



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

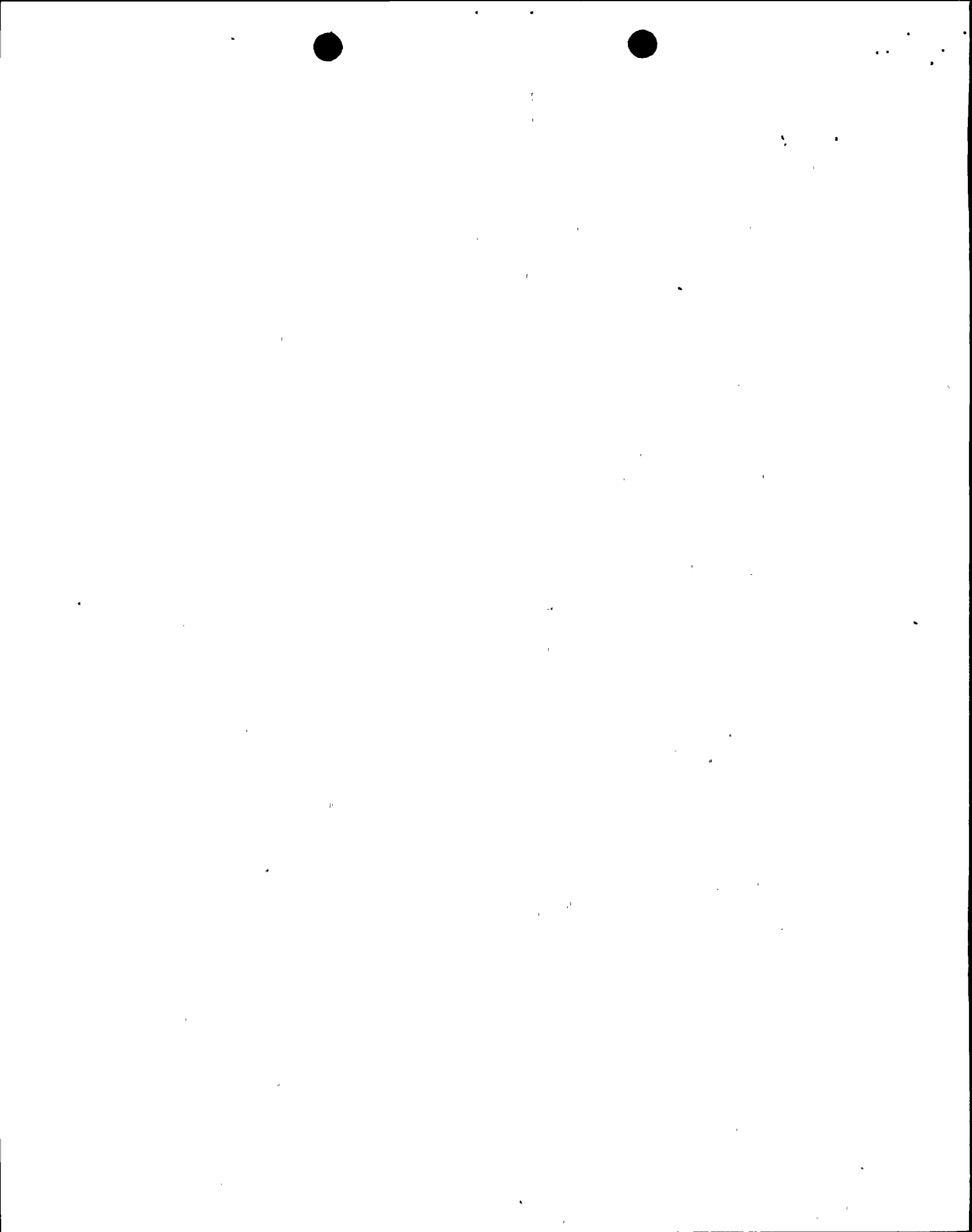
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 154  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated June 30, 1994, as supplemented March 7, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 154, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

