

07-187-91

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION

02-LOT-008-362-2-01 Revision 2

TITLE: OVERVIEW OF TECHNICAL SPECIFICATIONS

	<u>SIGNATURE</u>	<u>DATE</u>
PREPARER	<u>Wen Bridges</u>	<u>4/19/91</u>
TRAINING AREA SUPERVISOR	<u>M. White</u>	<u>4/23/91</u>
TRAINING SUPPORT SUPERVISOR	<u>F.A. Becking for J. Le Clair</u>	<u>4-24-91</u>
PLANT SUPERVISOR/ USER GROUP SUPERVISOR	<u>FOR D. Tolney</u>	<u>4/29/91</u>

Summary of Pages

(Effective Date: 4/29/91)

Number of Pages: 24

<u>Date</u>	<u>Pages</u>
April 1991	1 - 24

MASTER

CONTROLLED

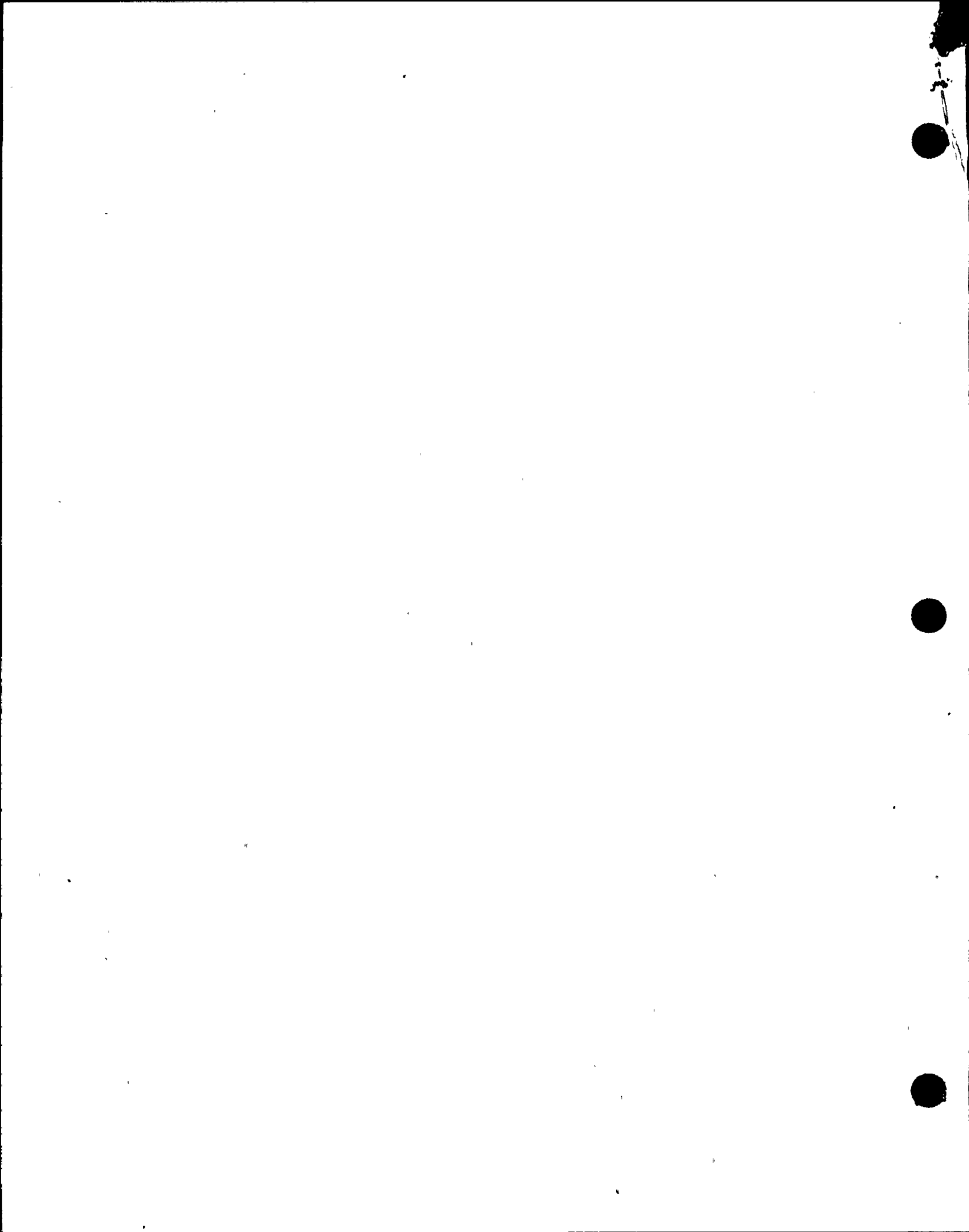
TRAINING DEPARTMENT RECORDS ADMINISTRATION ONLY

VERIFICATION: _____

DOCUMENT

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ATTACHMENT 6
LESSON PLAN TEMPORARY/PUBLICATION/ADDENDUM CHANGE FORM

The attached change was made to:

Lesson plan title: Overview of Technical Specifications

Lesson plan number: 02-LOT-008-362-2-01 Rev 2

Name of instructor initiating change: H.P. Strahler

Reason for the change: To include in the L.P. an overview of the USAR chapters. This overview is brief overview to ensure trainees have an understanding of what material can be found in the USAR

Type of change:

- 1. Temporary change
- 2. Publication change
- 3. Addendum change

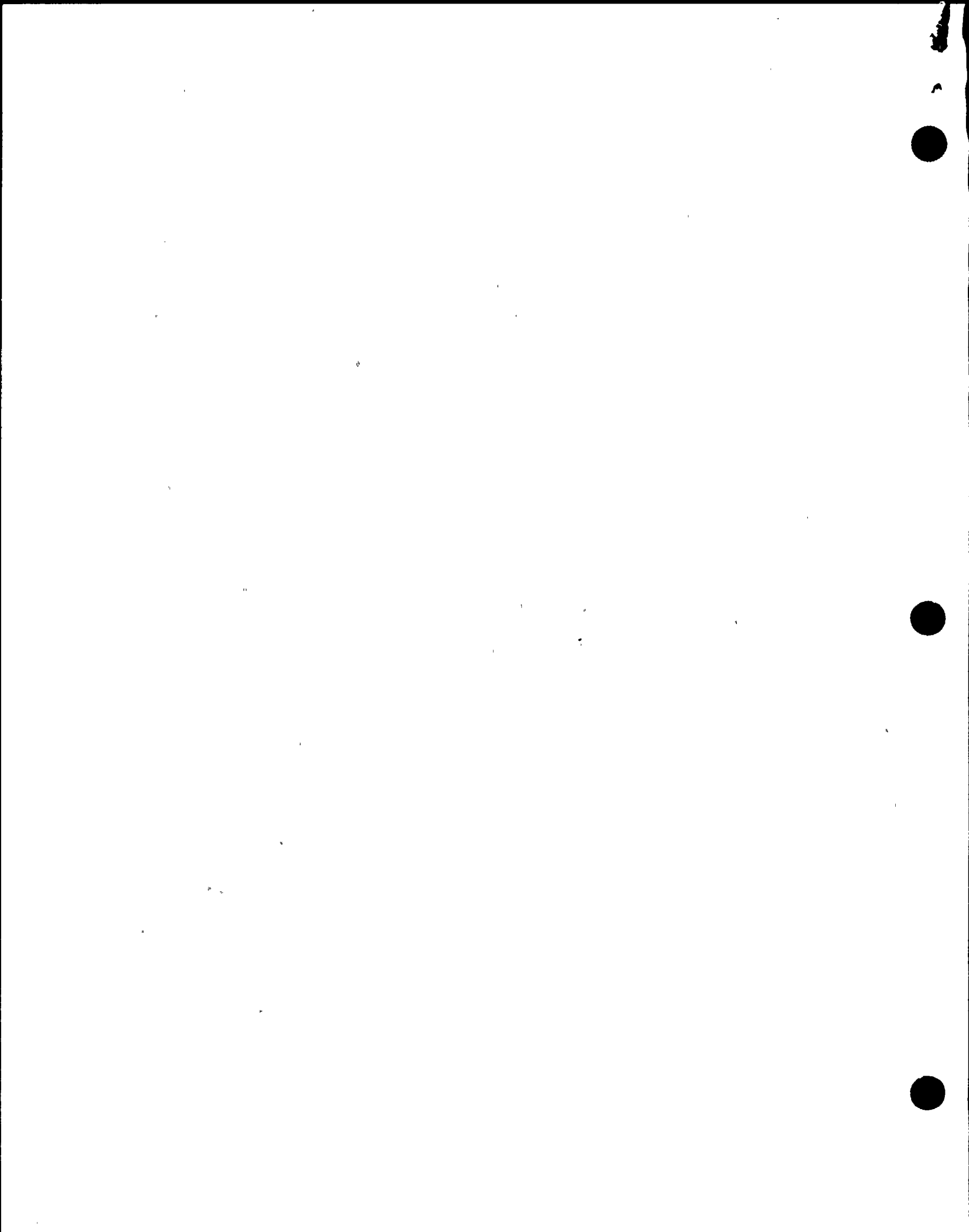
Disposition:

- 1. Incorporate this change during the next scheduled revision.
- 2. Begin revising the lesson plan immediately. Supervisor initiate the process.
- 3. To be used one time only.

Approvals:

Instructor: H.P. Strahler /Date 4/10/91

Training Area Supervisor (or designee): [Signature] /Date 6/10/91

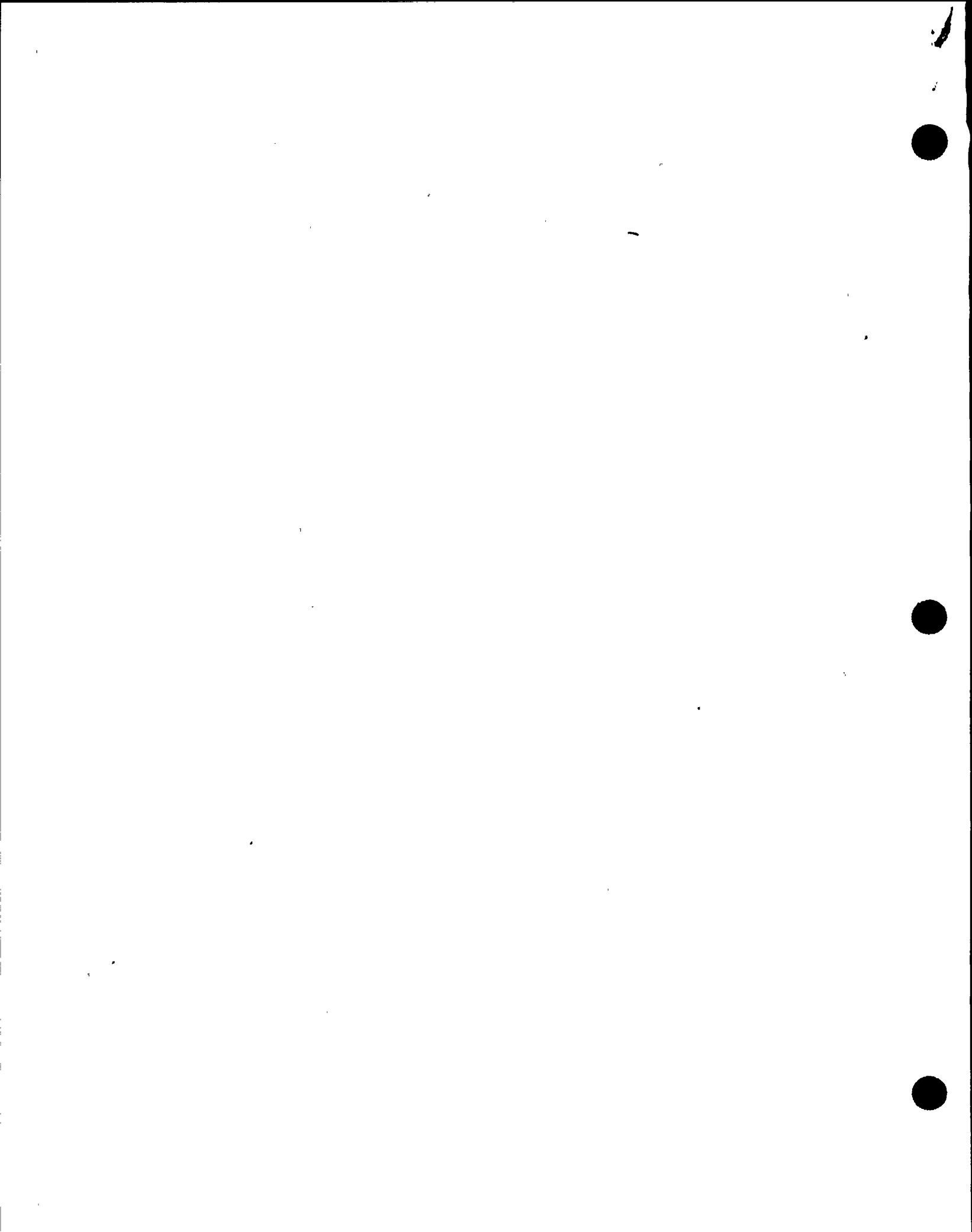


I. TRAINING DESCRIPTION

- A. Title of Lesson: Overview of Technical Specifications |2
- B. Lesson Description: This lesson contains information pertaining to |
the Nine Mile Point Nuclear Station Unit 2 Technical |
Specifications. The scope of the training is defined by the |
Learning Objectives and in general covers the knowledge required of |
a Licensed Control Room Operator. |
- C. Estimate of the Duration of the Lesson: 3 Hours |
- D. Method of Evaluation, Grade Format, and Standard of Evaluation: |
Written exam, passing grade of 80% or better. |
- E. Method and Setting of Instruction: This lecture should be conducted |
in the classroom. |
- F. Prerequisites: |
 - 1. Instructor: |
 - a. Certified in accordance with NTP-16. |
 - 2. Trainee: |
 - a. Initial License Candidate - In accordance with the |
eligibility requirements of NTP-10. |
 - b. Licensed Operator Requal - In accordance with the |
requirements of NTP-11. |
- G. References: |
 - 1. Technical Specifications, Nine Mile Point Nuclear Station, Unit |
2 Appendix "A" to license NPF-69. |
 - 2. 10CFR |
 - a. 10CFR 50.36 Technical Specifications |
 - 3. Technical Specifications Interpretations |

