

# UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

#### NIAGARA MOHAWK POWER CORPORATION

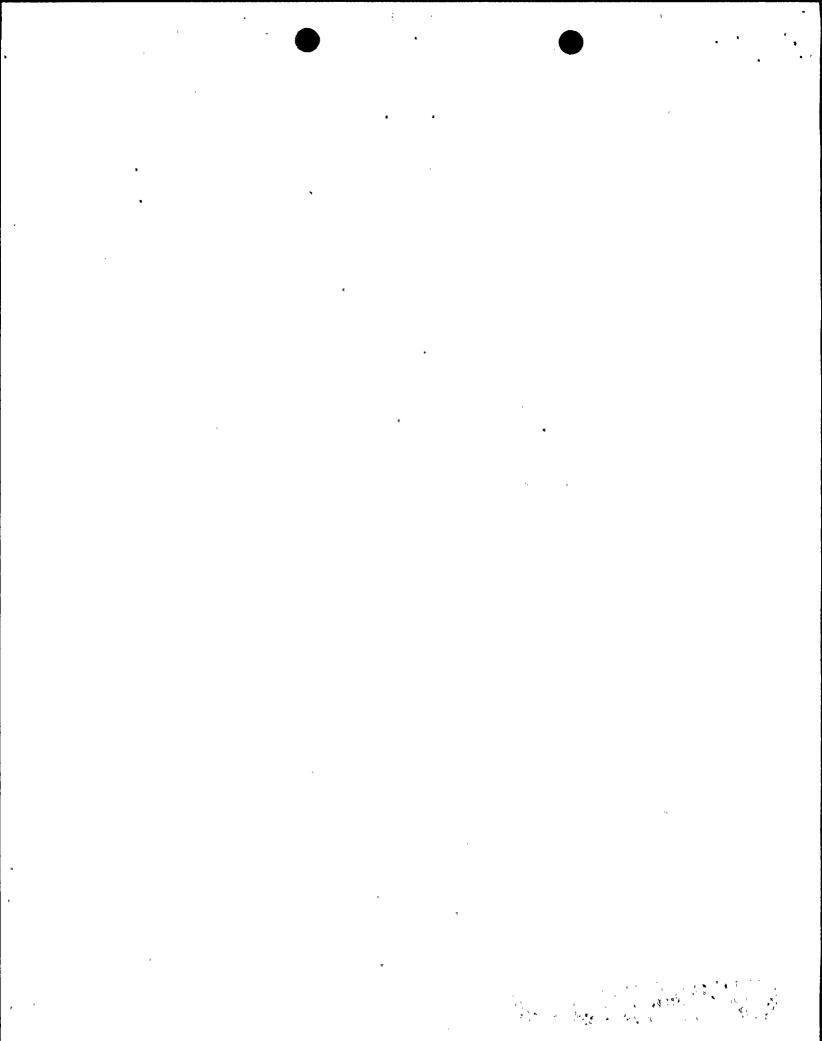
#### **DOCKET NO. 50-410**

### NINE MILE POINT NUCLEAR STATION, UNIT 2

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42 License No. NPF-69

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated November 24, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:



(2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 42 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

Robert a.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

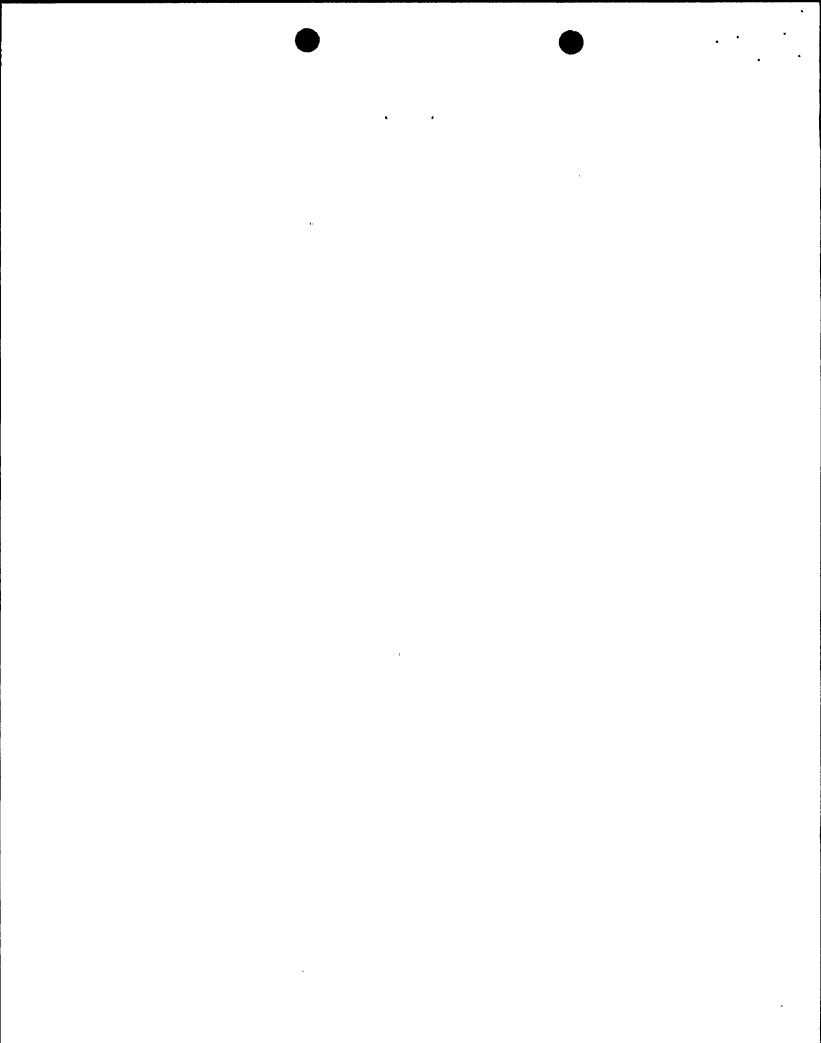
FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra, Director Project Directorate I-1

