



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 129 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated April 24, 1992, Niagara Mohawk Power Corporation (the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1), Technical Specifications (TS). The requested changes would revise Technical Specification 3.4.3/4.4.3 (Access Control) to:

- (a) define the operating conditions under which the specification applies,
- (b) include an allowable outage time for continued plant operation while restoration of secondary containment integrity is underway,
- (c) provide action statements associated with the loss of secondary containment due to access control, and
- (d) provide periodic surveillance requirements for access doors other than the core spray and containment spray pump compartments.

2.0 EVALUATION

Currently, NMP-1 TS 3.4.3/4.4.3 (Access Control) does not: (1) explicitly specify when the Limiting Condition for Operation (LCO) is applicable, (2) specify an allowable outage time for continued plant operation while restoration of secondary containment integrity is underway, (3) provide action statements (corrective action instructions) to be followed in the event of a loss of secondary containment integrity due to a loss of access control, or (4) provide periodic surveillance requirements for secondary containment access doors other than the core spray and containment spray pump compartments. The proposed change would add these provisions to NMP-1 TS 3.4.3/4.4.3:

The proposed changes would specify that the requirements of TS 3.4.3/4.4.3 would be applicable: (1) whenever the reactor is in the power operating condition, or (2) whenever irradiated fuel is being handled in the reactor building, or (3) during core alterations, or (4) during irradiated fuel cask handling operations in the reactor building. These proposed applicability requirements are consistent with the applicability guidelines of the NRC's General Electric Standard Technical Specifications (GE-STTS) for similar applications and with existing NMP-1 TS restrictions associated with reactor building (secondary containment) integrity. The NRC staff finds that the

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proposed applicability requirements are appropriate for NMP-1 and are, therefore, acceptable.

The proposed allowable outage time and action statement requirements would require restoration of reactor building (secondary containment) integrity within 4 hours of loss of integrity or the reactor shall be in at least the hot shutdown condition within the next 12 hours and in the cold shutdown condition within the following 24 hours. This allowable outage time and action statement requirements are consistent with similar provisions and requirements in the GE-STs and are considered reasonable and appropriate by the NRC staff. Therefore, we find the proposed allowable outage time and action statement requirements acceptable.

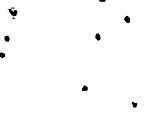
The existing surveillance requirements for TS 3.4.3/4.4.3 would be expanded to include periodic (once per 31 days) verifications that the access doors to the secondary containment are closed. The proposed surveillance requirements are consistent with the guidance provided by the GE-STs and are considered reasonable and appropriate by the NRC staff. Therefore, we find the proposed surveillance requirements acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 22264). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.



5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:  
Donald S. Brinkman

Date: June 29, 1992





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June 29, 1992

Docket No. 50-220

Mr. B. Ralph Sylvia  
Executive Vice President, Nuclear  
Niagara Mohawk Power Corporation  
301 Plainfield Road  
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT NO. 1  
(TAC NO. M83282)

The Commission has issued the enclosed Amendment No. 129 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated April 24, 1992.

The amendment revises Technical Specification 3.4.3/4.4.3 (Access Control) to: (a) define the operating conditions under which the specification applies, (b) include an allowable outage time for continued plant operation while restoration of secondary containment integrity is underway, (c) provide action statements associated with the loss of secondary containment due to access control, and (d) provide periodic surveillance requirements for access doors other than the core spray and containment spray pump compartments.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original Signed By  
Donald S. Brinkman, Senior Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.129 to DPR-63
2. Safety Evaluation

cc w/enclosures:  
See next page

OFFICE	LA:PDI-1	PM:PDI-1 <i>DAB</i>	NRR/SPLB	OGC	D:PDI-1 <i>DAB</i>
NAME	CSVogan <i>CV</i>	DBrinkman:pc	<i>CMcCracken</i>	<i>BmB</i>	RACpara <i>for</i>
DATE	6/11/92	6/11/92	6/17/92	6/22/92	6/29/92

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