



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 145 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION UNIT NO. 1

DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated December 27, 1993, Niagara Mohawk Power Corporation (the licensee or NMPC) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1), Technical Specifications (TSs). The requested changes would relocate TS Tables 3.2.7, "Reactor Coolant Isolation Valves," and 3.3.4, "Primary Containment Isolation Valves," from TS 3.2.7/4.2.7 and 3.3.4/4.3.4, respectively, to a plant procedure which governs lists removed from TSs per Generic Letter (GL) 91-08, "Removal of Component Lists from Technical Specifications." The plant procedure would be subject to the requirements specified in the Administrative Controls section of the NMP-1 TSs. The proposed amendment would also make conforming changes to the TS Bases. These lists of valves will continue to be included in the NMP-1 Updated Final Safety Analysis Report. NMPC stated that the proposed changes would be consistent with NRC staff guidance issued in GL 91-08.

2.0 EVALUATION

The current NMP-1 TSs include Tables 3.2.7, "Reactor Coolant Isolation Valves," and 3.3.4, "Primary Containment Isolation Valves." These tables are currently referenced in TSs 3.2.7/4.2.7 and 3.3.4/4.3.4, respectively. Generic Letter (GL) 91-08, "Removal of Component Lists from Technical Specifications," was issued on May 6, 1991. This GL provides guidance for the removal of tables from TS that list components.

The proposed amendment would: (1) relocate Tables 3.2.7 and 3.3.4 from the NMP-1 TSs to a plant procedure that would be subject to the change control requirements of the Administrative Controls section of the NMP-1 TSs; (2) delete reference to Table 3.2.7 in TS 4.2.7; (3) delete reference to Table 4.3.4 in TS 4.3.4; and (4) add the phrase "except as noted in Specification 3.1.1.e." to TS 3.2.7. The added phrase to TS 3.2.7 regards the operability of the scram discharge volume isolation valves that were previously the subject of a footnote to Table 3.2.7. The proposed TS changes and relocation of Tables 3.2.7 and 4.3.4 are consistent with the guidance provided in GL 91-08 and are, therefore, acceptable.

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The proposed amendment would also make conforming changes in the Bases for the affected TS to delete reference to the relocated tables, delete reference to the 60 second closure time associated with all primary containment isolation valves, and reference the issuance of previously issued License Amendment No. 140. Deletion of the 60 second closure time was proposed since previously approved license amendments have authorized closure times of greater than 60 seconds. The proposed changes to the Bases are consistent with the current amendment and previously issued license amendments. Therefore, the NRC staff offers no objection to the proposed Bases changes.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 4941). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors:
Donald S. Brinkman
Thomas G. Dunning

Date: March 7, 1994



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Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT NO. 1
(TAC NO. M88496)

The Commission has issued the enclosed Amendment No. 145 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated December 27, 1993.

The amendment relocates TS Tables 3.2.7, "Reactor Coolant Isolation Valves," and 3.3.4, "Primary Containment Isolation Valves," from TSs 3.2.7/4.2.7 and 3.3.4/4.3.4, respectively, to a plant procedure which governs lists removed from TSs per Generic Letter (GL) 91-08, "Removal of Component Lists from Technical Specifications." The plant procedure would be subject to the requirements specified in the Administrative Controls section of the NMP-1 TSs. The proposed amendment would also make conforming changes to the TS Bases. These lists of valves will continue to be included in the NMP-1 Updated Final Safety Analysis Report. Relocation of these valve lists from the NMP-1 TSs to the plant procedure is consistent with NRC staff guidance issued in GL 91-08.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,
Original signed by:
Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

- Enclosures:
1. Amendment No. 145 to DPR-63
2. Safety Evaluation

cc w/enclosures:
See next page

OFFICE	PDI-1:LA	PDI-1:PM <i>DB</i>	OGC	PDI-1:D	
NAME	CVogan <i>CV</i>	DBrinkman:av1	<i>C Marco</i>	RACapra <i>RC</i>	
DATE	2/15/94	2/16/94	2/17/94	3/17/94	1/1

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