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: June 3, 1993



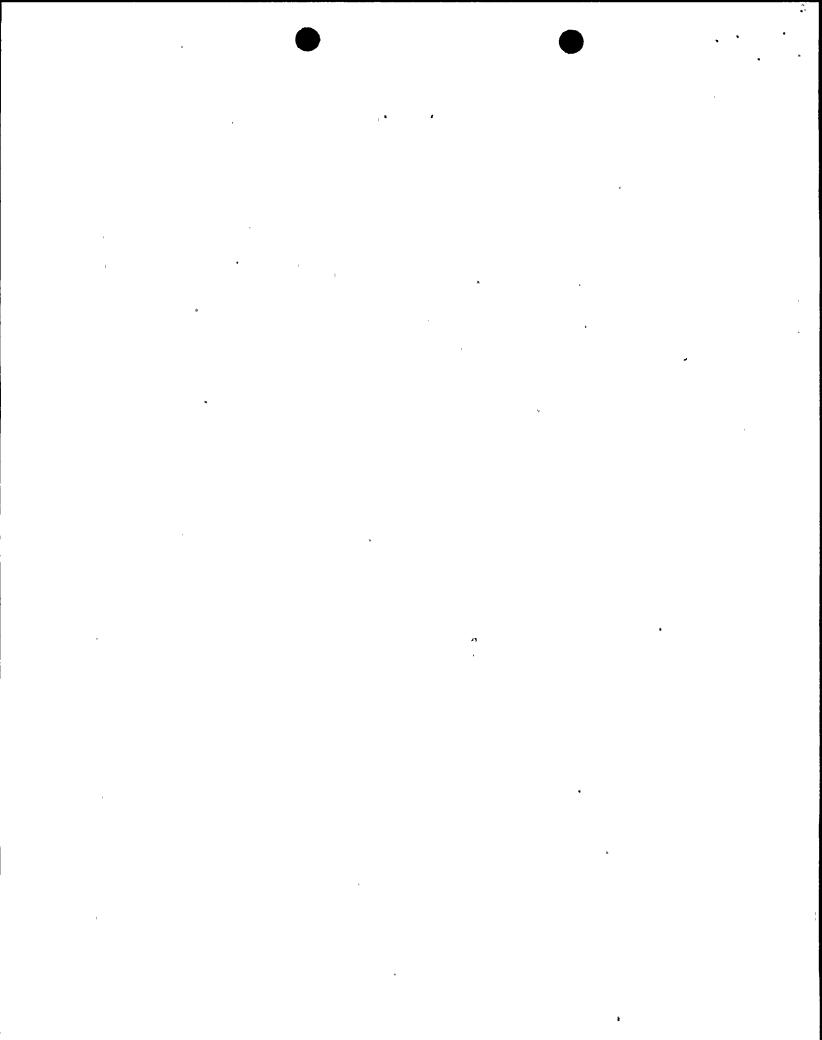
# ATTACHMENT TO LICENSE AMENDMENT

# AMENDMENT NO. 42 TO FACILITY OPERATING LICENSE NO. NPF-69

# **DOCKET NO. 50-410**

# Revise Appendix A as follows:

| <u>Remove Pages</u> ` | <u>Insert Pages</u> |
|-----------------------|---------------------|
| 6-7                   | 6-7                 |
| 6-8                   | 6–8                 |
| 6-12                  | 6-12                |



## ADMINISTRATIVE CONTROLS

#### **ORGANIZATION**

#### RECORDS

6.2.3.4 Records of activities performed by the ISEG shall be prepared, maintained, and forwarded each calendar month to the Vice President - Nuclear Engineering.

