



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

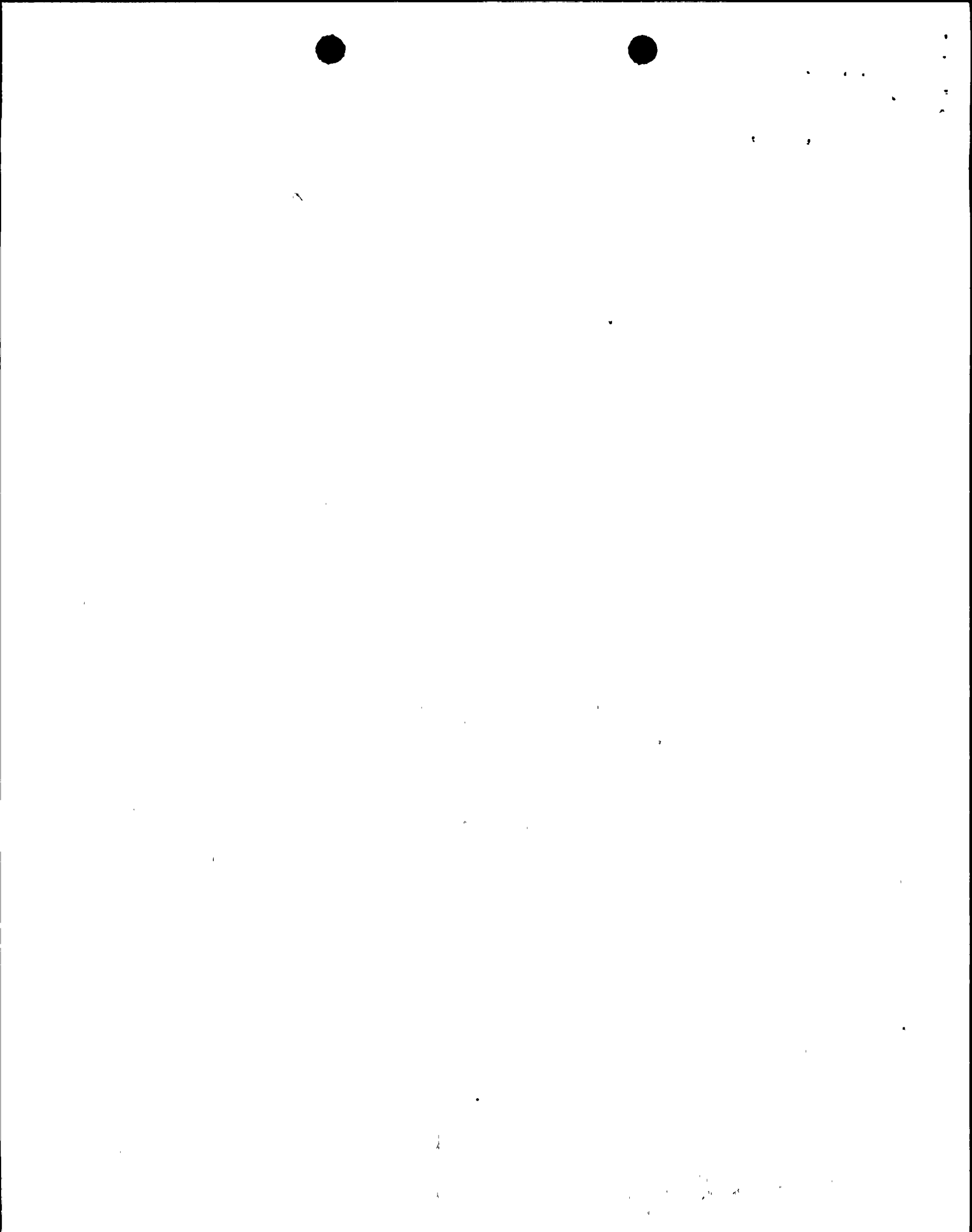
NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 49  
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated April 7, 1993, as superseded September 2, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 49 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

