



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 152
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated July 21, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 152, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 25, 1995



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ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 152 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Pages

23
25
118
119

Insert Pages

23
25
118
119



SAFETY LIMIT

2.2.1 REACTOR COOLANT SYSTEM

Applicability:

Applies to the limit on reactor coolant system pressure.

Objective:

To define those values of process variables which shall assure the integrity of the reactor coolant system to prevent an uncontrolled release of radioactivity.

Specification:

The reactor vessel or reactor coolant system pressure shall not exceed 1375 psig at any time with fuel in the vessel.

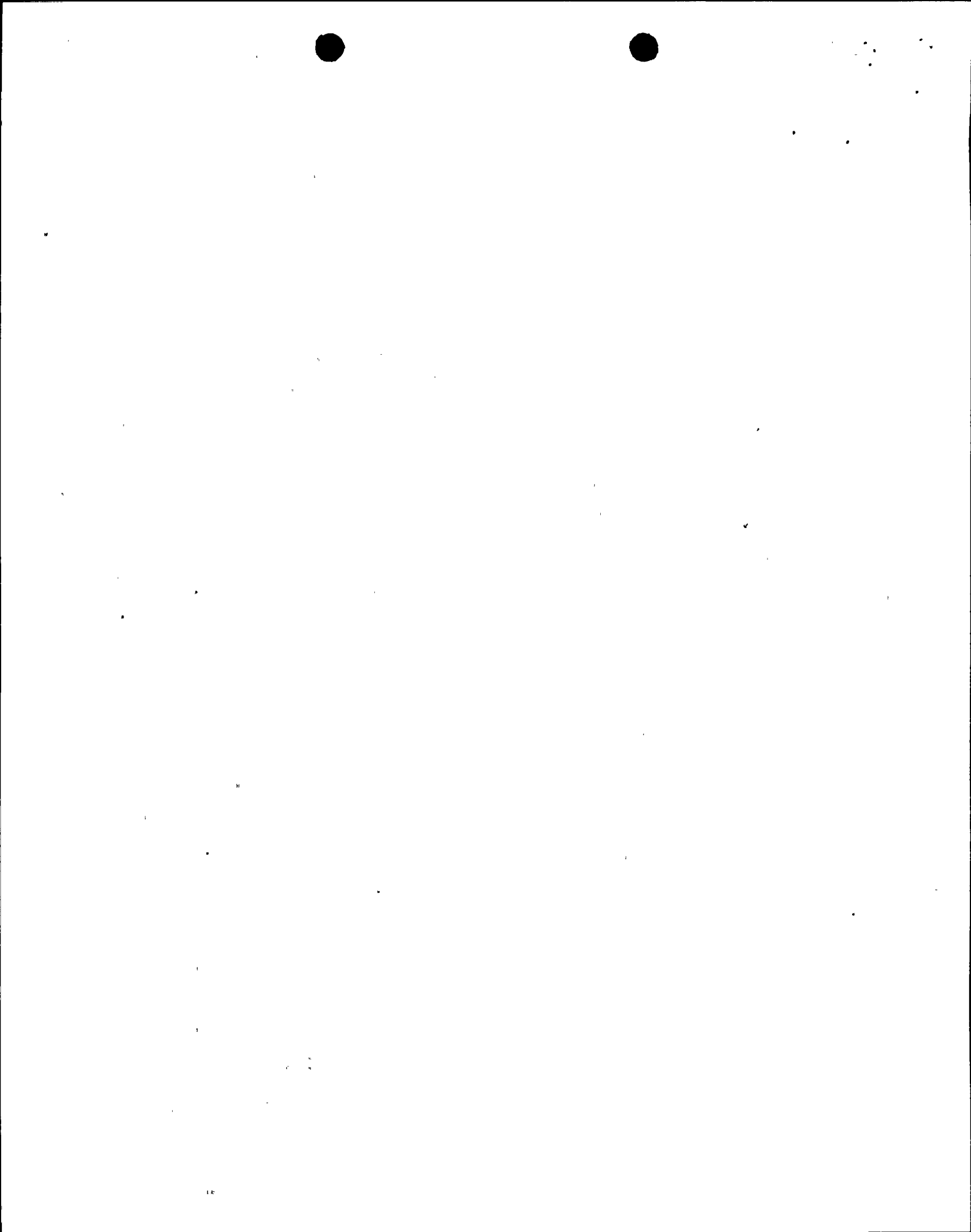
LIMITING SAFETY SYSTEM SETTING

2.2.2 REACTOR COOLANT SYSTEM

- a. The settings on the safety valves of the pressure vessel shall be as shown below. The allowable initial set point error on each setting will be ± 1 percent.