II. REQUIREMENTS

- A. AP-9 Administration of Training
- B. NTP-10 Training of Licensed Operator Candidates
- C. NTP-11 Licensed Operator Requalification Training
- D. NTP-12 Unlicensed Operator Training



III. TRAINING MATERIALS

| 2

A. Instructor Materials:

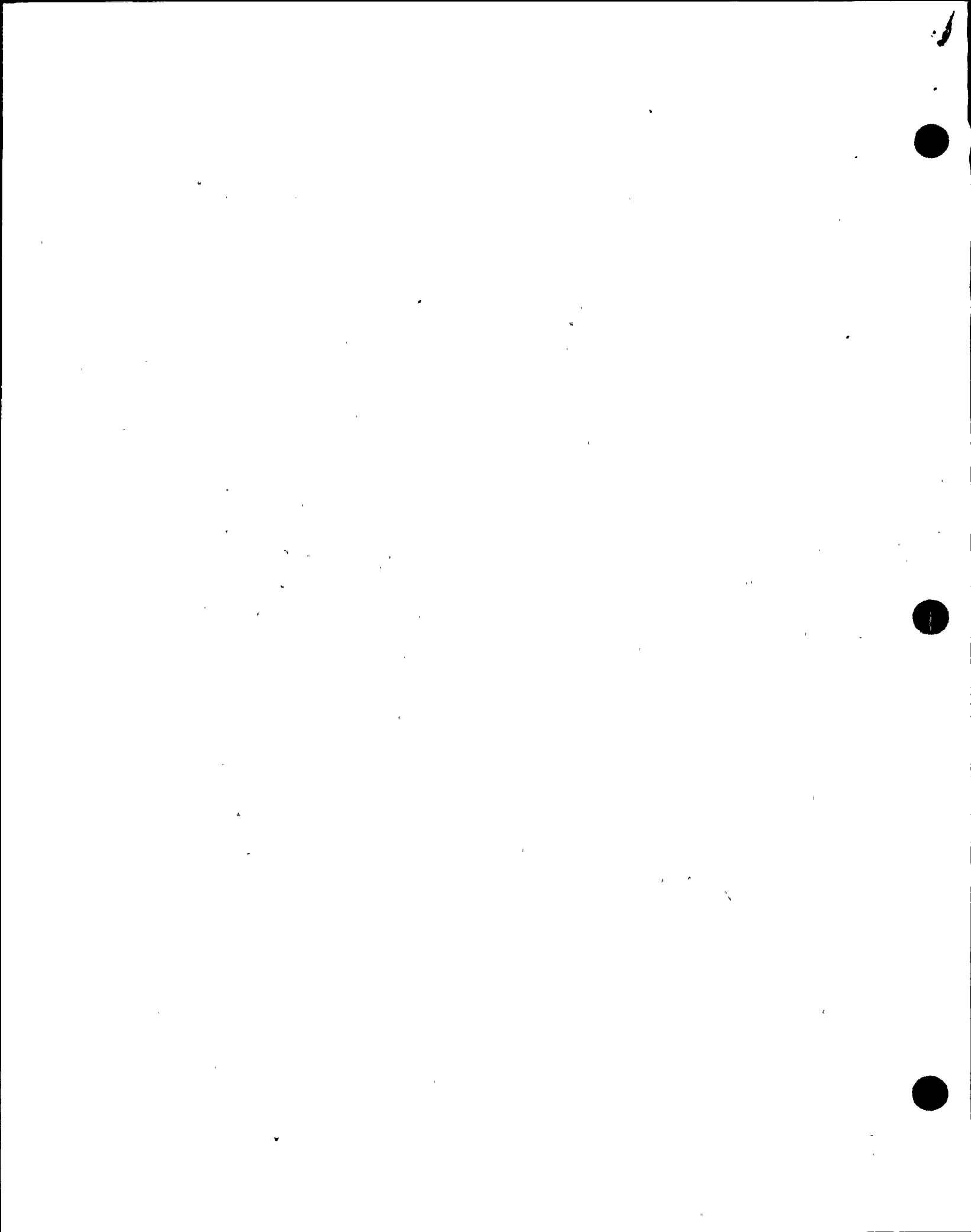
1. Classroom
2. Lesson Plan
3. TR
4. Transparency Package
5. Overhead Projector
6. Applicable References
7. Trainee Handouts
8. Course Evaluation Sheets

B. Trainee Materials:

1. Technical Specifications
2. Technical Specification Interpretations
3. Note Taking Materials

IV. EXAM AND MASTER ANSWER KEYS

- A. Exams will be generated and administered as necessary.
- B. Exams and Master Answer Keys will be on permanent file in the Records Room.



V. LEARNING OBJECTIVES

|2

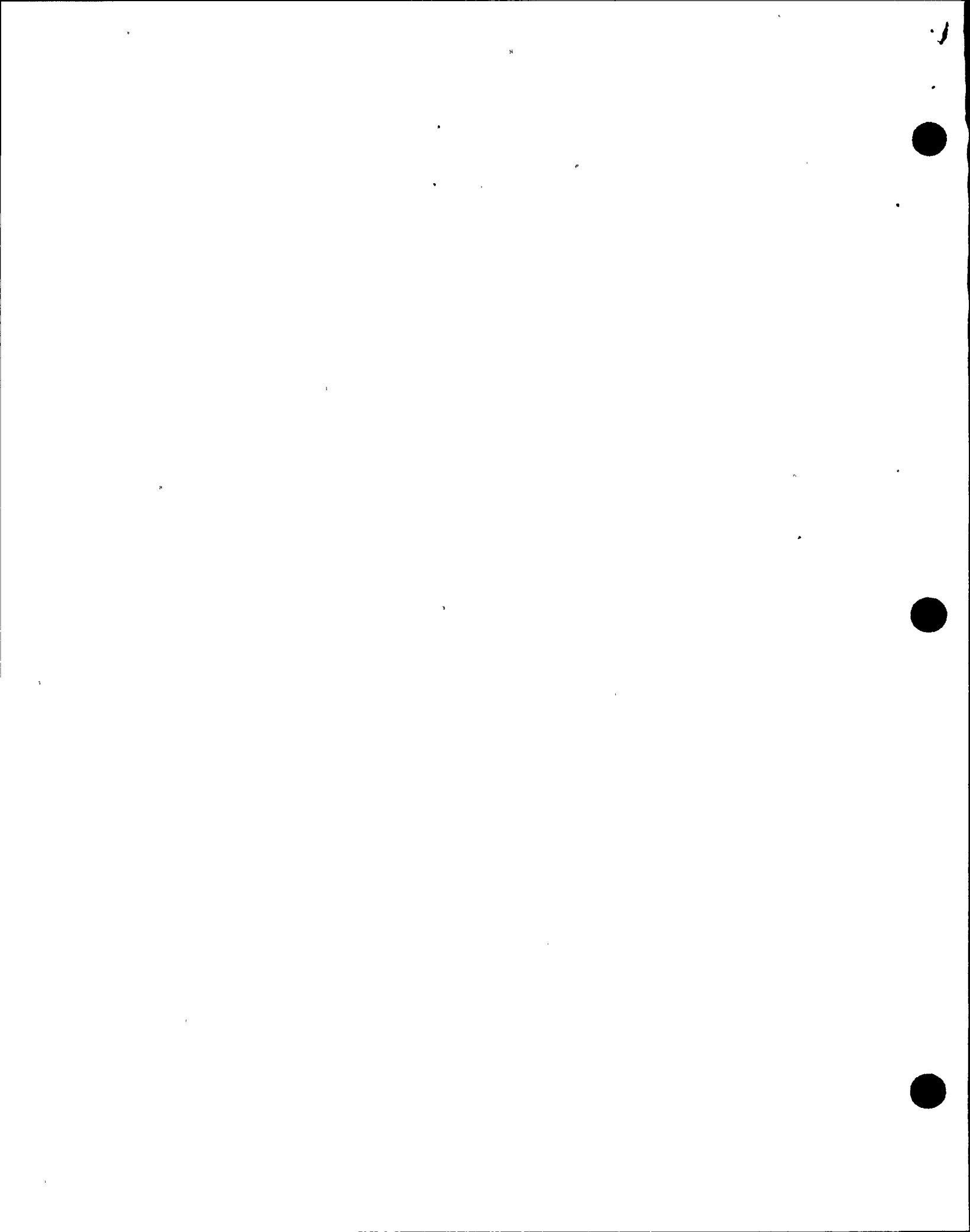
Upon completion of training the trainee will have gained the knowledge to:

A. Terminal Objectives:

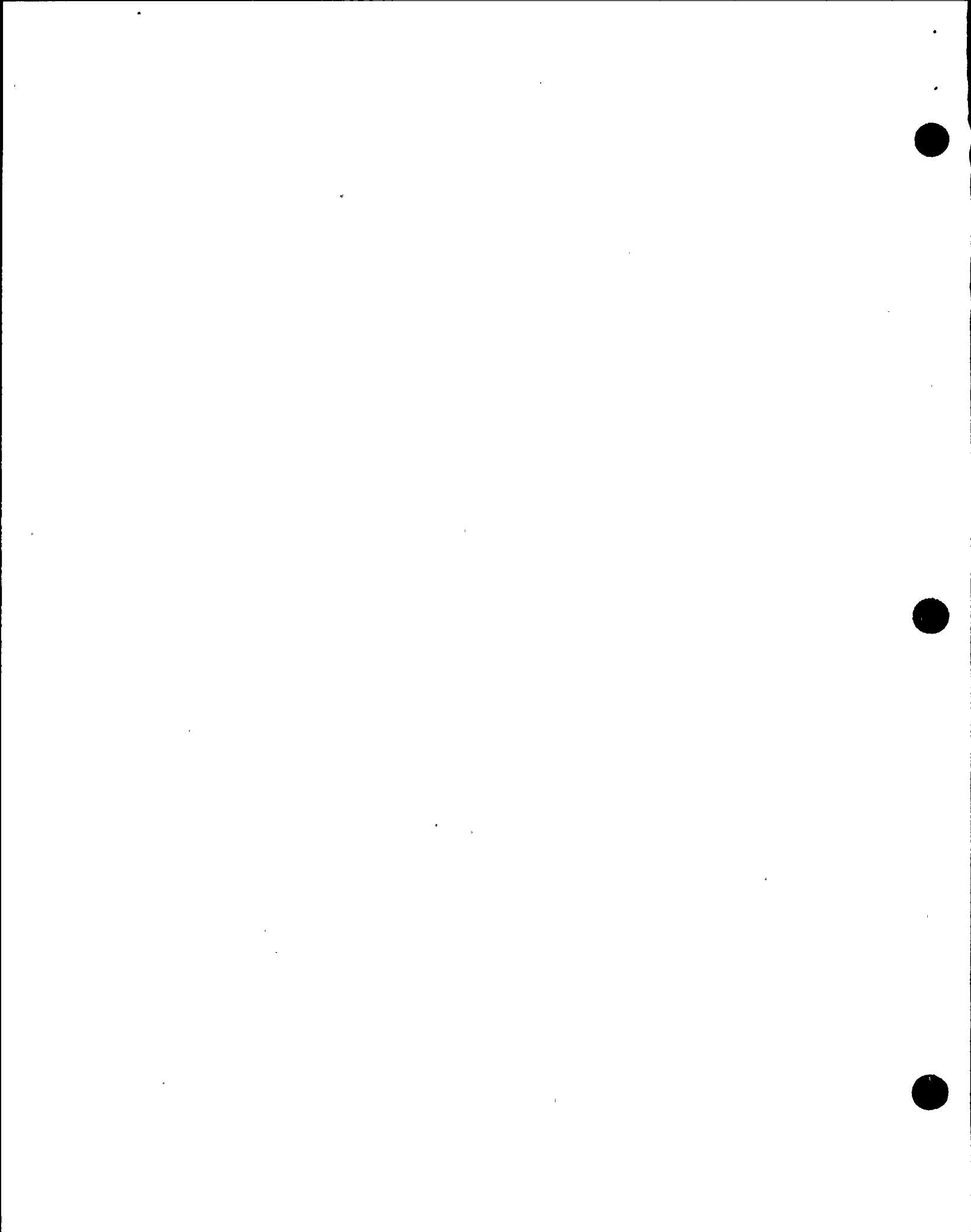
- TO-1 Apply Technical Specification directions for Safety Limits, LCOs, and Limiting Safety System Settings. 3410180303
- TO-2 Evaluate plant system performance and coordinate appropriate actions per Technical Specifications if LCO entered. 3410320303
- TO-3 Evaluate plant conditions and coordinate actions per Technical Specifications of LSSS is reached. 3410330303
- TO-4 Utilize Technical Specifications Interpretations and clarify actions and applications of action statements. 3440310303
- TO-5 Identify LCOs for remaining in the post accident condition during repair efforts. 3440350303
- TO-6 Authorize the deviation from Technical Specifications/EPPs during emergency condition. 3440480303
- TO-7 Review unit conditions to ensure compliance with Technical Specifications during short outages. 3510100103