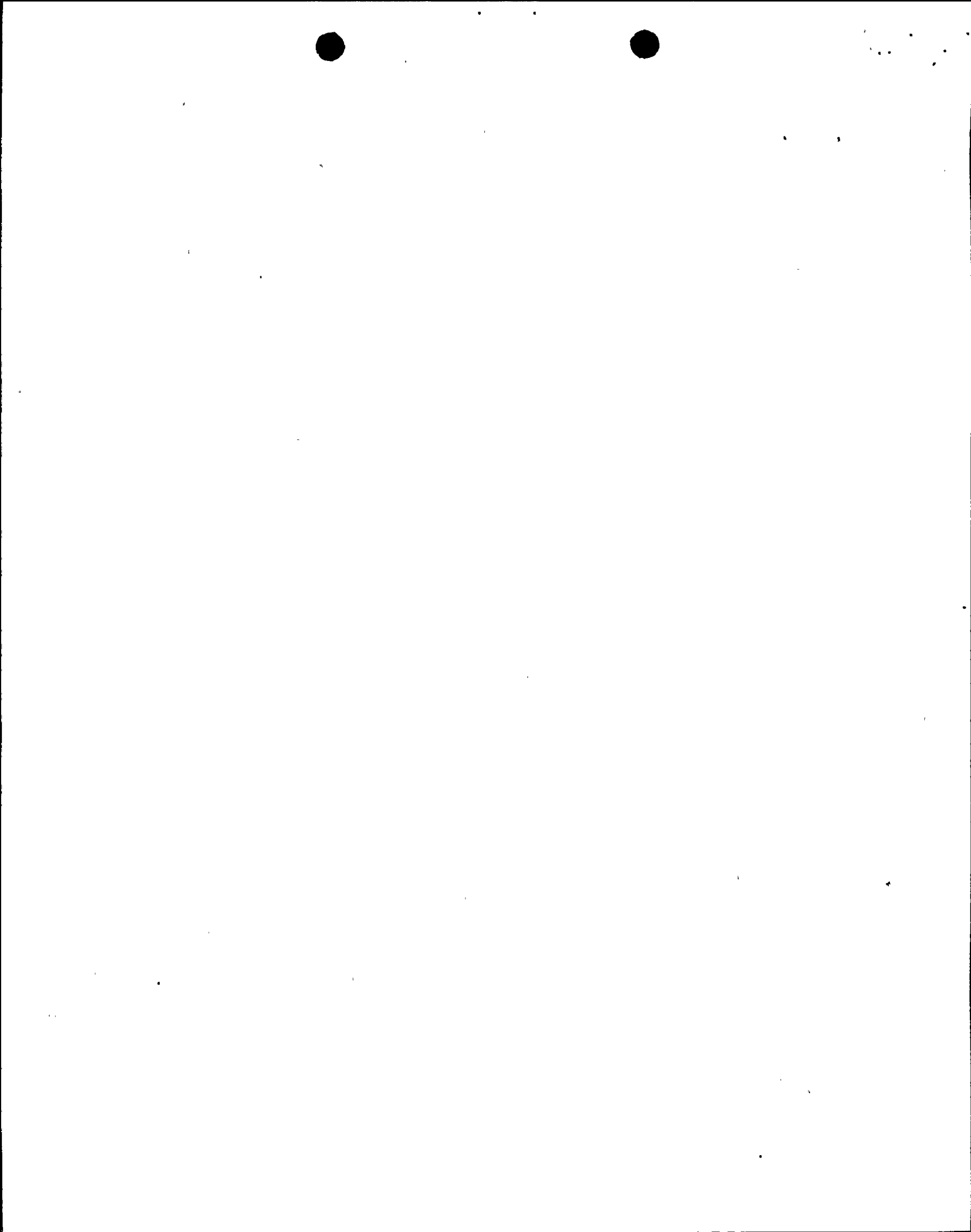
FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 20, 1995



ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 154 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

