



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

Docket
File
50-220/410

January 13, 1995

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
P.O. Box 63
Lycoming, NY 13093

SUBJECT: NINE MILE POINT NUCLEAR STATION, UNIT NOS. 1 AND 2 (NMP-1 AND NMP-2), APPROVAL OF CODE CASE N-498-1 AS AN ALTERNATIVE TO THE REQUIRED HYDROSTATIC PRESSURE TEST (TAC NOS. M90232 AND M90233)

Dear Mr. Sylvia:

By letter dated August 18, 1994, Niagara Mohawk Power Corporation (NMPC) requested approval to use American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Case N-416-1 in lieu of performing the Code-required hydrostatic pressure test for welded repairs or installation of replacement items by welding and ASME Code Case N-498-1 in lieu of performing the required 10-year pressure test at NMP-1 and NMP-2. NRC approval for NMPC to use ASME Code Case N-416-1, with certain provisions, was issued by our letter and attached safety evaluation dated October 18, 1994. However, as stated in our October 18, 1994, letter, the NRC staff's review of ASME Code Case N-498-1 was incomplete at that time and, therefore, use of ASME Code Case N-498-1 was not then approved.

The staff has now completed its review of the proposed use of ASME Code Case N-498-1 at NMP-1 and NMP-2. Based on the information submitted, NMPC's alternative (ASME Code Case N-498-1) is authorized pursuant to 10 CFR 50.55a(a)(3)(ii) since compliance with the specified requirements of Section XI of the ASME Code would result in hardship or unusual difficulty without a compensating increase in the level of safety and quality when applied to pressure testing at both NMP-1 and NMP-2.

NMPC's alternative, use of ASME Code Case N-498-1, is authorized until such time as this code case is published in a future revision of Regulatory Guide 1.147. At that time, if NMPC intends to continue to implement this Code case, NMPC is required to follow all provisions in ASME Code Case N-498-1 with limitations issued in Regulatory Guide 1.147, if any.

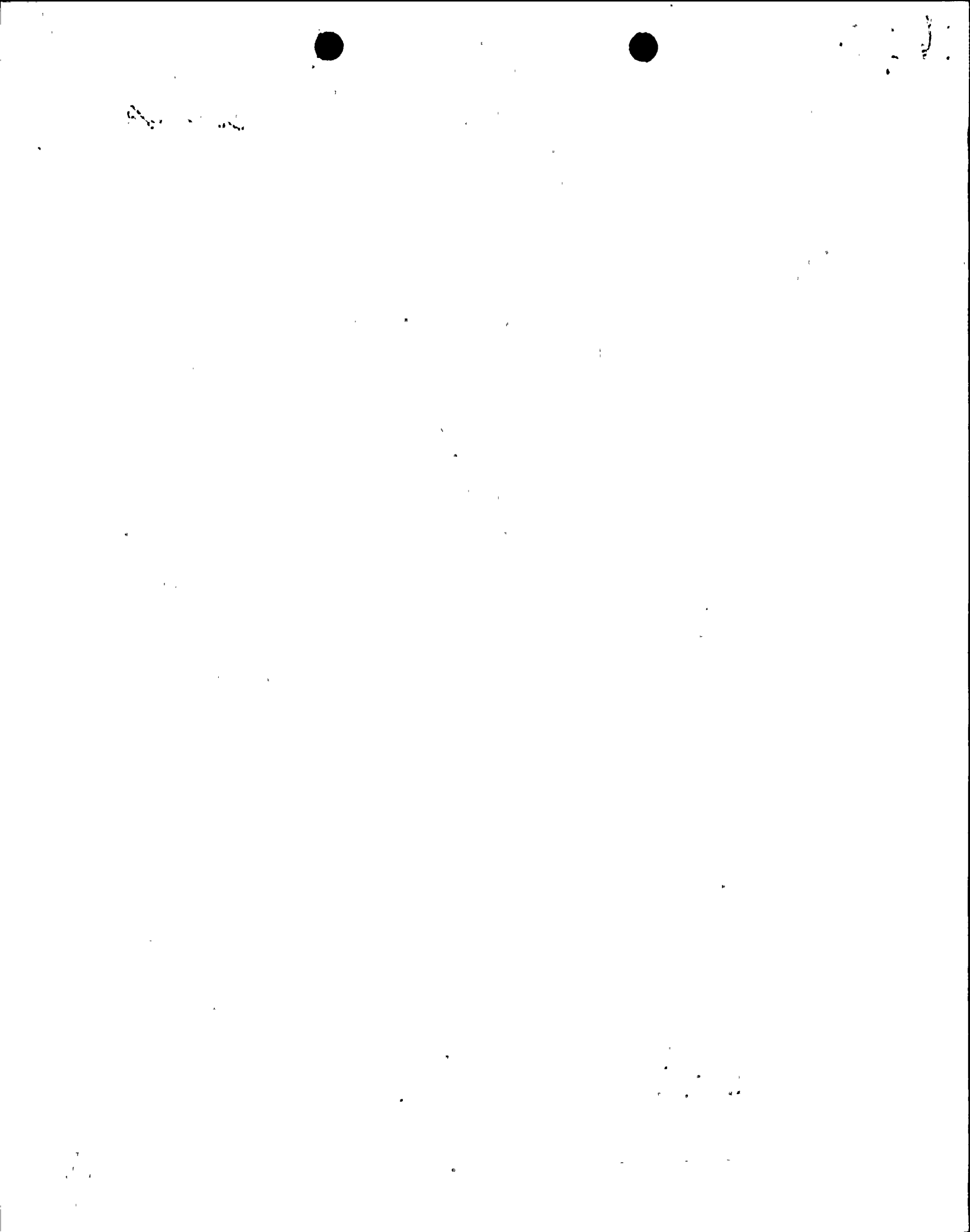
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The staff's evaluation and conclusions regarding ASME Code Case N-498-1 are contained in the attached safety evaluation.