B. Enabling Objectives:

- EO-1 State the origin of the Technical Specifications.
- EO-2 State the legal obligation of the plant operator in adhering to the Technical Specification.
- EO-3 List the six major sections.
- EO-4 State what is contained in each major section of the Technical Specifications.
- EO-5 Define the following terms concerning the Technical Specifications:
 - a. Safety Limit
 - b. Limiting Safety System Setting
 - c. Limiting Condition for Operation
 - d. Surveillance Requirements



- EO-6 Describe the following sections of a Limiting Condition for |2
Operation: |
a. Applicability |
b. Action |
- EO-7 State, in general terms, what actions are required in the event |
of the violation of the following: |
a. Safety Limit |
b. Limiting Safety System Setting |
c. Limiting Condition for Operation |
d. Surveillance Requirement |



I. INTRODUCTION

A. Learning Objectives

Show TP's of Objectives
 Inform students of method of evaluation.
 Pass out TR
 Pass out Course Evaluation Sheet as
 appropriate

|2

B. Purpose

1. The Technical Specifications are an appendix to the plant operating license that defines minimum operability requirements for safety related systems, components, and/or components and defines actions to be taken if these become inoperable. Administrative controls relative to the safe operation of the plant are also referred.

Show TP of Fig. 2

In other words:
 The Technical Specifications provide instructions and requirements to ensure the reactor plant is operated in a safe manner.

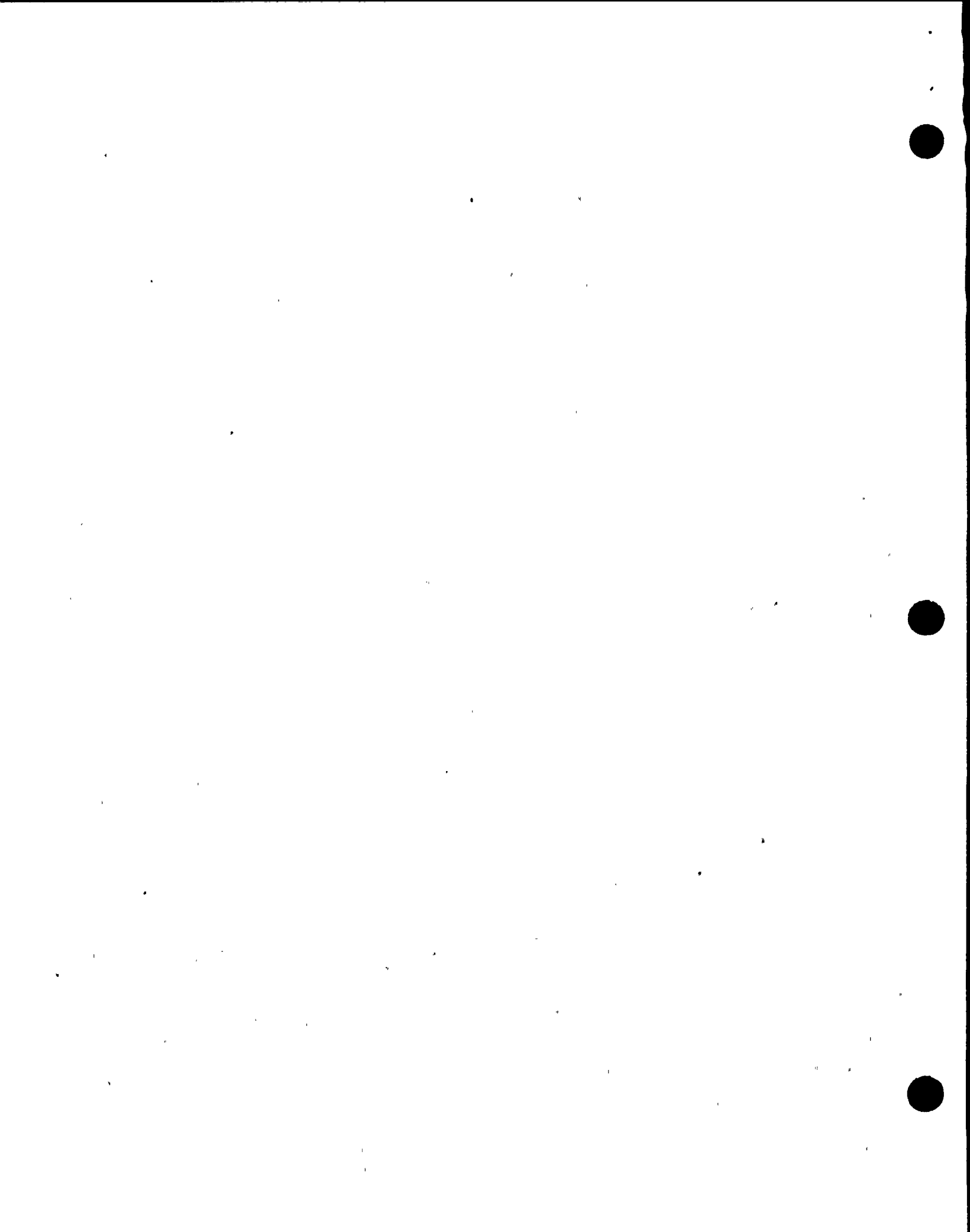
C. Origin of the Technical Specifications

1. Prior to starting construction of a nuclear power plant, a construction application must be submitted to the NRC.
 a. This application includes the Preliminary Safety Analysis Report (PSAR) which includes the first draft of plant technical specifications.

Show TP of Fig. 9

E0-1

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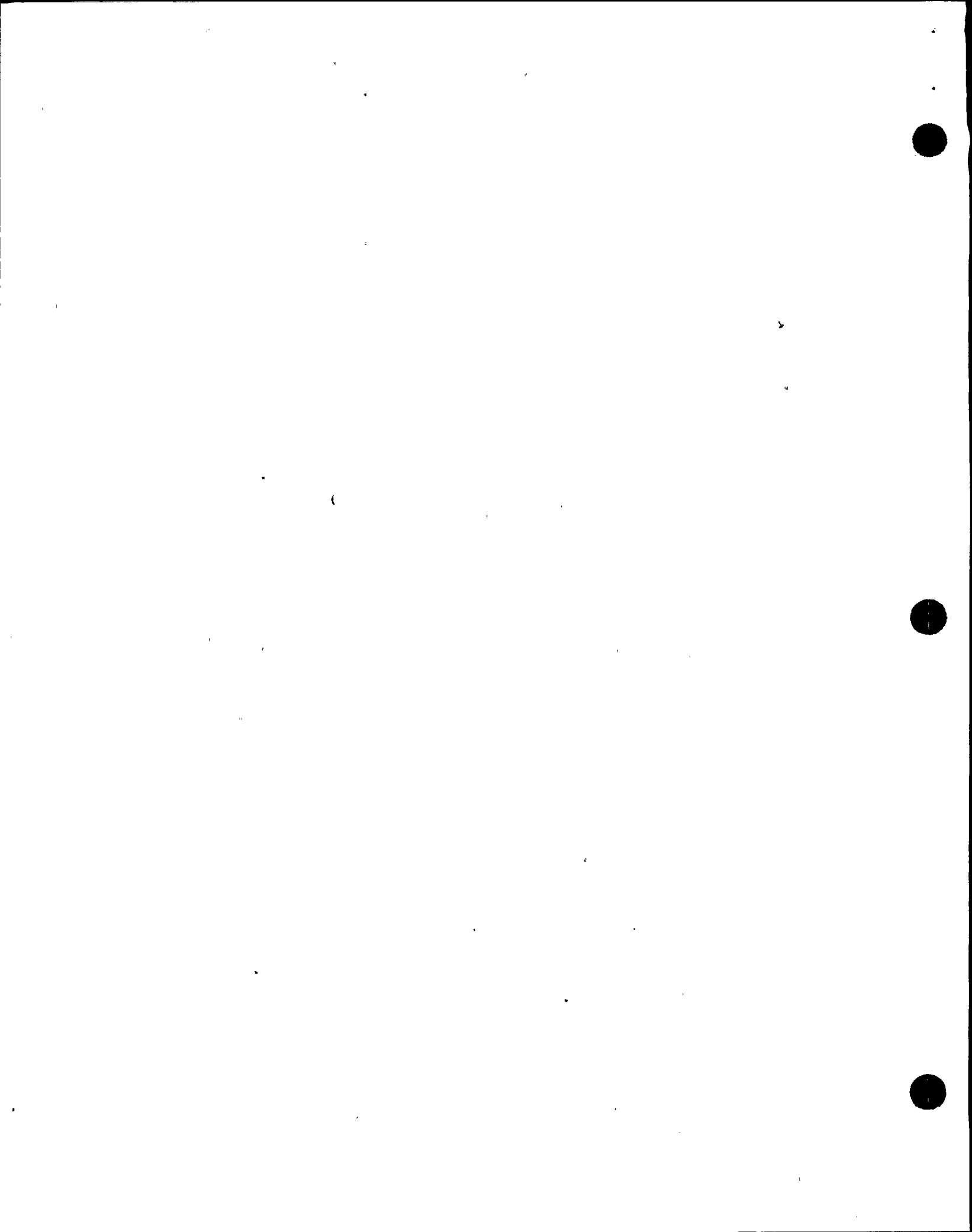
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|--|--|--|
| <ul style="list-style-type: none"> b. The technical specifications are technical requirements for safe operation of the facility. | | |
| <ul style="list-style-type: none"> 2. When application is approved, the NRC issues a Construction Permit (CP) which, <ul style="list-style-type: none"> a. Allows applicant to commence construction. b. Binds applicant to the requirements and laws for construction of a nuclear facility under Title 10 (Energy) of the Code of Federal Regulations. <ul style="list-style-type: none"> 1) The CP is a contractual agreement between utility and U.S. Government. c. Operating License <ul style="list-style-type: none"> 1) After construction, but prior to operation, an application for an Operating License (OL) is made. 2) This application includes the Final Safety Analysis Report (FSAR). | <ul style="list-style-type: none"> Show TP of Fig. 5 TP shows all things that go into a Power Reactor License. | <ul style="list-style-type: none"> 2 2 |



- a) The FSAR includes information that describes the facility, presents design basis, the limits on its operation, and presents a safety analysis of the structures, systems and components of the facility as a whole.
- b) A part of the FSAR are the proposed Technical Specifications. A summary statement of the bases or reasons for such specifications other than those covering admin. controls, shall also be included in the application but shall not become part of the Technical Specifications.
- 3) FSAR Technical Specification Derivation

Use the attached Table of Contents for the FSAR and discuss with class the material found within each Chapter.
7a-7e