#### 6.2.4 SHIFT TECHNICAL ADVISOR

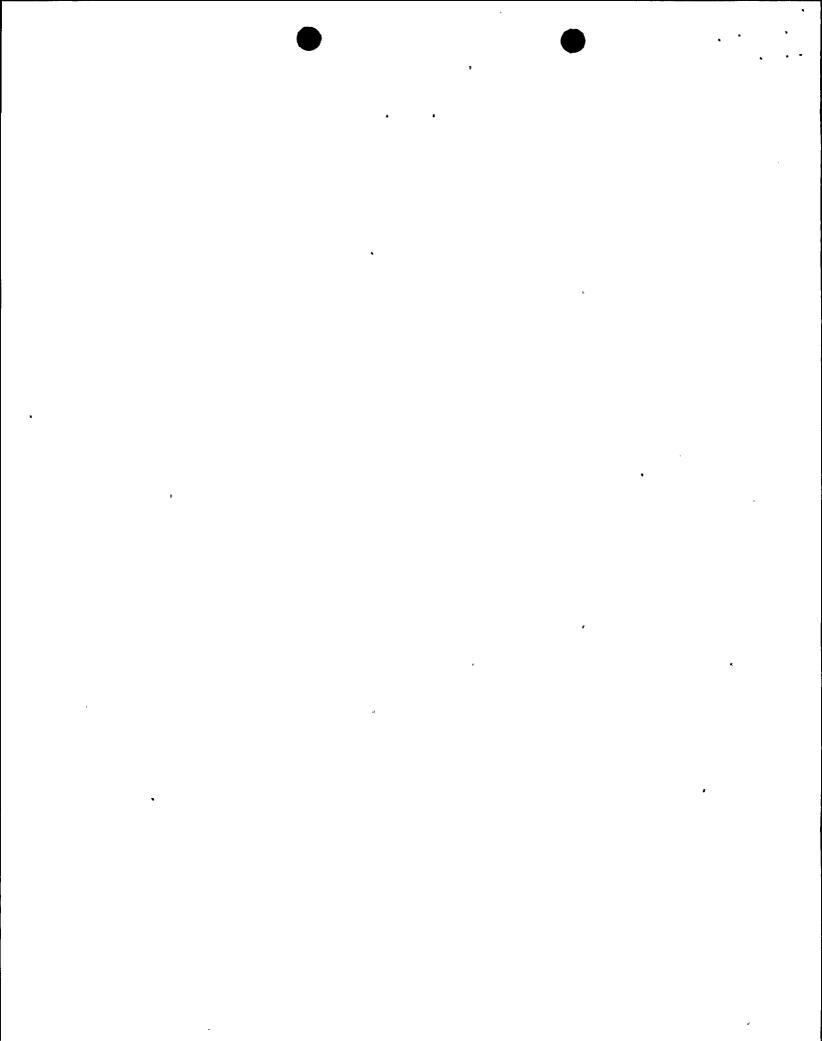
Normally, the Shift Technical Advisor (STA) shall be a dedicated position. If, however, a dedicated STA cannot be provided on a shift, then the Assistant Station Shift Supervisor (ASSS) shall function in a dual role (SRO/STA) and assume the duties of the Shift Technical Advisor (STA) when the Emergency Plan is activated in OPERATIONAL CONDITIONS 1, 2 or 3. The STA shall provide advisory technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to safe operation of the unit. The STA (and the ASSS, when fulfilling the role of the STA) shall have a bachelor's degree in a physical science, engineering, or a Professional Engineer's license issued by examination, and shall have received specific training in the response and analysis of the unit for transients and accidents, and in unit design and layout, including the capabilities of instrumentation and controls in the control room.

#### 6.3 FACILITY STAFF QUALIFICATIONS

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the Manager Radiation Protection who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

#### 6.4 TRAINING

A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager Training, shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI/ANS 3.1-1978 and 10 CFR 55, and shall include familiarization with relevant industry operational experience. A training program for the Fire Brigade shall be maintained under the direction of the Manager Training and the Supervisor - Fire Protection Nuclear and shall meet or exceed the requirements of Appendix R to 10 CFR 50.



#### 6.5 REVIEW AND AUDIT

#### 6.5.1 STATION OPERATIONS REVIEW COMMITTEE

#### **FUNCTION**

6.5.1.1 The Station Operations Review Committee (SORC) shall function to advise the Plant Manager on all matters related to nuclear safety.

