



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 16, 1994

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
P.O. Box 63
Lycoming, New York. 13093

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING RESPONSE TO
GENERIC LETTER (GL) 94-03 (TAC NO. M90102)

Dear Mr. Sylvia:

By letter dated August 23, 1994, Niagara Mohawk Power Corporation (NMPC) responded to GL 94-03, "Integranular Stress Corrosion Cracking of Core Shrouds in Boiling Water Reactors," for Nine Mile Point Nuclear Station, Unit No. 1. The NRC staff has begun its review of NMPC's response. However, we have determined that additional information, as identified in the enclosure, is required to complete our review of the NMPC response. NMPC is requested to respond to this RAI by September 26, 1994, in order for us to complete our review in a timely manner.

If you have any questions regarding this RAI, please contact me at 301-504-1409.

This requirement affects one respondent and, therefore, is not subject to Office of Management and Budget review under P.L. 96-511.

Sincerely,

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-220

Enclosure: Request for Additional
Information

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Nine Mile Point Nuclear Station
Unit No. 1

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REQUEST FOR ADDITIONAL INFORMATION

REGARDING RESPONSE TO GL 94-03

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

Generic Letter 94-03 required that a plant-specific assessment of the shroud response to the structural loadings resulting from design basis events assuming a postulated 360° through-wall crack be performed. The NRC staff has reviewed the licensee's submittal and has determined that the licensee has not met the requirements of the generic letter. Therefore, the following additional information is required.

1. Specifically, the licensee used Oyster Creek's RELAP 5 model to assess the consequence of a main steamline break (MSLB). The licensee did not specify why Oyster Creek's model was used and not a plant-specific model. The staff did not agree with Oyster Creek's conclusion that 0.025 inches provided sufficient margin of safety to consider the postulated MSLB with a 360° through-wall crack as not significant. Niagara Mohawk Power Corporation (NMPC) should perform a plant-specific consequence assessment to ensure that the top guide will not clear the top of the active fuel during an MSLB.
2. The licensee did not perform a plant-specific assessment of any structural loadings due to a seismic event, a seismic event plus a recirculation line break (RLB), and a seismic event plus an MSLB. NMPC should perform a consequence assessment of seismic events stated above to ensure core spray operability and control rod insertion assuming a 360° through-wall crack.
3. Furthermore, the licensee did not assess a postulated 360° through-wall crack at the H8 weld during an RLB at Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The licensee did not believe that this was a credible failure. The licensee stated that cracking in H8 would have to be about 90 percent through-wall for 360° in order for H8 to fail during an RLB. The staff questions why continued operation of NMP-1 is justified if a 360° crack that is 90 percent or greater through-wall is present at the H8 weld. The licensee did not evaluate the acoustic and blowdown loads associated with the RLB. NMPC should justify continued operation assuming a 360° through-wall crack at the H8 weld.



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