Sincerely,

Michael J. Case, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-220
and 50-410

Enclosure: Safety Evaluation

cc w.encl: See next page

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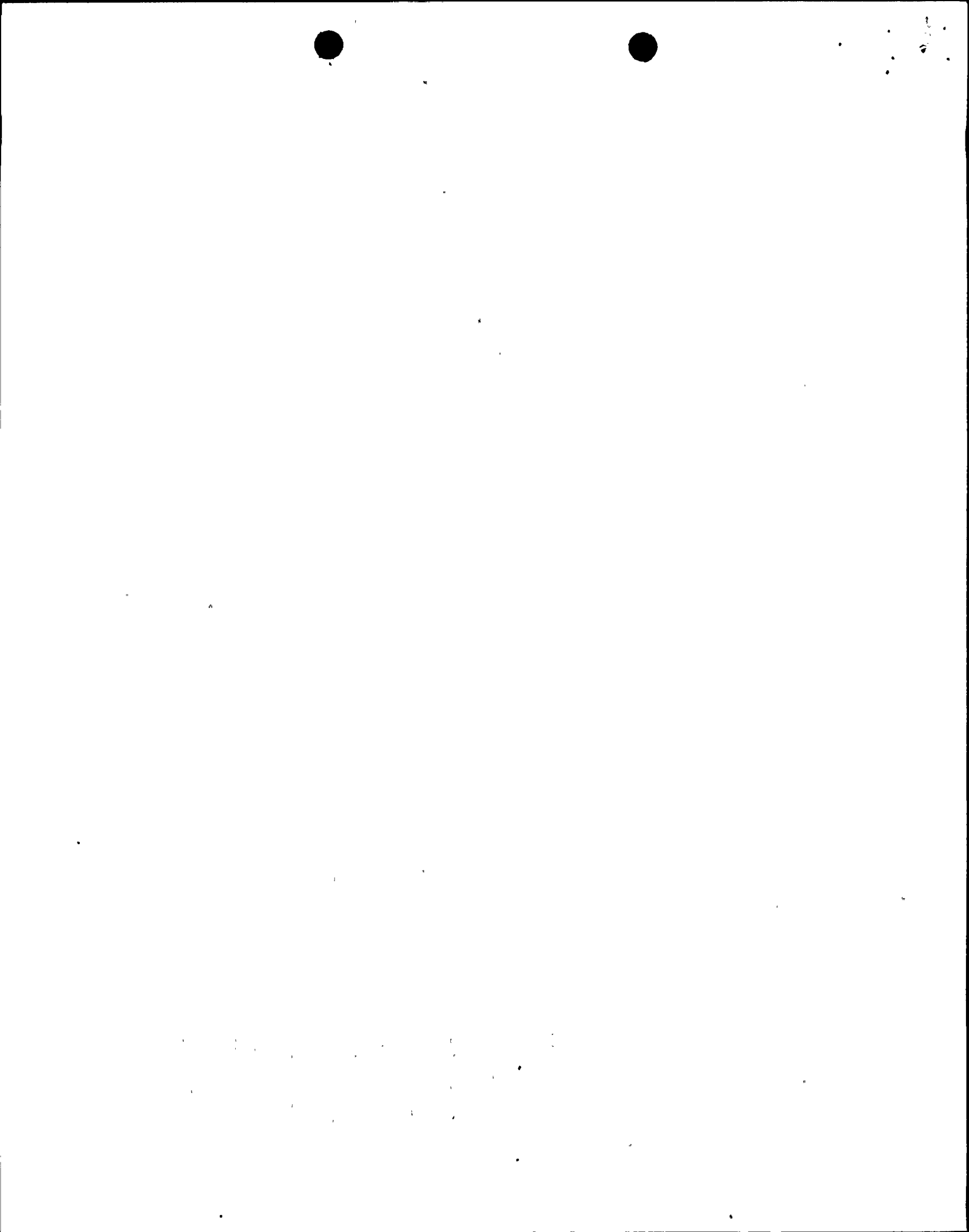
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B. Sylvia

-2-

January 13, 1995

The staff's evaluation and conclusions regarding ASME Code Case N-498-1 are contained in the attached safety evaluation.

Sincerely,



Michael J. Case, Acting Director
Project Directorate I-1
Division of Reactor Projects - I/II
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Enclosure: Safety Evaluation

cc w/encl: See next page



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