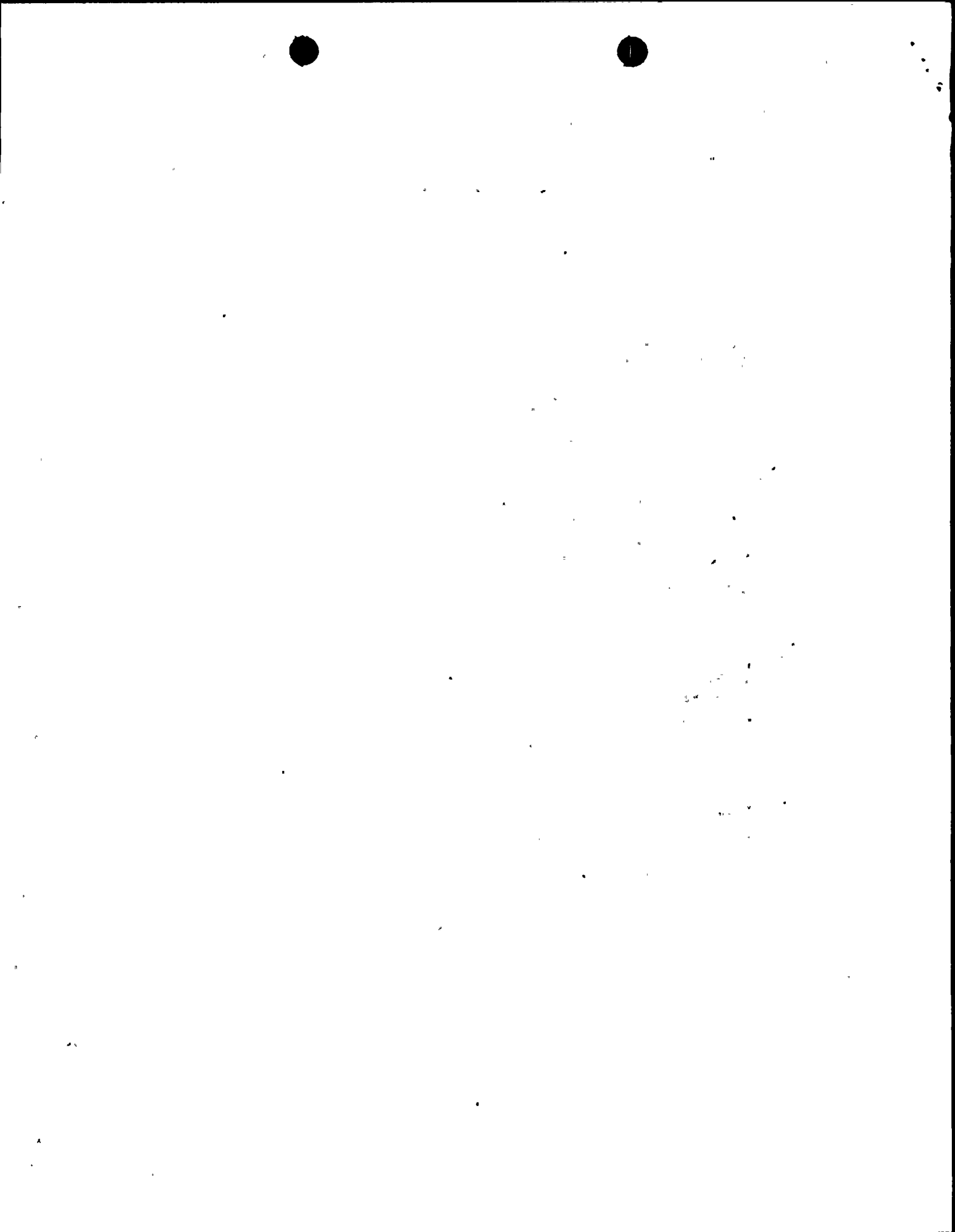
General Electric Company  
Advanced Products Operation  
310 DeGuigne Drive  
Sunnyvale, California 94806  
Attention: Mr. Karl P. Cohen  
General Manager

Docket No. 50-16

Power Reactor Development Company  
1911 First Street  
Detroit, Michigan 48226  
Attention: Mr. Myron C. Beckman  
General Manager

Docket No. 50-206

Southern California Edison Company  
P. O. Box 351  
Los Angeles, California 90053  
Attention: Mr. J. B. Moore  
Vice President





*Facility*

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D. J. Skovholt, DRL

OCT 9 1969

Mr. P. Allister Burt  
Station Superintendent  
Nine Mile Point Nuclear Plant  
Niagara Mohawk Corporation  
P. O. Box 32  
Lycoming, New York 13093

Dear Mr. Burt:

We have received and reviewed the fifteen (15) applications for operator licenses and two (2) reapplications for senior operator licenses.