#### COMPOSITION

6.5.1.2 The SORC shall be composed of the

Chairman: Plant Manager

Vice-Chairman/Member: Manager Operations

Vice-Chairman/Member: Manager Technical Support
Member: Manager QA Operations
Member: Manager Maintenance
Member: Manager Chemistry

Member: Manager Radiation Protection

#### **ALTERNATES**

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman or Vice-Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

#### MEETING FREQUENCY

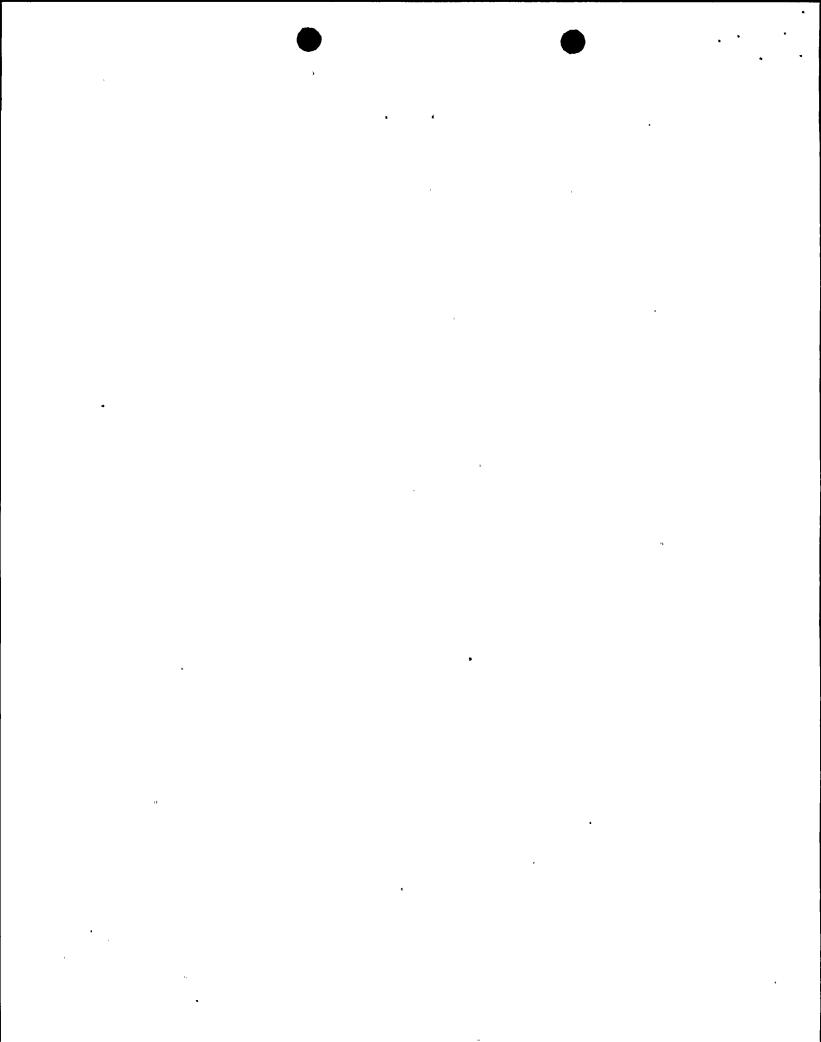
6.5.1.4 The SORC shall meet at least once every calendar month and as convened by the SORC Chairman, Vice-Chairman, or a designated alternate.

#### **OUORUM**

6.5.1.5 The quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman, or a Vice-Chairman and four members including alternates.

#### RESPONSIBILITIES

- 6.5.1.6 The SORC shall be responsible for:
- a. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Vice President Nuclear Generation and to the Safety Review and Audit Board;



## ADMINISTRATIVE CONTROLS

#### REVIEW AND AUDIT

### SAFETY REVIEW AND AUDIT BOARD

#### **ALTERNATES**

6.5.3.3 All alternate members shall be appointed in writing by the SRAB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRAB activities at any one time.

#### CONSULTANTS

6.5.3.4 Consultants shall be utilized as determined by the SRAB Chairman to provide expert advice to the SRAB.

#### MEETING FREQUENCY

6.5.3.5 The SRAB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

#### **QUORUM**

6.5.3.6 The quorum of the SRAB necessary for the performance of the SRAB review and audit functions of these Technical Specifications shall consist of not less than a majority of the members including alternates. The quorum requires the presence of the Chairman or the Chairman's designated alternate and no more than a minority of the quorum shall have line responsibility for operations of the facility.

#### REVIEW

- 6.5.3.7 The SRAB shall be responsible for the review of:
- a. The safety evaluations for (1) changes to procedures, equipment, or systems and (2) tests or experiments completed under the provision of 10 CFR 50.59 to verify that such actions did not constitute an unreviewed safety question;
- Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;

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