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Nine Mile Point Unit 2 FSAR

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Nine Mile Point Unit 2 FSAR

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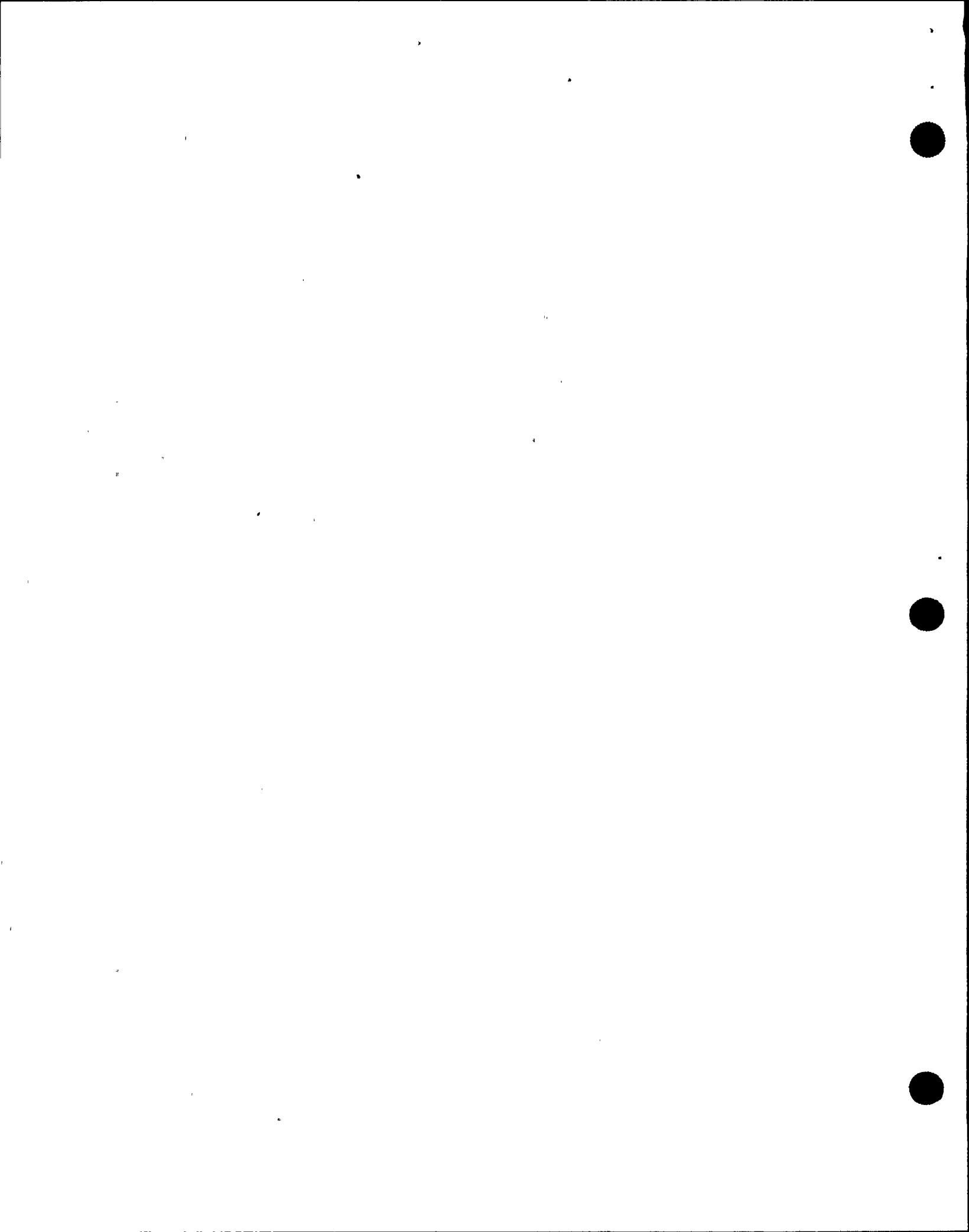
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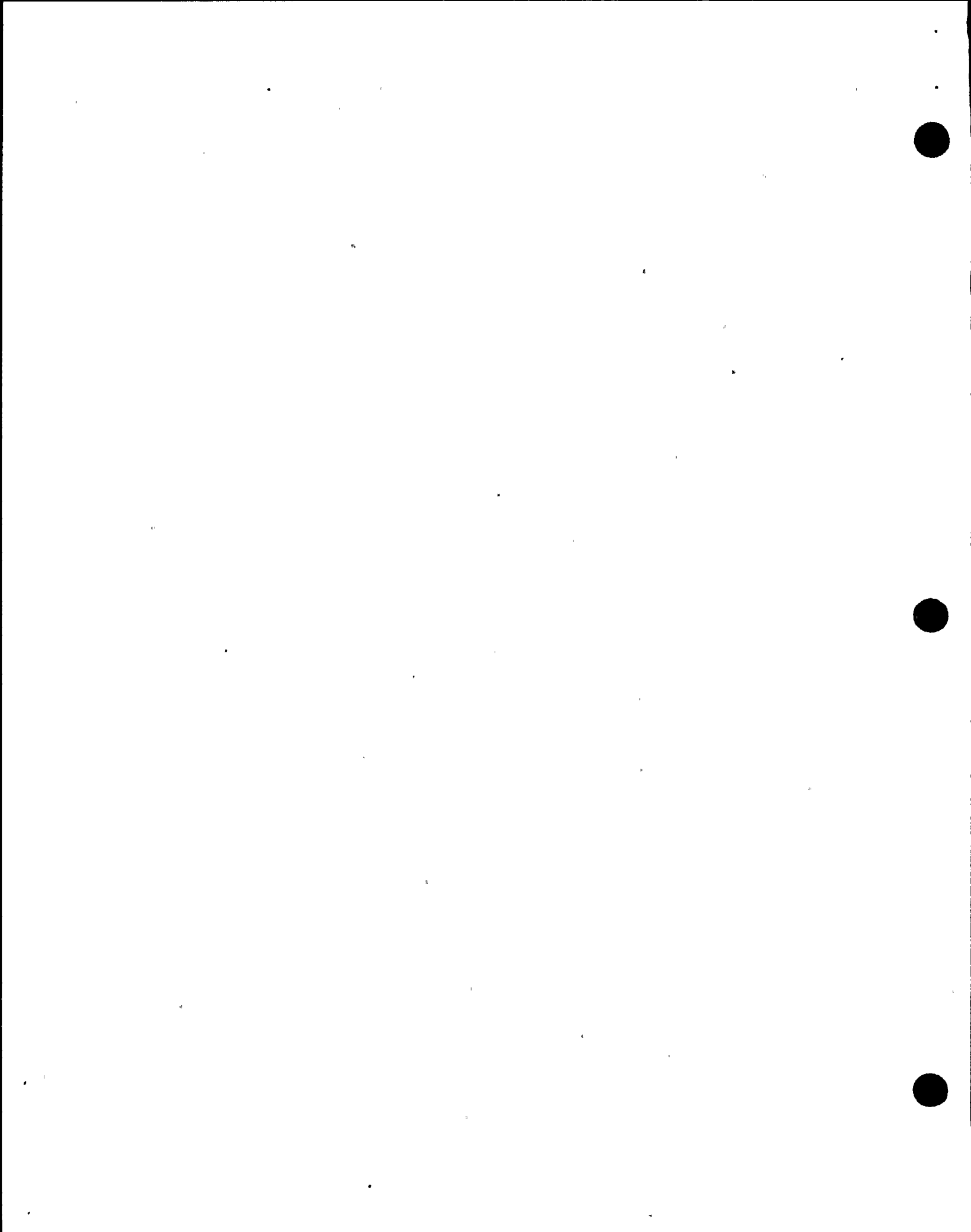


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|---|---|--|
| <p>a) Chapter 15 of the FSAR looks at Accident Analysis or Transient Analysis to determine their consequences and to evaluate the capability built into the plant to control or accommodate such transients/failures.</p> | | 2

 |
| <p>b) Part of the Accident Analysis takes into account Single Active Component Failure (SACF) or Single Operator Error (SOE) to assure a single failure proof success path to acceptable consequences should a transient or accident occur.</p> | <p>Technical Specifications prepared in accordance with requirements of 10CFR50.36. Spec's derived from the analysis and evaluation included in the safety analysis report.</p> |

 |
| <p>c) The complex "operational requirements" are conservatively simplified as a final step in the process so that a practical set of Tech. Specs. and operating procedures may be obtained.</p> | |

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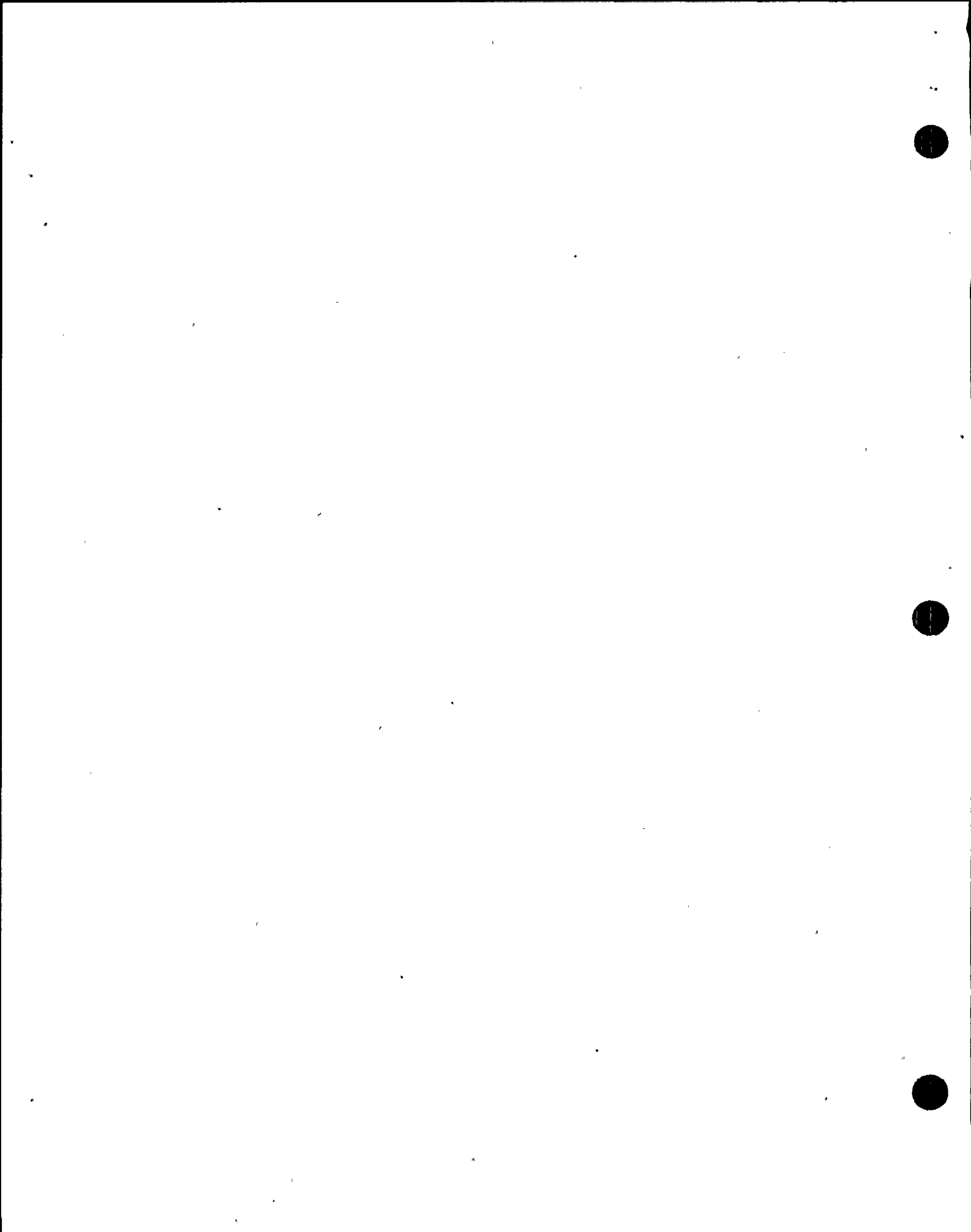




- ii. To assure the existence of a single failure proof success path to acceptable consequences should a transient or accident occur by confirming SACF or SOE criteria conformance.
- e) Two kinds of Operation Requirements for Plant Hardware are derived from the Accident Analysis.
 - i. Limiting Condition for Operation: The required condition for a system while the reactor is operating in a specified state.
 - ii. Surveillance Requirement: The nature and frequency of tests required to assure that the system is capable of performing its essential functions.

|2





- 5) Encouraging improvement of license performance, and by example, that of industry, including the prompt identification and reporting of potential safety problems. Consistent with the purpose of this program, prompt and vigorous enforcement action will be taken when dealing with Licensees who do not achieve the necessary meticulous attention to detail and the high standard of compliance which the NRC expects of its licensees.
3. Violations may lead to
- a. Revocation of the facility's construction permit or operating license.
 - b. The licensee can also be fined for violations of T.S.
4. Individuals are held responsible for willful violations of T.S. and other regulations.