We find the applications to be in order with the following exceptions:

The operator license applications do not contain (1) the details of the training program and (2) the startup and shutdown experience pursuant to subparagraph 55.10(a)(6) of 10 CFR 55.

The reapplications for senior operator licenses do not contain the details of the additional training the applicants have received pursuant to paragraph 55.12(a) of 10 CFR 55.

This information is necessary for us to determine the eligibility of the applicants to be administered the examinations requested in their applications. Consequently, examinations cannot be administered prior to receipt of the above information.

Upon the receipt of this information we will continue to process the applications.

Sincerely yours,

ORIGINAL SIGNED BY  
P. F. COLLINS  
Paul F. Collins, Chief  
Operator Licensing Branch  
Division of Reactor Licensing

cc: R. J. Campbell, DRL

OFFICE ▶	OLB:DRL	DRL				
SURNAME ▶	<i>PFC</i> PFCollins:eh	<i>DJS</i> DJSkovholt				
DATE ▶	10/8/69	10/8/69				

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D. J. Skovholt, DRL  
R. J. Campbell, OLB

MAY 27 1969

Mr. P. Allister Burt  
Superintendent  
Nine Mile Point Nuclear Station  
P. O. Box 32  
Lycoming, New York 13093

Dear Mr. Burt:

This is in regard to the forthcoming "cold" operator license examinations which will be administered at the Nine Mile Point Station.

We have begun the writing of the written examinations. However, as we advised you earlier, it is necessary that we have a complete set of approved operating, emergency, and special procedures for the Nine Mile Point Station before we can complete this task. At present, we have received the majority of the operating and emergency procedures, but none of the special procedures. It is imperative that we receive all of the approved procedures no later than June 2, 1969, in order to administer the written examinations and "cold" operating tests as presently scheduled.

The examining team will be led by AEC Headquarters Examiner Robert J. Campbell. All matters pertaining to scheduling dates and times for the examinations should be discussed with Mr. Campbell or myself.

Sincerely yours,

*of P. F. Collins, Acting*  
Frank L. Kelly, Chief  
Operator Licensing Branch  
Division of Reactor Licensing

OFFICE ▶	OLB:DRL	DRL	DRL			
SURNAME ▶	<i>P.F.C.</i> PFCollins:jh	<i>P.F.C. for</i> FLKelly	<i>MS for</i> DJSkovholt			
DATE ▶	5/27/69	5/25/69	5/27/69			

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UNITED STATES GOVERNMENT

# Memorandum

*Kelly*  
*ES Campbell*  
*Fac File*

TO : Roger S. Boyd, Assistant Director  
for Reactor Projects, DRL  
(THRU) Robert L. Tedesco, Chief, RPB-2, DRL *R*

FROM : V. Stello, RPB-2, DRL and *V Stello*  
R. L. Ferguson, E&CB, DRS *R L Ferguson*

SUBJECT: MINUTES OF STAFF-NIAGARA MOHAWK POWER CORPORATION  
(NINE MILE POINT) MEETING - DECEMBER 19, 1967.  
DOCKET NO. 50-220

DATE: January 3, 1968

A meeting was held on December 19, 1967 with representatives of Niagara Mohawk Power Corporation (NMPC), GE and members of the staff. An agenda indicating specific topics to be discussed at the meeting was made available to NMPC two weeks prior to the meeting. A number of topics noted on the agenda were based on problems identified during the Oyster Creek review. We anticipated that the applicant would have been informed of these problems and therefore would have been prepared to discuss them in detail. ~~A summary of the discussion on each agenda item is presented herein.~~ In addition, NMPC stated that fuel loading was scheduled for June 1, 1968. A list of attendees is attached.

## Conclusions

1. The problem areas identified on the Oyster Creek design have not been corrected on the Nine Mile Point Plant.
2. NMPC did not provide sufficient information to assure the staff that the problems would be corrected.
3. There is a schedule conflict, since our scheduled date for license issuance is July 1, 1968, and NMPC's scheduled fuel loading date is June 1, 1968.
4. Our review should be expedited by:
  - (a) Presenting DRL positions on the problem areas to NMPC as soon as possible.\*
  - (b) Completing our review as rapidly as possible.\*\*

\* On December 20, 1967, R. Boyd met with key personnel of NMPC. It was agreed we would present NMPC with the DRL positions established for these problem areas. NMPC would consider these positions, comply with those they found acceptable and prepare arguments for those they found unacceptable. Copies of letters to OC stating these positions were given to NMPC for their information.

