UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-410

NINE MILE POINT NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 18
License No. NPF-69

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated April 27, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-69 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 18 are hereby incorporated into this license. Niagara Mohawk Power Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 10, 1990

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 18 TO FACILITY OPERATING LICENSE NO. NPF-69

DOCKET NO. 50-410

Revise Appendix A as follows:

| Remove Pages | Insert Pages |
|---|---|
| iv | iv |
| - | 3/4 0-2 |
| 3/4 0-4 | 3/4 0-4 |

BASES FOR SECTION 2.0

4.0.1 Surveillance Requirements shall be met during the OPERATIONAL CONDITIONS or other conditions specified for individual Limiting Conditions for Operation unless otherwise stated in an individual Surveillance Requirement.

4.0.2 Each Surveillance Requirement shall be performed within the specified time interval with:

a. A maximum allowable extension not to exceed 25% of the surveillance interval, but

b. The combined time interval for any three consecutive surveillance intervals shall not exceed 3.25 times the specified surveillance interval.

c. For the purposes of the first refueling outage, those Surveillance Requirements listed on Tables 4.0.2-1 and 4.0.2-2 are exempted from the provisions of a and b above and their surveillance intervals are extended to the date specified in the table.

4.0.3 Failure to perform a Surveillance Requirement within the specified time interval shall constitute a failure to meet the OPERABILITY requirements for a Limiting Condition for Operation. Exceptions to these requirements are stated in the individual specifications. Surveillance requirements do not have to be performed on inoperable equipment.

4.0.4 Entry into an OPERATIONAL CONDITION or other specified applicable condition shall not be made unless the Surveillance Requirement(s) associated with the Limiting Condition for Operation have been performed within the applicable surveillance interval or as otherwise specified.

4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2, and 3 components shall be applicable as follows:

a. Inservice inspection of ASME Code Class 1, 2, and 3 components and inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10CFR50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10CFR50.55a(g)(6)(i).

b. Surveillance intervals specified in Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda for the inservice inspection and testing activities required by the ASME Boiler and Pressure Vessel Code and applicable addenda shall be applicable as follows in these Technical Specifications:

TABLE 4.0.2-1

SURVEILLANCE TEST INTERVALS EXTENDED TO SEPTEMBER 8, 1990

| NO. | SURVEILLANCE REQUIREMENT | DESCRIPTION |
|---|---|---|
| 1 | Table 4.3.1.1-1, Item 4 | RPV Water Low Level 3 Cal |
| 2 | 4.3.1.2 | RPS LSFT (RPV Low Level 3) |
| 3 | 4.3.1.3 | APRM Response Time Testing |
| 4 | Table 4.3.2.1-1, Item 1.a.3 | RPV Water Low Level 3 Cal |
| 5 | 4.3.2.2 | RPV Water Low Level 3 LSFT |
| 6 | Table 4.3.7.5-1, Item 1 | RPV Pressure Cal (RG 1.97) |
| 7 | Table 4.3.7.5-1, Item 16 | CIV Position Indication |
| 8 | 4.4.3.2.2.a | RCS Isolation Valve Leak Test |
| 9 | 4.5.1.e.2.c | ADS Accumulator LP Alarm Setpt |
| 10 | 4.6.1.2.f | LLRT's-Bypass Leakage Valves |
| 11 | 4.6.4.b.3.a | Vacuum Breaker Open Setpoint |
| 12 | 4.6.4.b.3.b | Vac Breaker Position Indicator |

TABLE 4.0.2-2

SURVEILLANCE TEST INTERVALS EXTENDED TO SEPTEMBER 30, 1990

| NO. | SURVEILLANCE REQUIREMENT | DESCRIPTION |
|---|---|---|
| 1 | Table 4.3.3.1-1, Item B.1.d | LPCI B Time Delay Relay Cal |
| 2 | Table 4.3.3.1-1, Item B.1.e | LPCI C Time Delay Relay Cal |
| 3 | Table 4.3.3.1-1, Item B.1.f | LPCI B Time Delay Relay Cal |
| 4 | Table 4.3.3.1-1, Item B.1.g | LPCI C Time Delay Relay Cal |
| 5 | Table 4.3.3.1-1, Item C.1.a | RPV Low Level 2 Cal (HPCS) |
| 6 | Table 4.3.3.1-1, Item C.1.c | RPV High Level 8 Cal (HPCS) |
| 7 | Table 4.3.3.1-1, Item D.1 | Div II Undervoltage (Loop) Cal |
| 8 | Table 4.3.3.1-1, Item D.2 | Div II Undervoltage (Degraded) |
| 9 | 4.3.3.2 | ECCS LSFT (HPCS & Undervoltage) |
| 10 | 4.3.3.3 | ECCS HPCS Response Time Test |
| 11 | 4.8.2.1.d | Battery Service Test (Div I/II) |