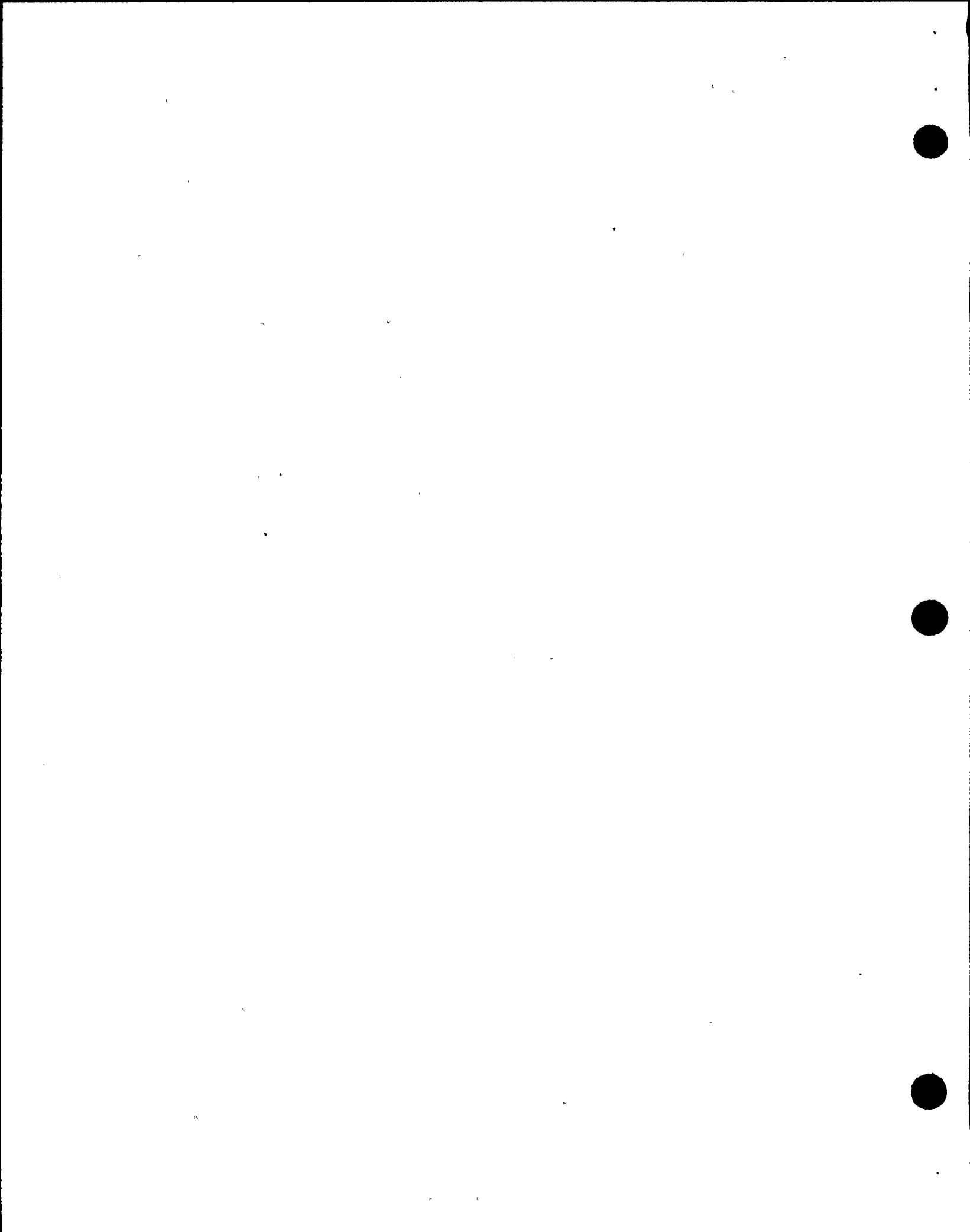
EO-2



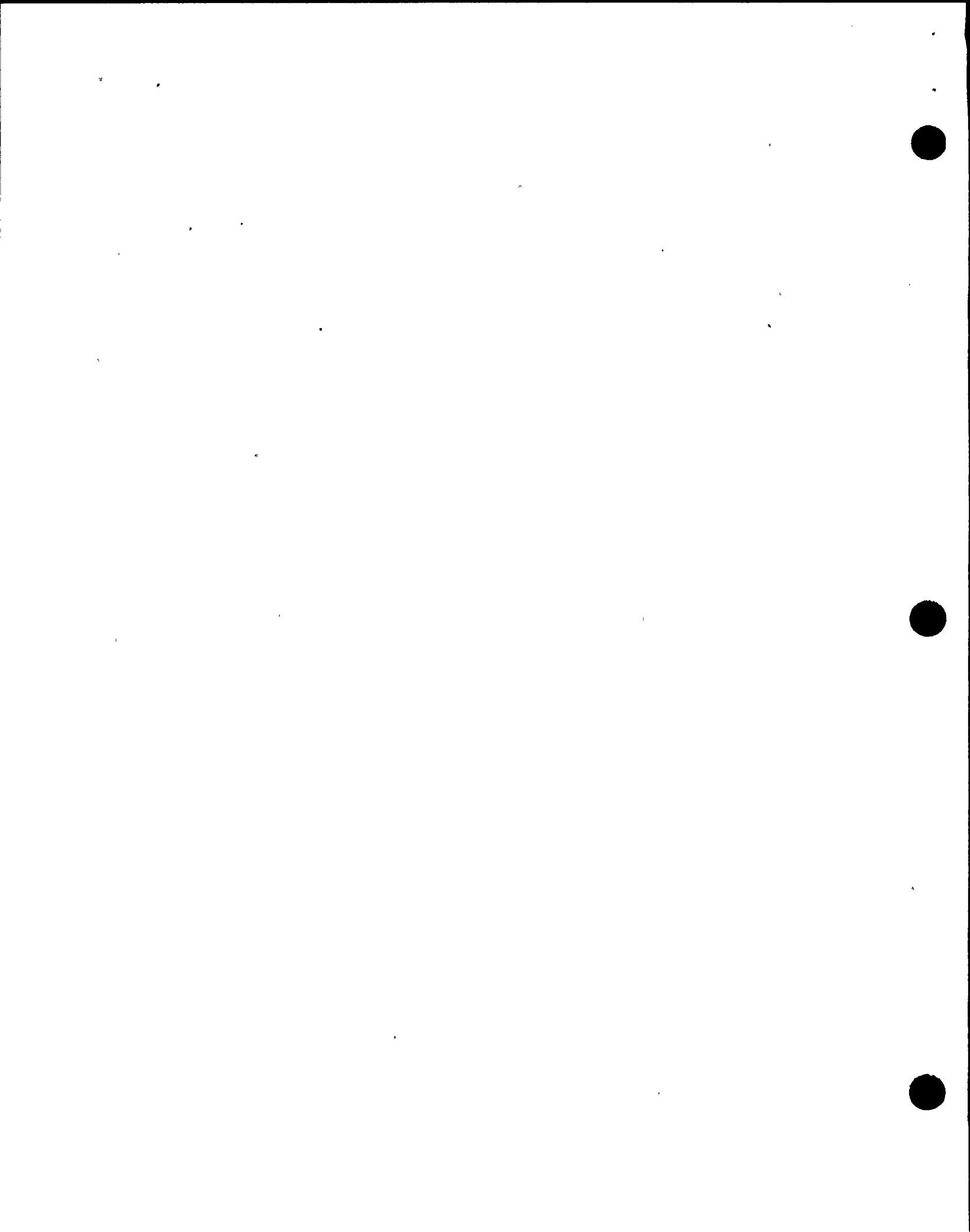
5. 10CFR55.71 states:
 "Any person who willfully violates any provision of the Atomic Energy Act or any regulation or order issued there under may be guilty of a crime and, upon conviction, may be punished by fine or imprisonment or both, as provided by law."

II. DETAILED DESCRIPTION

- | | | | |
|--|--|--|------|
| A. General Content of the Technical Specifications | Show TP of Fig. 19 | | 2 |
| 1. Technical Specifications for all plants are divided into six sections: | Show TP of Fig. 20 | | |
| a. Section One; Definitions | Pages are numbered corresponding to section ie. Section 1 1-1, 1-2, etc. | | |
| b. Section Two; Safety Limits and Limiting Safety System Setpoints | | | |
| c. Section Three; Limiting Conditions for Operation and Bases | | | |
| d. Section Four; Surveillance Requirements | | | |
| e. Section Five; Design Features | | | |
| f. Section Six; Administrative Controls | | | EO-4 |
| 2. Definitions - Section 1 | Show Fig. 21 | | 2 |
| a. Section One; Ensures uniform interpretation of important words or phrases frequently used in Tech. Specs. | | | |



b. The defined terms appear in capitalized type and shall be applicable throughout the Tech. Specs.	Use TP of Operational Conditions to discuss/cover this section of the Lesson Plan.		2
c. Included in the definitions section	Pay attention to the notes on the TP		
1) Operational Conditions used to define plant status			
2) Mode 1 Power Operation			
3) Mode 2 Start Up			
4) Mode 3 Hot Shut Down			
5) Mode 4 Cold Shut Down			
6) Mode 5 Refueling			
3. Safety Limits Section 2			
a. A Safety Limit is defined as: Limits upon important process variables which are found to be necessary to reasonably protect the integrity of certain physical barriers which guard against the uncontrolled release of radioactivity.	Show TP of Fig. 24	EO-4	
	Show TP of Fig. 25	EO-5a	
b. Safety Limit Format	Refer candidates to their T.S.'s		2
1) Operational statement and specific limit.	Show TP of Safety Limits		
2) Applicability statement (when it applies)			



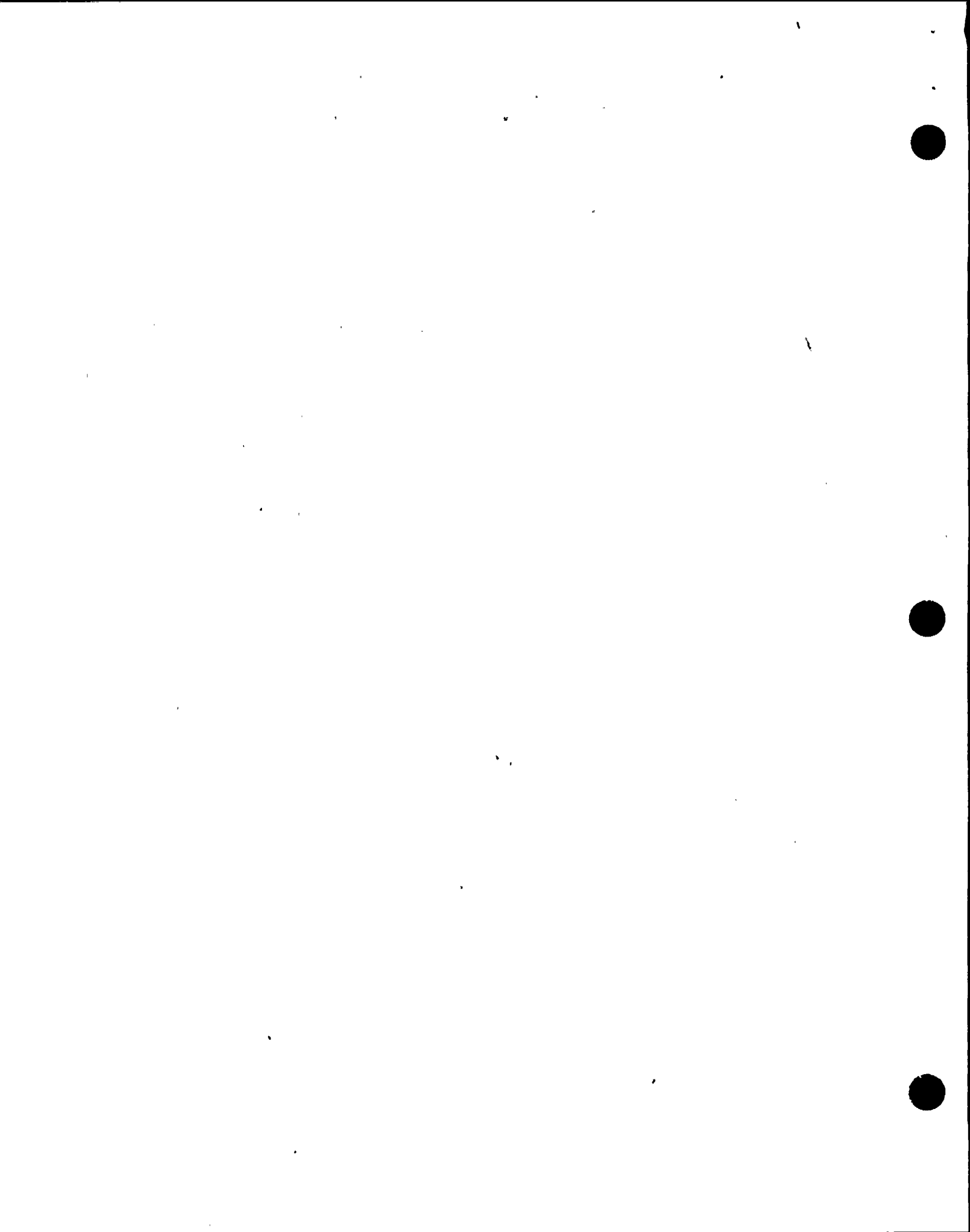
- 3) Action statement - what to do if the limit is exceeded.
- c. Bases provides reasons and explanations of the safety limit; they are not part of the safety limit.
- d. 4 Safety limits
 - 1) Thermal Power Low Pressure OR Low Flow
 - 2) Thermal Power High Pressure AND High Flow
 - 3) Reactor Coolant System Pressure
 - 4) Reactor Vessel Water Level
- e. Consequences of violating safety limits
 - 1) Reactor shut down, until restart is authorized by NRC.
- 4. Limiting Safety System Settings - LSSS Section 2 (continued)
 - a. LSSS defined:
 - The setting for automatic protective devices related to those variable having significant safety functions.