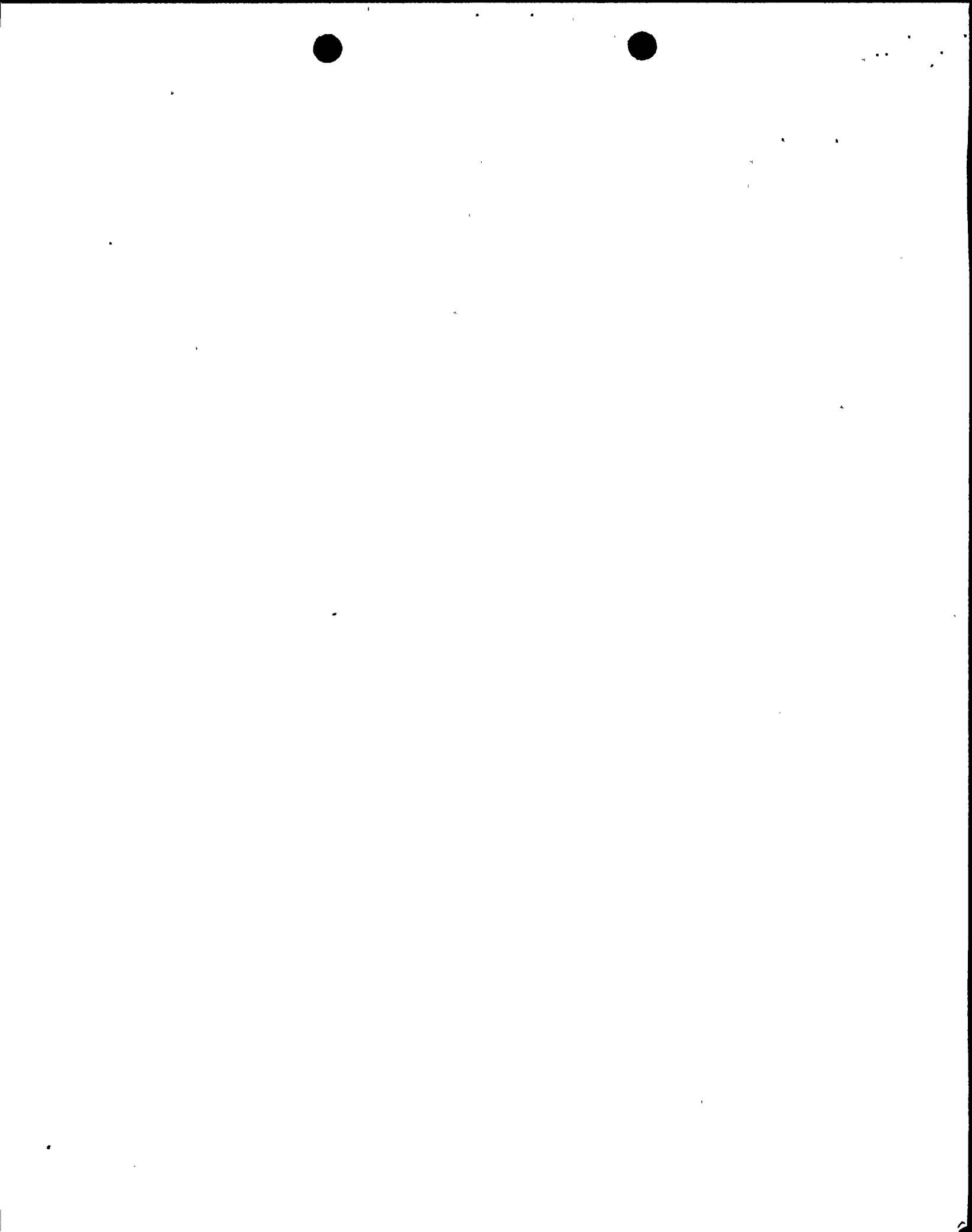
Revise Appendix A as follows:

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**TABLE 3.2.7.1**

**PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES**

	<b><u>System</u></b>	<b><u>Valve No.</u></b>	<b><u>Maximum<sup>(a)</sup> Allowable Leakage</u></b>
1.	Core Spray System	40-03	≤ 5.0 gpm
		40-13	≤ 5.0 gpm
2.	Condensate Supply to Core Spray (Keep Fill System)	40-20	≤ 5.0 gpm
		40-21	≤ 5.0 gpm
		40-22	≤ 5.0 gpm
		40-23	≤ 5.0 gpm
3.	Core Spray Supply to Shutdown Cooling (Waterseal)	38-165	≤ 0.375 gpm
		38-166	≤ 0.375 gpm
		38-167	≤ 0.375 gpm
		38-168	≤ 0.375 gpm
		38-169	≤ 0.375 gpm
		38-170	≤ 0.375 gpm
		38-171	≤ 0.375 gpm
		38-172	≤ 0.375 gpm

**Footnote:**

- (a) 1. Leakage rates shall be limited to 0.5 gpm per nominal inch of valve diameter up to a maximum of 5 gpm.
2. Test differential pressure shall not be less than 150 psid.
3. The observed leakage at test differential pressure shall be adjusted to the functional maximum pressure differential.