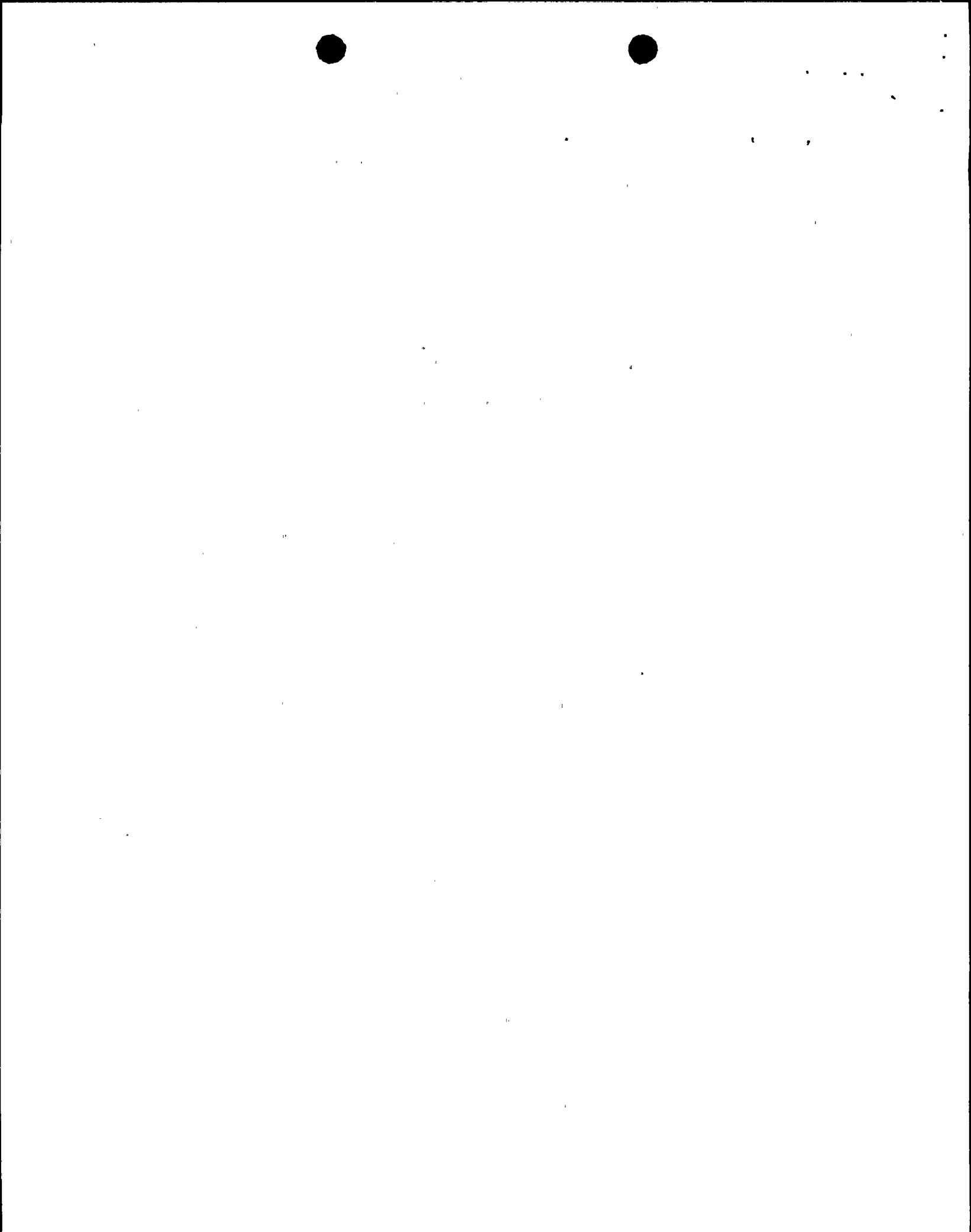
FOR THE NUCLEAR REGULATORY COMMISSION

*Robert A. Capra*

Robert A. Capra, Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 13, 1993



ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 49 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 1-9	3/4 1-9
3/4 1-10	3/4 1-10



# REACTIVITY CONTROL SYSTEMS

## CONTROL ROD SCRAM ACCUMULATORS

### LIMITING CONDITIONS FOR OPERATION

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3.1.3.5 All control rod scram accumulators shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 5\*.

ACTION:

- a. In OPERATIONAL CONDITIONS 1 or 2:
1. With one control rod scram accumulator inoperable:
    - a) If reactor steam dome pressure is greater than or equal to 900 psig, within 8 hours declare the control rod associated with the inoperable accumulator inoperable.  
  
Otherwise, place the reactor mode switch in the Shutdown position.
    - b) If reactor steam dome pressure is less than 900 psig, declare the associated control rod inoperable and if the associated control rod is withdrawn, immediately verify that at least one control rod drive pump is operating by inserting at least one withdrawn control rod at least one notch.  
  
Otherwise, place the reactor mode switch in the Shutdown position.
  2. With more than one control rod scram accumulator inoperable, declare the associated control rods inoperable and:
    - a) If the control rod associated with any inoperable scram accumulator is withdrawn, immediately verify that at least one control rod drive pump is operating by inserting at least one withdrawn control rod at least one notch. Otherwise:
      - 1) If reactor steam dome pressure is greater than or equal to 900 psig, restart at least one control rod drive pump within 20 minutes and then immediately insert at least one withdrawn control rod at least one notch or place the reactor mode switch in the Shutdown position, or
      - 2) If reactor steam dome pressure is less than 900 psig, immediately place the reactor mode switch in the Shutdown position.
    - b) Insert the inoperable control rods and disarm the associated control valves either:
      - 1) Electrically, or

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\* At least the accumulator associated with each withdrawn control rod. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.





## REACTIVITY CONTROL SYSTEMS

### CONTROL ROD SCRAM ACCUMULATORS

#### LIMITING CONDITIONS FOR OPERATION

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##### 3.1.3.5.a.2.b) (Continued)

##### ACTION:

2) Hydraulically by closing the drive water and exhaust water isolation valves.

Otherwise, be in at least HOT SHUTDOWN within 12 hours.

##### b. In OPERATIONAL CONDITION 5\*:

1. With one withdrawn control rod with its associated scram accumulator inoperable, insert the affected control rod and disarm the associated directional control valves within 1 hour, either:
  - a) Electrically, or
  - b) Hydraulically by closing the drive water and exhaust water isolation valves.
2. With more than one withdrawn control rod with the associated scram accumulator inoperable or no control rod drive pump operating, immediately place the reactor mode switch in the Shutdown position.

c. The provisions of Specification 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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##### 4.1.3.5 Each control rod scram accumulator shall be determined OPERABLE:

- a. At least once per 7 days by verifying that the indicated pressure is greater than or equal to 940 psig unless the control rod is inserted and disarmed or scrammed.
- b. At least once per 18 months by:
  1. Performance of a:
    - a) CHANNEL FUNCTIONAL TEST of the leak detectors, and
    - b) CHANNEL CALIBRATION of the pressure detectors, and verifying an alarm setpoint of greater than or equal to 940 psig on decreasing pressure.

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\* At least the accumulator associated with each withdrawn control rod. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2