Refer candidates to Section 6 of their Tech Specs. Section 6.7 Safety Limit Violation

Show TP of Fig. 26

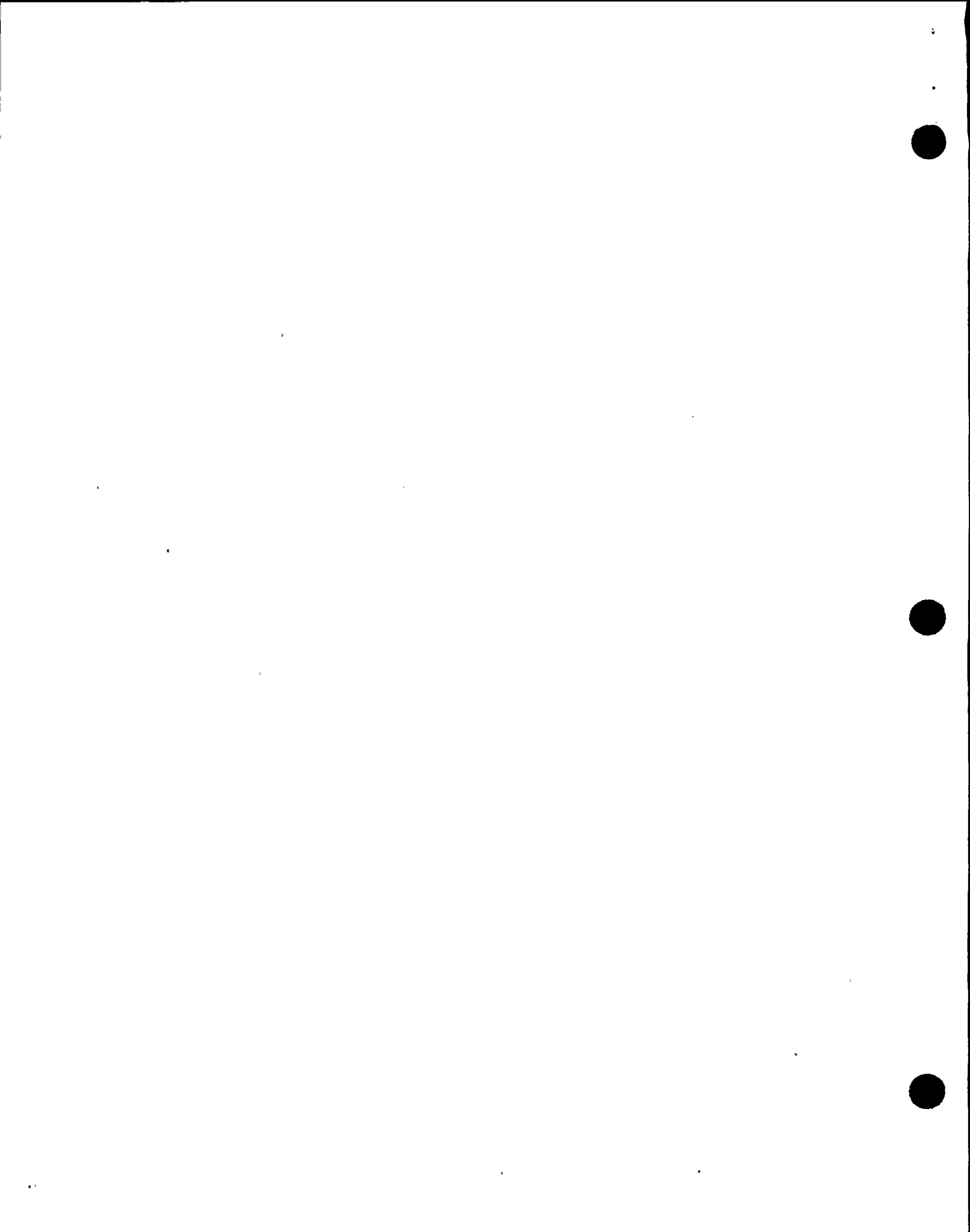
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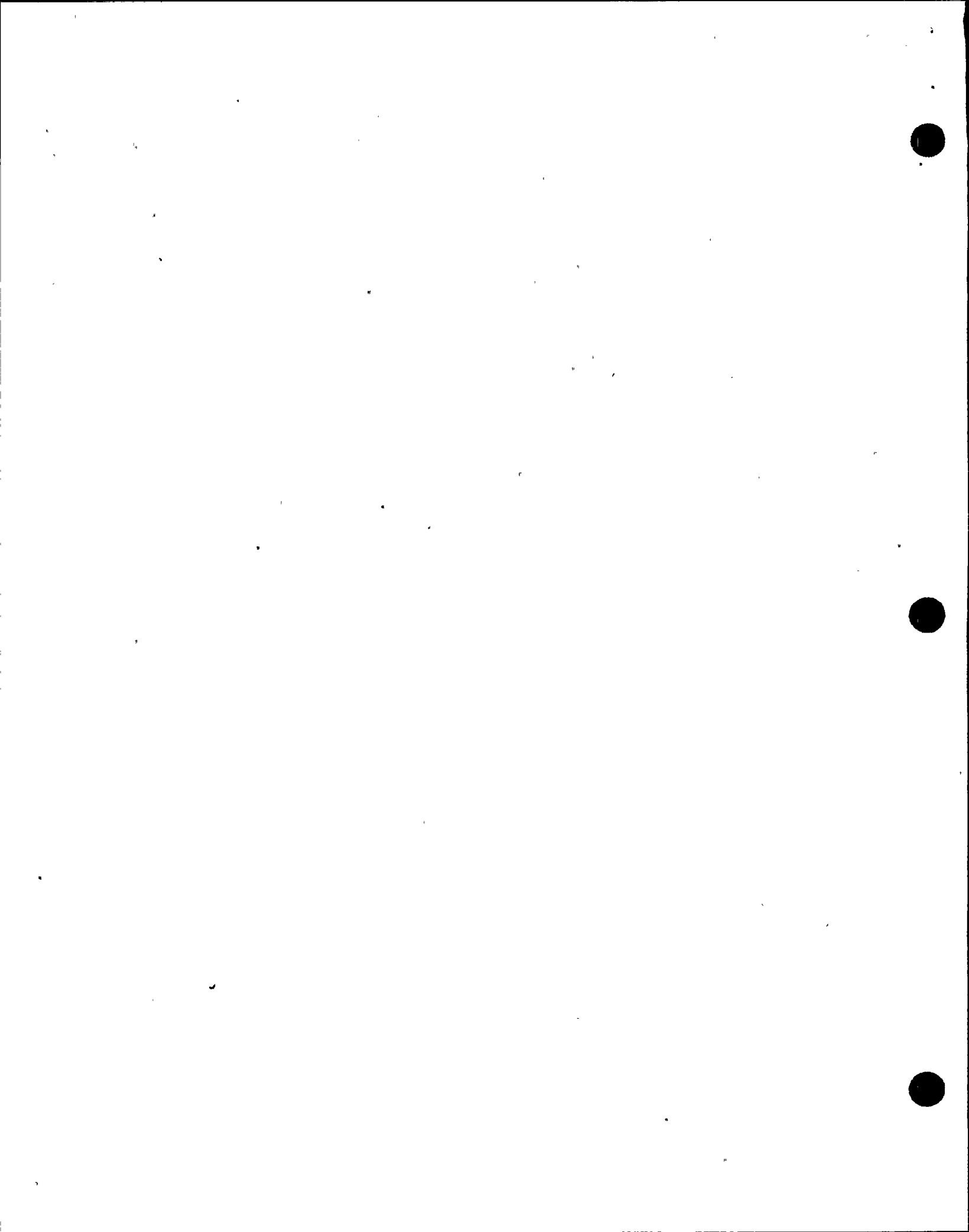
EO-4
EO-5b



- | | | |
|---|---|---|
| <p>b. Where a limiting safety system setting is specified for a variable on which a safety limit has been placed, the setting shall be so chosen that automatic protective action will correct the abnormal situation before a safety limit is exceeded.</p> <p>c. Presented in tabular form including:</p> <p>1) Parameter</p> <p>2) Trip Setpoint</p> <p>3) Trip Setpoint Allowable Value</p> <p>d. Bases for trips and setpoints are provided for justification.</p> <p>5. Limiting Condition for Operation-LCO, Section 3</p> <p>a. LCO defined:</p> <p>The lowest functional capability or performance levels of equipment required for safe operation of the facility.</p> <p>b. Limiting conditions of operation and surveillance are subdivided into twelve (12) sections covering:</p> | <p>Show TP of Fig. 27</p> <p>Refer candidates to Section 2 of their T.S. Section 2.2, page 2-2, note page number designation 2-2.</p> <p>Show TP of Table 2.2.1-1</p> <p>After covering the generalities of 5a, b and c have candidates refer to their T.S.'s section 3/4.0</p> <p>Show TP of Fig. 28 & 29</p> <p>Applicability - read and discuss each paragraph - 3.0.1 through 3.0.4</p> <p>IMPORT a TS Interp. exists for paragraph 3.0.4 TSI*62 show candidate a TP of this TS Interpretation.</p> | <p> 2</p> <p>E0-4
E0-5c</p> <p> 2</p> <p> 2

 </p> |
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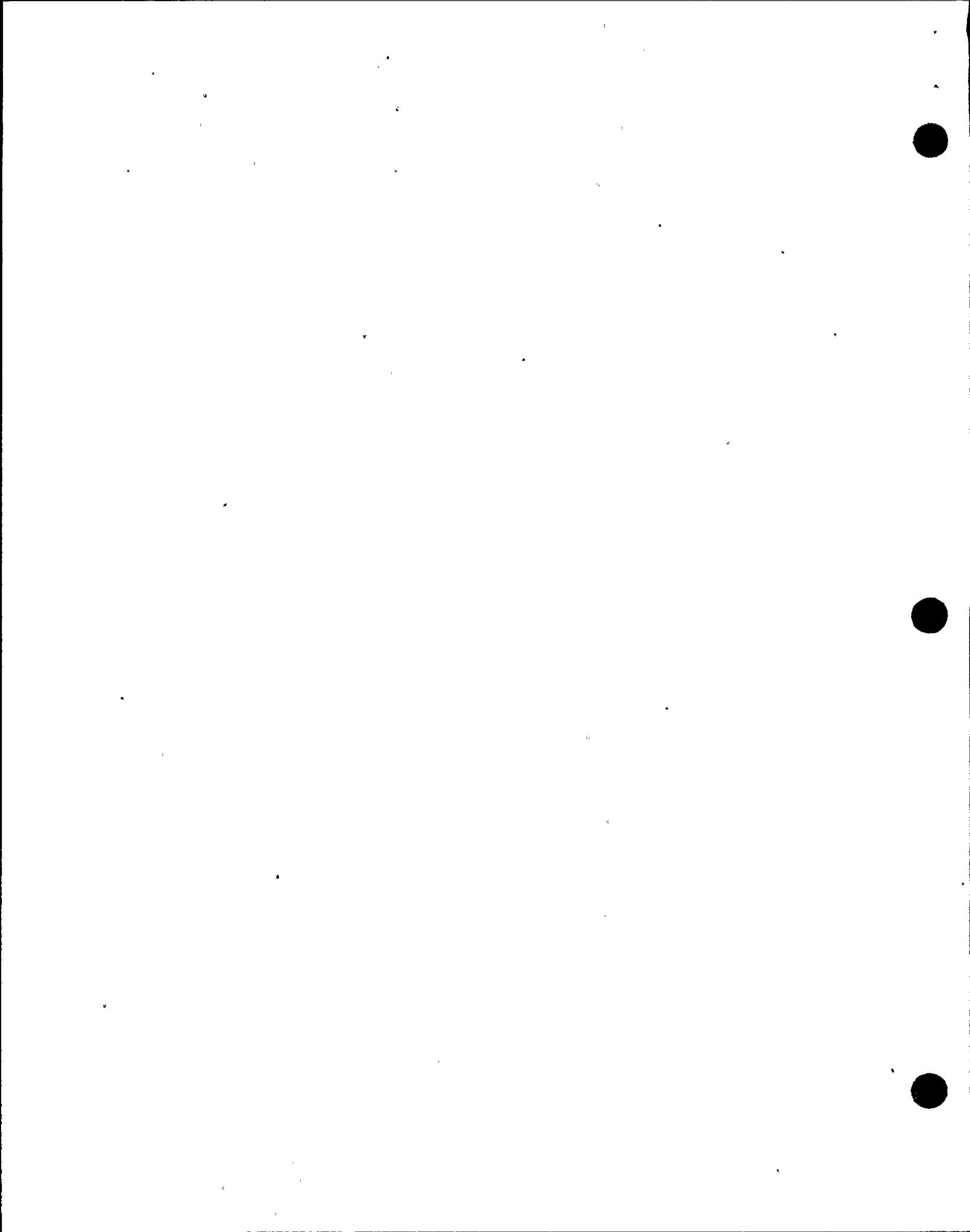


- 7) Plant Systems - Service water, RCIC, Control Room emergency filtration systems.
- 8) Electrical Power System - A.C. sources, and on-site power distribution systems.
- 9) Refueling Operations - Reactor mode switch, instrumentation, water level, control rod position, crane and hoist operability, communications and others.
- 10) Special Test Exceptions - Shutdown margin tests, primary containment integrity, rod sequence control system, oxygen concentration and others.
- 11) Radioactive Effluents - Liquid effluents, dose, liquid waste treatment, gaseous effluents, gaseous waste treatment, solid radwaste.
- 12) Radiologic Environmental Monitoring - Program as directed in OCDM.
- a) Format
- i. Condition statement

OCDM - Off Site Dose Assessment Manual

EO-6

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- ii. Applicability - under what operational conditions the LCO is in effect
 - iii. Action - tells what must be done and how much time is allowed when exceeding the condition statement.