\*\* NMPC review plan is not yet approved. Reviewers for specific areas are not yet assigned.





1. The first part of the document discusses the importance of maintaining accurate records of all transactions. It emphasizes that this is crucial for ensuring the integrity of the financial statements and for providing a clear audit trail. The text notes that any discrepancies or errors in the records can lead to significant financial losses and legal complications.

2. The second part of the document outlines the specific procedures for recording transactions. It details the steps involved in identifying the source of the transaction, determining the amount, and recording it in the appropriate ledger account. The text also discusses the importance of using consistent accounting methods and standards to ensure comparability of the data over time and across different periods.

3. The third part of the document addresses the issue of reconciling the records. It explains that regular reconciliations are necessary to identify and correct any errors or omissions in the records. The text provides a step-by-step guide for performing a reconciliation, including comparing the records to the source documents and resolving any differences.

4. The fourth part of the document discusses the role of internal controls in maintaining accurate records. It highlights that a strong system of internal controls is essential for preventing and detecting errors and fraud. The text describes various control measures, such as segregation of duties, authorization requirements, and regular audits, that can be implemented to enhance the reliability of the records.

5. The fifth part of the document concludes by summarizing the key points discussed. It reiterates the importance of accurate record-keeping and the need for a robust system of internal controls. The text also provides some final thoughts on the overall goal of maintaining high-quality financial records to support the organization's success and growth.

Summary of Discussions

## I. Engineered Safety Features

## A. Core Spray

## 1. System Operations -

- a. System operating sequence without off-site power is the same as that with off-site power available. It appears to depend upon a single sequencer which starts the core spray pumps in a serial sequence. The redundant set of pumps will not be started until the pressure measurements show the first set is not effective ( $\sim 34$  seconds after the start of the sequence). On standby power, only one diesel generator will be loaded unless the preferred diesel does not start or the core spray pumps do not come up to pressure. The method by which the sequencer determines what is happening was not explained. The consequences of interlock failures or instrument failures due to pipe whip or missiles have not been analyzed.
- b. The core spray nozzles have been changed from conical or shaped sprays to alternate conical and open nozzles.
- c. A high drywell pressure signal has been added to initiate the core spray system.

2. Passive Failures - A failure of some passive components (i.e. torus and suction lines for the core spray and containment spray pumps) would result in a total loss of water to these pumps. Pump motors would not be flooded because the maximum possible water level is below the motor elevation. Motors are capable of operating if submerged.

3. Containment Flooding - Time did not permit discussion in detail.

## 4. Programming Automatic Relief System -

- a. The Reactor Vessel Water Level Trip point has not been lowered per our review of the similar system on Oyster Creek.
- b. The applicant does not consider AC powered valve operators (or the equivalent) necessary to positively prevent the automatic relief when AC is not available. The applicant believes redundancy in the AC system justifies the assumption that AC power will always be available to the plant.



The following information was obtained from the records of the  
 Bureau of the Census, Department of Commerce, Washington, D. C.  
 for the year 1954:

State	Population	Area (sq. miles)	Density (per sq. mile)
Alabama	2,049,000	52,420	39.1
Alaska	100,000	588,000	0.17
Arizona	1,000,000	113,970	8.8
Arkansas	1,500,000	53,170	28.2
California	10,000,000	163,690	61.1
Colorado	1,500,000	104,030	14.4
Connecticut	2,000,000	5,540	361.0
Delaware	500,000	2,480	201.6
District of Columbia	200,000	680	294.1
Florida	3,000,000	57,060	52.6
Georgia	2,500,000	59,730	41.9
Idaho	1,000,000	84,360	11.9
Illinois	10,000,000	149,990	66.7
Indiana	4,000,000	36,420	109.8
Iowa	2,500,000	71,480	35.0
Kansas	2,500,000	82,270	30.4
Kentucky	3,000,000	40,360	74.3
Louisiana	2,500,000	52,430	47.7
Maine	1,000,000	33,090	30.2
Maryland	3,000,000	12,160	247.1
Massachusetts	4,000,000	8,010	500.0
Michigan	6,000,000	96,860	62.0
Minnesota	4,000,000	225,180	17.8
Mississippi	2,000,000	46,920	42.6
Missouri	4,000,000	69,700	57.4
Montana	1,000,000	147,040	6.8
Nebraska	2,000,000	77,340	25.9
Nevada	500,000	110,600	4.5
New Hampshire	1,000,000	9,340	107.1
New Jersey	6,000,000	19,270	311.9
New Mexico	1,000,000	121,740	8.2
New York	15,000,000	47,190	317.8
North Carolina	4,000,000	50,810	78.7
North Dakota	1,000,000	70,690	14.1
Ohio	6,000,000	44,820	133.9
Oklahoma	2,000,000	69,560	28.9
Oregon	1,500,000	98,380	15.3
Pennsylvania	10,000,000	46,050	217.2
Rhode Island	1,000,000	1,540	649.4
South Carolina	2,000,000	32,020	62.5
South Dakota	1,000,000	77,100	13.0
Tennessee	4,000,000	42,330	94.5
Texas	8,000,000	695,620	11.5
Utah	1,000,000	165,690	6.0
Vermont	1,000,000	9,610	104.1
Virginia	4,000,000	40,770	98.1
Washington	2,500,000	71,300	35.1
West Virginia	1,500,000	62,030	24.2
Wisconsin	4,000,000	65,490	61.1
Wyoming	1,000,000	97,810	10.3



January 3, 1968

5. The Environmental Design Conditions - The conditions for electrical components were given as:

<u>Component</u>	<u>Pressure</u>	<u>Temperature</u>	<u>Humidity</u>
Pump motors		150°F	100%
Valve Operators		310°F	100%
Instrumentation & Controls	62 psi	150°F	100%

Valve Operators have Class H insulation and are presumed to operate continuously with the above design conditions. The applicant stated that adequate test data is available to confirm the performance of the components under these design conditions.

6. Small Breaks - The plant response to small breaks in the primary coolant system will be as stated in the FSAR. The feedwater system will be modified so that it will be available during a LOCA if off-site power is available. The current arrangement, feedwater pump driven by the main steam turbine, precludes the availability of feedwater even if off-site power is available. Without off-site power, the plant will be blowdown and core spray will provide water.
7. Core Spray Test - The core spray test program has been completed.
8. Preoperational and Operational Tests - The Preoperational and Operational Testing of the core spray system will consist of an operational test from sensor to final actuators. It will be performed with both off-site and standby power.
9. The Quality Control Program - The quality control program for all engineered safeguards is under NMPC, however, time did not permit discussion of this item in detail.

A list of systems for which an analysis of the systems' dynamic response to seismic disturbances has been made was not available. Apparently, all Class I systems were not analyzed.

B. Containment Spray: Time did not permit the discussion of this item.

## II. Reactor Building and Turbine Building Cooling Water Systems-

The applicant stated that the Reactor Building Water Systems is not required for the operation of any engineered safety features. DRL stated we would review this system in more detail at a later date.



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III. Reactor Coolant System (Primary and Emergency)

1. System response to total loss of AC power: although these items were included on the agenda transmitted to the applicant, the applicant has not evaluated:
  - a. the system response for 0 or 10 gpm leakage.
  - b. the time the isolation condenser is effective if a malfunction of the emergency condenser level control valve permits the maximum uncontrolled flow through the emergency condenser.
  - c. the amount of leakage because of a failure in the return line from the emergency condenser that is permitted before the leak is detected by area temperature monitors.
  - d. the time for automatic blowdown as a function of leakage.
2. Leak Detection Methods
  - a. The normal leakage from the total system is thought to be less than 6000 lbs/hr. The fraction of this that is within the isolatable portion of the primary system is unknown and has not been estimated.
  - b. The sensitivity of any of the leak detection systems was not known.
  - c. The operator's response to leakage detection is unknown. At present it is left to the operator's discretion.
3. Quality Control -
  - a. Prime responsibility for quality control is with the component manufacturer. GE's resident inspector for the reactor vessel performed a 100% review of QC test data e.g. mill reports, radiographs, Ultrasonic tests, etc.
  - b. Applicant could not explain the meaning of some of the "extra-code" quality control items mentioned in the FSAR.
4. Pressure Vessel Surveillance Program

At present, plans are not firm for the analysis of samples received from this program. Applicant has not considered spreading out the sample points as the dose level approaches  $10^{17}$  NVt.



