



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

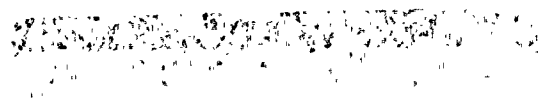
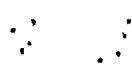
NINE MILE POINT NUCLEAR STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 146
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated December 22, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 146, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 8, 1994



ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Page
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LIMITING CONDITION FOR OPERATION

- c. The results of laboratory carbon sample analysis shall show $\geq 90\%$ radioactive methyl iodide removal when tested in accordance with ANSI N.510-1980 at 80°C and 95% R.H.
- d. Fans shall be shown to operate within $\pm 10\%$ design flow.
- e. During reactor operation, from and after the date that one circuit of the emergency ventilation system is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such circuit is sooner made operable, provided that during such seven days all active components of the other emergency ventilation circuit shall be operable.

During refueling, from and after the date that one circuit of the emergency ventilation system is made or found to be inoperable for any reason, fuel handling is permissible during the succeeding seven days unless such circuit is sooner made operable, provided that during such seven days all active components of the other emergency ventilation circuit shall be operable. Fuel handling may continue beyond seven days provided the operable emergency ventilation circuit is in operation.

- f. If these conditions cannot be met, within 36 hours, the reactor shall be placed in a condition for which the emergency ventilation system is not required.

SURVEILLANCE REQUIREMENT

- b. The tests and sample analysis of Specification 3.4.4b, c and d shall be performed at least once per operating cycle or once every 24 months, or after 720 hours of system operation, whichever occurs first or following significant painting, fire or chemical release in any ventilation zone communicating with the system.
- c. Cold DOP testing shall be performed after each complete or partial replacement of the HEPA filter bank or after any structural maintenance on the system housing.
- d. Halogenated hydrocarbon testing shall be performed after each complete or partial replacement of the charcoal adsorber bank or after any structural maintenance on the system housing.
- e. Each circuit shall be operated with the inlet heater on at least 10 hours every month.
- f. Test sealing of gaskets for housing doors downstream of the HEPA filters and charcoal adsorbers shall be performed at and in conformance with each test performed for compliance with Specification 4.4.4b and Specification 3.4.4b.





UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 146 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated December 22, 1993, Niagara Mohawk Power Corporation (the licensee or NMPC) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1, Technical Specifications (TSs). The requested changes would revise TS 3.4.4.e (Emergency Ventilation System). TS 3.4.4.e currently permits fuel handling operations to continue during refueling for up to 7 days with one circuit of the emergency ventilation system inoperable, provided all active components of the other emergency ventilation system circuit are operable. The proposed revision would permit fuel handling operations to continue during refueling beyond 7 days with one circuit of the emergency ventilation system inoperable, provided the remaining emergency ventilation system circuit is operable and in operation. The licensee stated that the proposed revision is consistent with recently issued Amendment No. 47 to the Nine Mile Point Unit 2 TSs and with the NRC's Improved Standard Technical Specifications, NUREG-1433.

2.0 EVALUATION

TS 3.4.4.e currently permits reactor operation and fuel handling to continue for up to 7 days when one emergency ventilation system circuit is inoperable, provided that during such 7 days all active components of the other emergency ventilation system circuit are operable. The proposed change would remove the 7-day limit on fuel handling operations during refueling, provided the remaining emergency ventilation system circuit is operable and in operation.

The emergency ventilation system is provided to filter particulates and iodines from the reactor building atmosphere prior to exhausting to the stack and release to the environment during secondary containment isolation conditions. The emergency ventilation system is composed of two 100 percent capacity circuits which are normally maintained in a standby status. Each circuit starts automatically upon detection of high radiation levels in the exhaust duct of the normal reactor building ventilation system, or from high radiation at the refueling platform during refueling operations. Each circuit contains a filter bank for removal of particulates and halogens and has a



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rated flow capacity of 1600 cfm with the reactor building at a negative pressure of 0.25-inch water gauge relative to the outside atmosphere. This negative pressure ensures that air discharged from the reactor building is filtered before its release to the environment so as to minimize the release of radioactivity to the environment.

During normal operations (both reactor operations and refueling), the reactor building is ventilated by its normal reactor building ventilation system. The normal ventilation system maintains the reactor building at a negative pressure of at least 0.25-inch water gauge with respect to the outside environment, but this system does not have the capability to filter radioactivity from the discharged air. If radioactive materials are released into the reactor building atmosphere and their concentrations exceed a predetermined limit at the radiation detectors in the exhaust duct, the normal ventilation system is automatically stopped and isolated. The operable emergency ventilation system circuits are then automatically started. Operation of a single circuit of the emergency ventilation system will reestablish the required negative pressure in the reactor building and provide a filtered release to the environment.

The proposed change would remove the 7-day limit on continuing fuel handling operations during refueling with one circuit of the emergency ventilation system inoperable, provided the remaining circuit is operable and in operation. The proposed change is acceptable since placing the operable circuit in operation ensures that its safety function (filtering of the reactor building atmosphere before release to the environment) is being accomplished. The proposed change is also consistent with the NRC staff's current position which permits fuel handling operations to continue during refueling with one operable emergency ventilation system circuit, provided the operable circuit is in operation. The NRC staff's current position is reflected in the guidance provided in the NRC's Improved Standard Technical Specifications for Boiling Water Reactors (NUREG-1433) and satisfies the Commission's Final Policy Statement on TS (58 FR 39132). The NRC staff is considering this change as a potential TS line-item improvement.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no



significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 4940). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
Donald S. Brinkman

Date: March 8, 1994

March 8, 1994

Mr. B. Ralph Sylvia
 Executive Vice President, Nuclear
 Niagara Mohawk Power Corporation
 301 Plainfield Road
 Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT NO. 1
 (TAC NO. M88497)

The Commission has issued the enclosed Amendment No. 146 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated December 22, 1993.

The amendment revises TS 3.4.4.e (Emergency Ventilation System) to permit fuel handling operations to continue during refueling beyond 7 days with one circuit of the emergency ventilation system inoperable, provided the remaining emergency ventilation system circuit is operable and in operation. The change to TS 3.4.4.e is consistent with the NRC's Improved Standard Technical Specifications, NUREG-1433 and is being considered by the NRC staff as a potential TS line-item improvement.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Donald S. Brinkman, Senior Project Manager
 Project Directorate I-1
 Division of Reactor Projects - I/II
 Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 146 to DPR-63
2. Safety Evaluation

cc w/enclosures:

See next page

Distribution:

See attached sheet

94-038

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LA:PDI-1	PM:PDI-1 <i>Asb</i>	BC:OTSB	BC:SPLB	OGC	D:PDI-1
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OFFICIAL RECORD COPY
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The following information was obtained from the records of the
 Department of the Interior, Bureau of Land Management, regarding
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 for your information and is not intended to constitute a warranty
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