- 6. Surveillance Requirements Section 4
 - a. Defined:

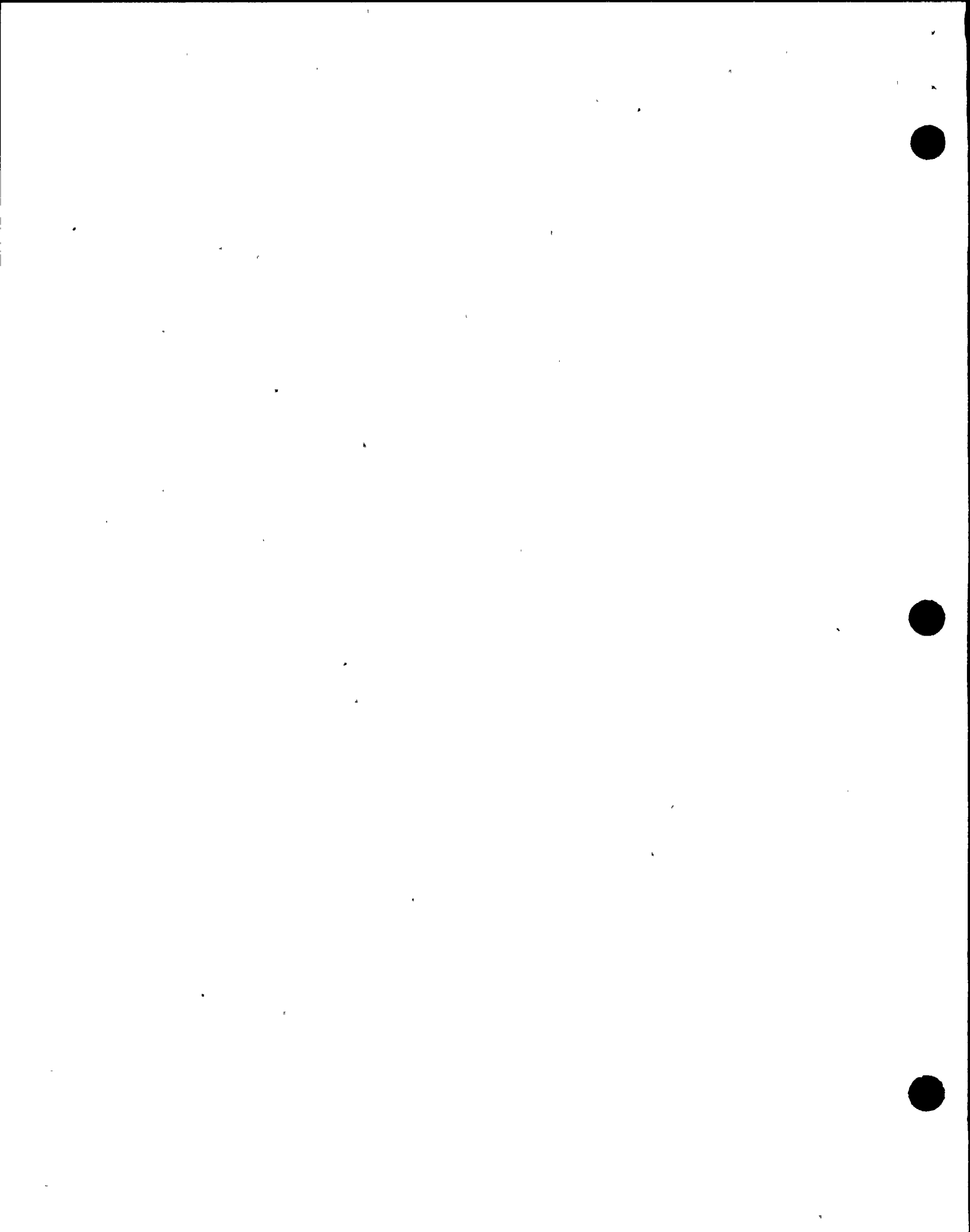
Requirements relating to test, calibration, or inspections to assure that the necessary quality of systems and components is maintained, that facility operation will be within the safety limits and that the LCO's will be met.
 - b. States the actions required and their specified frequency to verify or demonstrate operability of the system or components.

Show TP of Fig. 33

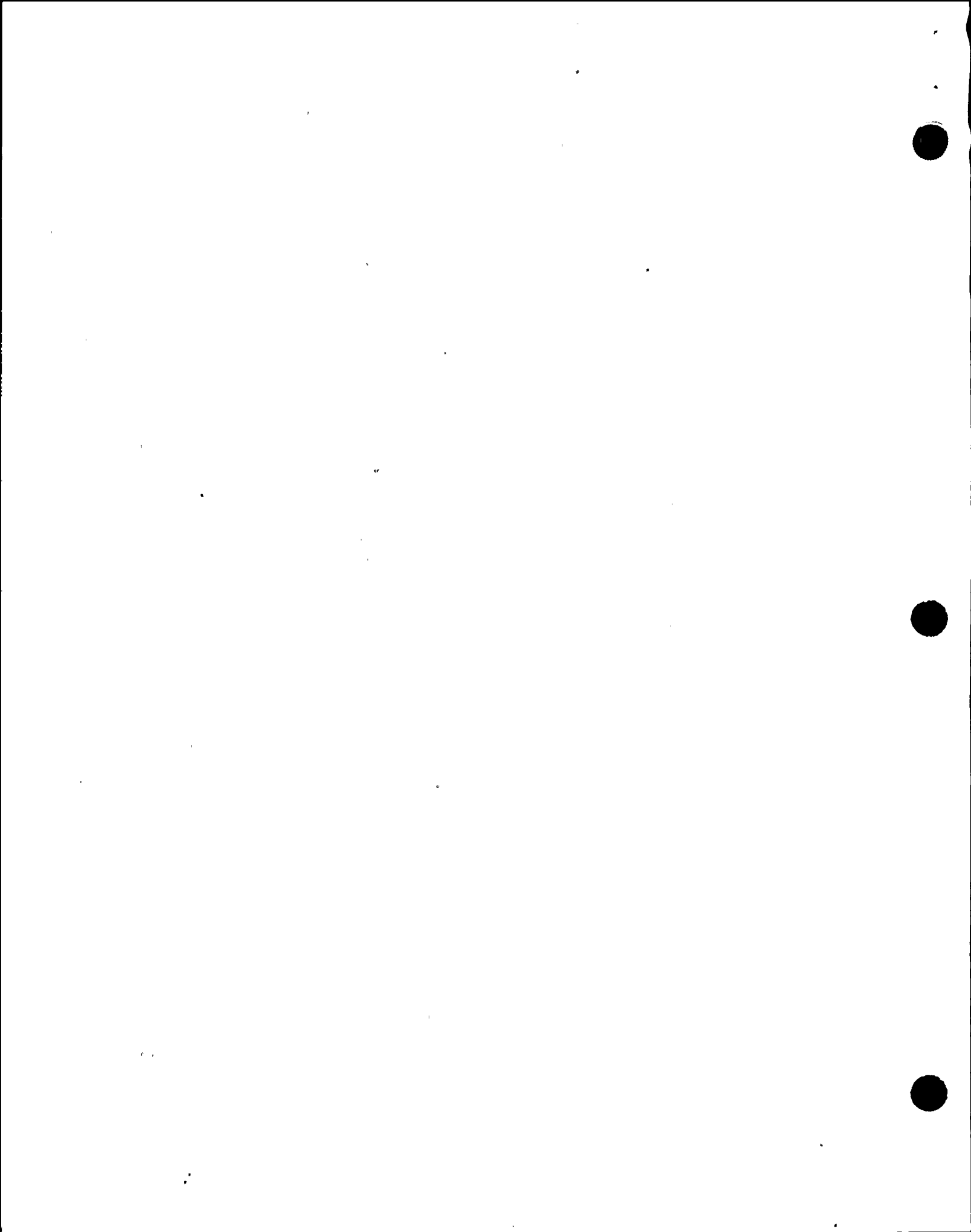
Show TP of Fig. 34

After covering 6a, b, refer candidates to their T.S.'s Section 4, Surveillance Requirements read and discuss paragraphs 4.01 through 4.04 and the table in paragraph 4.05.

EO-4
EO-5d



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| <p>7. Bases - Section B</p> <p>a. Explains the reason for an LCO.</p> <p>b. Tech. Spec. bases are <u>NOT</u> part of the legally binding specifications, but they are required by Code of Federal Regulations.</p> | <p>Show TP of Fig. 39</p> <ul style="list-style-type: none"> • Refer candidates to this section of their T.S. Emphasis that the SRO's in particular are required to be knowledgeable of the bases. • Caution the RO's that they are not exempt from also being familiar with the bases. • Interesting facts in this section | <p>EO-4 2</p> |
| <p>8. Design Features - Section 5</p> <p>a. Those features of the facility such as materials of construction and geometric arrangements, which, if altered or modified, would have a significant effect upon safety and are not covered by safety limits, LCO's or Surveillances.</p> <p>b. Examples of Design Features:</p> <ol style="list-style-type: none"> 1) Core descriptions 2) Containment configuration 3) Component cyclic limits | <p>Refer candidate to this section of their T.S.'s</p> <p>Show TP of Fig. 36</p> | <p>EO-4 2</p> |
| <p>9. Administrative Controls - Section 6.</p> <p>a. Provisions relating to organization and management, procedures, recordkeeping, review and audit, and the reporting system necessary to assure operation of the facility in a safe manner.</p> | <p>Refer candidates to this section of their T.S.'s.</p> <p>Show TP of Fig. 37, 38</p> | <p>EO-4 2</p> |



- b. Policies and requirements required by Code of Federal Regulations.

B. Technical Specifications Usage

1. Operation of Nine Mile Point 2 is accomplished by joint effort of various departments and the use of appropriate instructions, procedures, and guidelines.
2. Procedures and other instructions should reference the applicable Tech. Specs.
3. General Actions Required for Violation of Technical Specifications.

a. Safety Limit Violation

- 1) Shutdown Reactor
- 2) Notify NRC
- 3) Review the matter and record the results. In the written report:
 - a) State the cause of the violation.
 - b) Describe the corrective action to prevent reoccurrence.
- 4) Operation of reactor may not resume until authorized by NRC.