#### 5. Preoperational Test Program

Control Rod Stub Tube Welds (similar to those that cracked in OC vessel) have been examined by dye penetrant and Ultrasonics. No cracks were detected. The flush and cleaning procedure, the chemicals to be used and the sequence (before or after the hydro test) are presently being reviewed.

#### 6. Periodic Inspections

Program is not fully developed - the locations which will be available for periodic inspections, the methods used to perform periodic inspection and the method used to determine the significance of the results has not been established.

#### 7. Pipe Whip and Missile Criteria

Applicant believes redundancy of systems will protect the plant from a cascading failure due to pipe whip and missiles. Maximum possible physical separation has been used between redundant systems in the drywell. Systems have not been analyzed for possible damage to systems or instrumentation as a result of pipe whip.

### IV. Conduct of Operations

The Staff outlined what is expected as a description of emergency plans, operating procedures, review and audit groups and preoperational test procedures.

### V. Instrumentation

This area was not covered in detail as the applicant did not bring persons who were familiar with the details of these systems although specific areas of concern were noted on the agenda. Several schematic drawings (requested prior to meeting) were left with Mr. Parr.

The discussions on the Core Spray and Primary System did indicate several potential problem areas that should be investigated. These are:

1. Sequencer for Core Spray System.
2. Load sequencer for Core Spray System.
3. Interlocks in Core Spray System for test operations.
4. Prevention of Auto Blowdown if AC power is not available.



The following information was obtained from the records of the  
 Department of the Interior, Bureau of Land Management, on  
 the subject of the above-captioned land.

The land is situated in the County of \_\_\_\_\_, State of \_\_\_\_\_, and is  
 more particularly described as follows:

Section \_\_\_\_\_, Township \_\_\_\_\_, Range \_\_\_\_\_,

5. Sensitivity of Leak Detection Systems

6. Compliance with applicant's criterion that all safety system transducers have an analog readout. Preoperational testing of the safety system, engineered safety feature system and the protective relaying will include complete system operation from sensor to final actuator.

IV. Electrical Power

The system was described as being the same as in the FSAR. Schematics were left with Mr. Parr.

Distribution:

Suppl.

DRL Reading

RFB-2 Reading

Orig: V. Stello

R. L. Tedesco

Branch Chiefs, DRL

R. L. Ferguson

O. D. Parr

D. C. Fischer





ATTENDEES - DECEMBER 19, 1967

R. L. Tedesco	DRL
R. L. Ferguson	DRL
V. Stello	DRL
O. D. Parr	DRL
D. C. Fischer	DRL
R. T. Carlson	Compliance
J. N. Ewart	Niagara Mohawk Power Corp.
P. A. Burt	Niagara Mohawk Power Corp.
H. D. Philipp	Niagara Mohawk Power Corp.
G. K. Rhode	Niagara Mohawk Power Corp.
Z. E. Chilaze	Niagara Mohawk Power Corp.
M. H. Pratt	Niagara Mohawk Power Corp.
F. J. Schneider	Niagara Mohawk Power Corp.
H. R. Nimms	Millstone Pt. Co.
P. W. Ianni	General Electric
R. B. Lemmon	General Electric
A. E. Upton	LeBoeuf, Lamb & Leiby
J. A. Lodge	LeBoeuf, Lamb & Leiby



UNITED STATES GOVERNMENT

Memorandum

→ 7. L Kelly  
~~R. J. Campbell~~  
Fac. File

TO : Harold L. Price  
Director of Regulation

DATE: March 30, 1967

FROM : Peter A. Morris, Director *P.A. Morris*  
Division of Reactor Licensing

SUBJECT: TELEPHONE CALL FROM ED LEGAN, IBEW

I had a telephone call today from Ed Legan who is affiliated with the International Brotherhood of Electrical Workers. He told me that he had attended a meeting with representatives of Niagara Mohawk in Syracuse this week concerning the Nine Mile Point Nuclear Power Plant. He told me that Niagara Mohawk was planning to operate the plant with three men per shift with only one licensed operator in the control room. He expressed great concern about this proposal and was interested in knowing if there had been a change in Commission policy. He stated that the Union would request a public hearing at the time of licensing should this proposal still be in effect at that time. He plans to request that the AEC's Advisory Committee on Labor Management discuss this matter.

cc: H. Shapar, OGC  
C. K. Beck, REG  
M. M. Mann, REG  
H. T. Herrick, LABR  
R. S. Boyd, RL  
D. J. Skovholt, RL *OP*