<u>Set Point (Psig)</u>	<u>Number of Safety Valves</u>
1218	3
1227	2
1236	2
1245	1
1254	<u>1</u>
	9

- b. The reactor high-pressure scram trip setting shall be ≤ 1080 psig.
- c. The flow biased APRM scram trip settings shall be in accordance with Specification 2.1.2a.



BASES FOR 2.2.2 REACTOR COOLANT SYSTEM - LIMITING SAFETY SYSTEM SETTING

- a. The range of set points for a safety valve actuation is selected in accordance with code requirements. The safety valves are sized according to the code for a condition of main steam isolation valve closure while operating at 1850 Mwt, followed by a reactor scram on high neutron flux. Under these conditions, a total of nine (9) safety valves are required to limit reactor pressure below the safety limit of 1375 psig.

In addition to the safety valves, the solenoid-actuated relief valves are used to prevent safety valve lift during rapid reactor isolation at power coupled with failure of the bypass system. Any five of these valves opening at 1090 psig to 1100 psig will keep the maximum vessel pressure below the lowest safety valve setting, as demonstrated in Appendix E-1.3.11 (p. E-35)*. (The Technical Supplement to Petition to Increase Power Level, and letter from T. J. Brosnan, Niagara Mohawk Power Corporation, to Peter A. Morris, Division of Reactor Licensing, USAEC, dated February 28, 1972). Subsequently, six valves were provided due to the blowdown requirements, following a small line break.

- b. The reactor high pressure scram setting is relied upon to terminate rapid pressure transients if other scrams, which would normally occur first, fail to function. As demonstrated in Appendix E-1 of the FSAR and the Technical Supplement to Petition to Increase Power Level, Page II-12, the reactor high pressure scram is a backup to the neutron flux scram, generator load rejection scram, and main steam isolation-valve closure scram for various reactor isolation incidents. However, rapid isolation at lower power levels generally results in high pressure scram preceding other scrams because the transients are slower and those trips associated with the turbine-generator are bypassed.

The operator will set the trip setting at 1080 psig or lower. However, the actual set point can be as much as 15.8 psi above the 1080 psig indicated set point due to the deviations discussed above.

*FSAR



LIMITING CONDITION FOR OPERATION

3.2.8 PRESSURE RELIEF SYSTEMS-SAFETY VALVES

Applicability:

Applies to the operational status of the safety valves.

Objective:

To assure the capability of the safety valves to limit reactor overpressure below the safety limit in the event of rapid reactor isolation and failure of all pressure relieving devices.

Specification:

- a. During power operating conditions and whenever the reactor coolant pressure is greater than 110 psig and temperature greater than saturation temperature all nine of the safety valves shall be operable.
- b. If specification 3.2.8a is not met, the reactor coolant pressure and temperature shall be reduced to 110 psig or less and saturation temperature or less, respectively, within ten hours.

SURVEILLANCE REQUIREMENT

4.2.8 PRESSURE RELIEF SYSTEMS-SAFETY VALVES

Applicability:

Applies to the periodic testing requirements for the safety valves.

Objective:

To assure the capability of the safety valves to limit reactor overpressure to below the safety limit.

Specification:

At least once during each operating cycle, the number of safety valves as determined by the IST Program Plan shall be removed, tested for set point and partial lift, and then returned to operation or replaced.



BASES FOR 3.2.8 AND 4.2.8 PRESSURE RELIEF SYSTEM-SAFETY VALVES

The required number of operable safety valves is based on a condition of main steam isolation valve closure while operating at 1850 Mwt, followed by a reactor scram on high neutron flux. Operation of all 9 safety valves will limit reactor pressure below the safety limit of 1375 psig.

The safety valve testing and intervals between tests are based on manufacturer's recommendations and past experience with spring actuated safety valves.

