



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 33
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated February 19, 1991, as supplemented on March 8, 1991, and May 20, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

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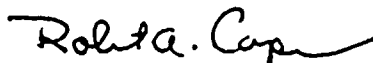
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(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 33 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 15, 1991



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ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 33 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

Remove Pages

2-3
3/4 3-17

Insert Pages

2-3
3/4 3-17
3/4 3-19a (added page)



TABLE 2.2.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

NINE MILE POINT - UNIT 2

2-3

Amendment No. 33

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. Intermediate Range Monitor, - Neutron Flux - High	<120/125 divisions of full scale	<122/125 divisions of full scale
2. Average Power Range Monitor:		
a. Neutron Flux - Upscale, Setdown	<15% of RATED THERMAL POWER	<20% of RATED THERMAL POWER
b. Flow-Biased Simulated Thermal Power - Upscale		
1) Flow-Biased	<0.66 (W-ΔW) ^(a) + 51%, with a	<0.66 (W-ΔW) ^(a) + 54%, with
2) High-Flow-Clamped	maximum of <113.5% of RATED THERMAL POWER	maximum of <115.5% of RATED THERMAL POWER
c. Fixed Neutron Flux - Upscale	<118% of RATED THERMAL POWER	<120% of RATED THERMAL POWER
d. Inoperative	NA	NA
3. Reactor Vessel Steam Dome Pressure - High	<1037 psig	<1057 psig
4. Reactor Vessel Water Level - Low, Level 3	>159.3 in. above instrument zero*	>157.8 in. above instrument zero
5. Main Steam Line Isolation Valve - Closure	<8% closed	<12% closed
6. Main Steam Line Radiation ^(b) - High	<3.0 x full-power background	<3.6 x full-power background
7. Drywell Pressure - High	<1.68 psig	<1.88 psig

* See Bases Figure B3/4 3-1.

(a) The Average Power Range Monitor Scram Function varies as a function of recirculation loop drive flow (W). ΔW is defined as the difference in indicated drive flow (in percent of drive flow which produces rated core flow) between two loop and single loop operation at the same core flow. ΔW=0 for two loop operation. ΔW=5% for single loop operation.

(b) See footnote (***) to Table 3.3.2-2 for trip setpoint during hydrogen addition test.



TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>Primary Containment Isolation Signals (Continued)</u>		
a. Reactor Vessel Water Level*		
1) Low, Low, Low, Level 1	>17.8 in.	>10.8 in.
2) Low, Low, Level 2	>108.8 in.	>101.8 in.
3) Low, Level 3	>159.3 in.	>157.8 in.
b. Drywell Pressure - High	<1.68 psig	<1.88 psig
c. Main Steam Line		
1) Radiation - High**	<3x Full Power Background	<3.6x Full Power Background
2) Pressure - Low	>766 psig	>746 psig
3) Flow - High	<103 psid	<109.5 psid
d. Main Steam Line Tunnel		
1) Temperature - High	<165.7°F	<169.9°F
2) ΔTemperature - High	<66.7°F	<71.3°F
3) Temperature - High MSL Lead Enclosure	<146.7°F	<150.9°F
e. Condenser Vacuum Low	>8.5 in Hg vacuum	>7.6 in. Hg vacuum
f. RHR Equipment Area Temperature - High (HXs/A&B Pump Rooms)	<135°F	<144.5°F
g. Reactor Vessel Pressure - High (RHR Cut-in Permissive)	<128 psig	<148 psig
h. SGTS Exhaust - High Radiation	<5.7x10 ⁻³ μCi/cc	<1.0x10 ⁻² μCi/cc



Table 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

- ** Within 24 hours prior to the planned start of the hydrogen injection test and with the reactor power at greater than 20% rated power, the normal full-power radiation background level and associated trip and alarm setpoints may be changed based on a calculated value of the radiation level expected during the test. The background radiation level and associated trip and alarm setpoints may be adjusted during the test program based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and associated trip and alarm setpoints shall be reset within 24 hours after completion of the hydrogen injection test. At reactor power levels below 20% rated power hydrogen injection shall be terminated, and control rod withdrawal is prohibited until the Main Steam Line Radiation Monitor trip setpoint is restored to its pre-test value.