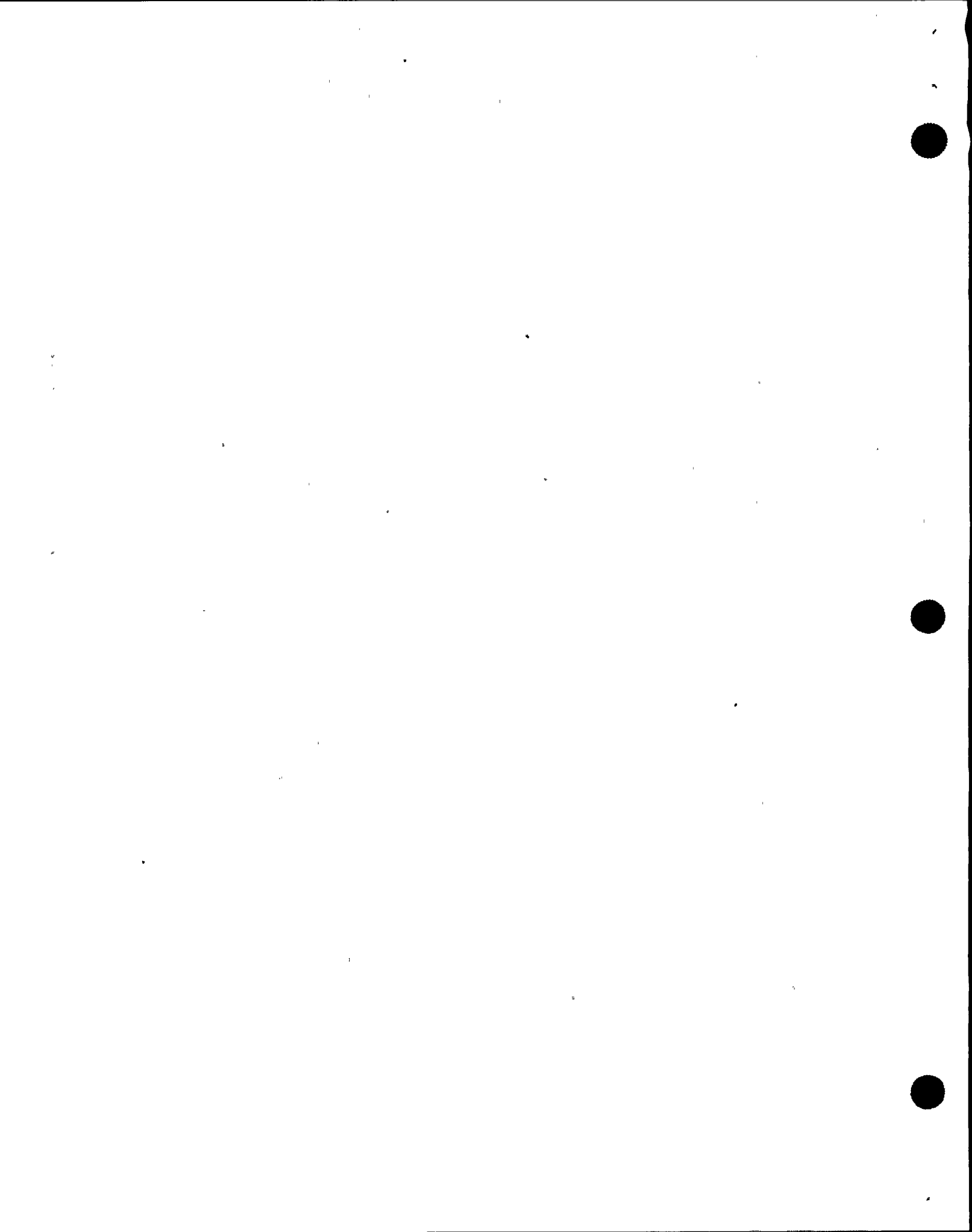
Ask candidate if they recall what the Safety Limits are.

EO-7a |2
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b. Limiting Safety System Setting Violation

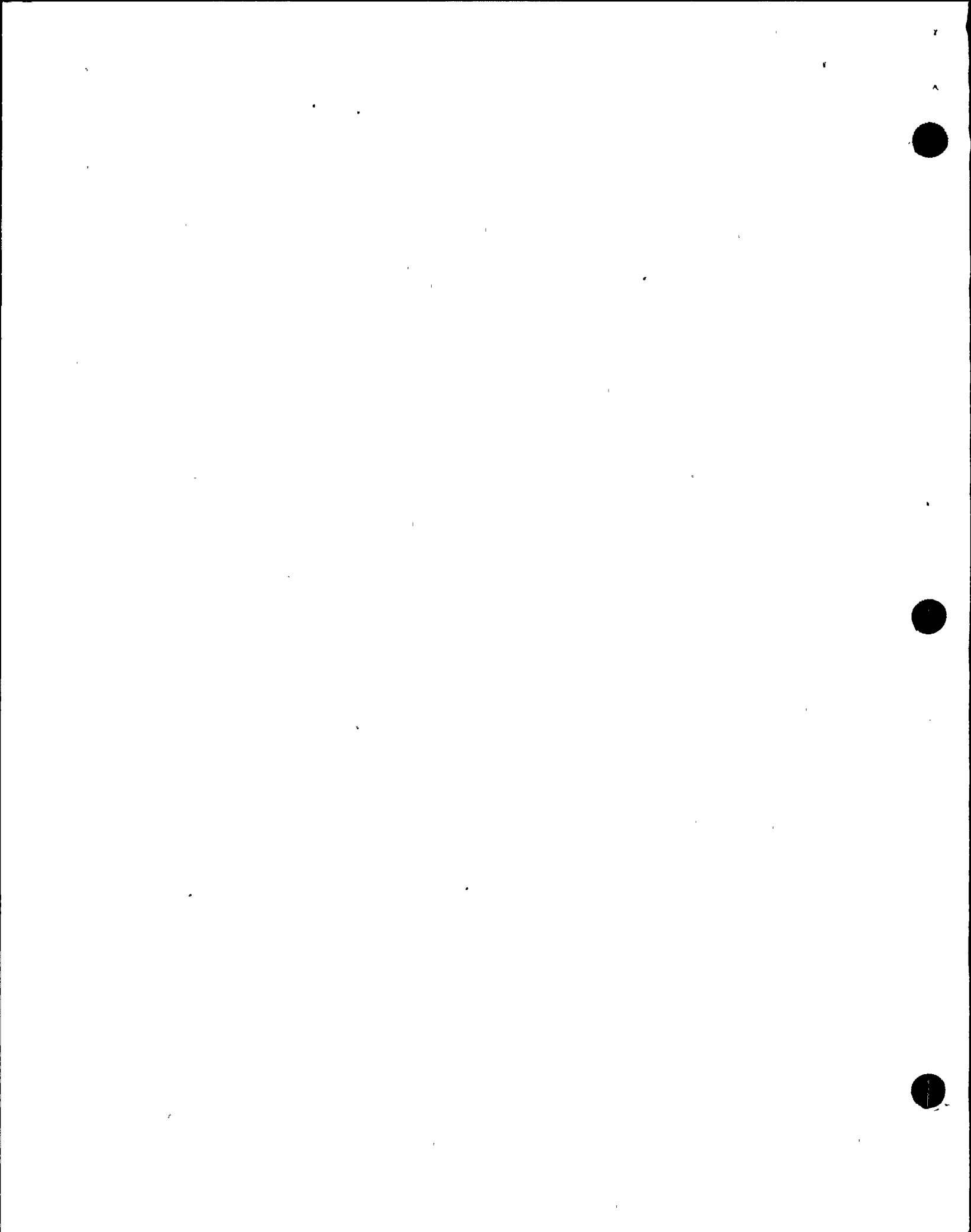
- 1) Take actions per T.S. to correct the violation.

EO-7b |2
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|--|--|--------------|-----------------------------|
| <ul style="list-style-type: none"> 2) Notify NRC <ul style="list-style-type: none"> a) Review the violation b) State the cause and the condition of the violation. c) Describe the corrective action taken to prevent reoccurrences. | | | |
| <ul style="list-style-type: none"> c. <u>Limiting Condition for Operation Violation</u> <ul style="list-style-type: none"> 1) Take the remedial action described in the T.S. 2) Notify NRC stating: <ul style="list-style-type: none"> a) Cause b) The corrective action taken to preclude reoccurrence. | <p>10CFR50.72 "Immediate notification requirements for operating nuclear power reactor" directs the notification of the NRC.</p> | <p>EO-7c</p> | <p> 2

 </p> |
| <ul style="list-style-type: none"> d. <u>Surveillance requirement violation</u> <ul style="list-style-type: none"> 1) Declare the component inoperable and take action in accordance with the appropriate LCO. 2) Failure to perform a Surveillance Requirement within the specified time interval shall constitute a failure to meet the operability requirements for an LCO unless otherwise stated in the individual specification. | | <p>EO-7d</p> | |



4. Inform the proper authorities of T.S. Violation. The station shift supervisor will be responsible to inform the appropriate management personnel and any Regulatory Agency as per Administrative Procedures.

5. Interpretation of Tech. Specs.

a. When Tech. Spec. Interpretation is a concern, the specification should be referred to an immediate supervisor, for resolution.

b. The final interpretation will be resolved by Licensing.

1) This resolution is kept as a permanent record in a Tech. Spec. interpretation binder in the Control Room.

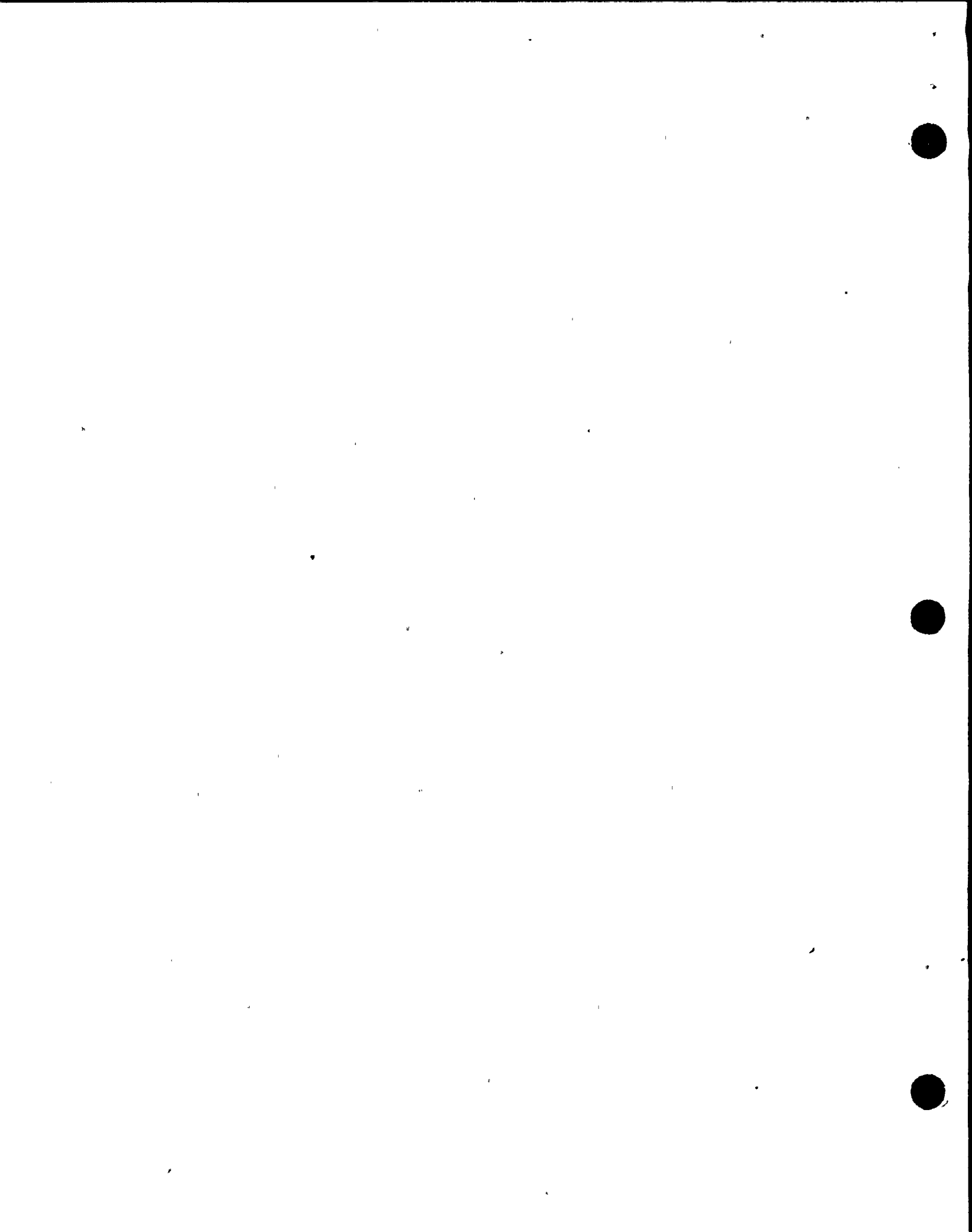
c. The conservative approach should be followed until resolution is received.

6. Communication

a. Operations personnel should be kept informed of maintenance or other evolutions that impact Tech. Spec. requirements.

Refer candidates to their copy of
Technical Specification Interpretations

|2
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7. Amendments to T.S.
 - a. May be required due to plant modification or regulatory changes.
 - b. Submit a formal request for changes to NRC for their approval.
 - c. If approved, FSAR and T.S. are amended.
- C. Wrap Up
 1. Go over learning objectives again

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