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ACCESSION NBR: 9007090214 DOC. DATE: 90/06/28 NOTARIZED: NO DOCKET #
 FACIL: 50-410 Nine Mile Point Nuclear Station, Unit 2, Niagara Moha 05000410
 AUTH. NAME AUTHOR AFFILIATION
 COLOMB, M.J. Niagara Mohawk Power Corp.
 WILLIS, J.L. Niagara Mohawk Power Corp.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 90-012-00: on 900529, violation of Tech Specs due to personnel error.
 W/9 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: S
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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INTERNAL:	ACNW	2 2	ACRS	2 2
	AEOD/DOA	1 1	AEOD/DSP/TPAB	1 1
	AEOD/ROAB/DSP	2 2	NRR/DET/ECMB 9H	1 1
	NRR/DET/EMEB9H3	1 1	NRR/DLPQ/LHFB11	1 1
	NRR/DLPQ/LPEB10	1 1	NRR/DOEA/OEAB11	1 1
	NRR/DREP/PRPB11	2 2	NRR/DST/SELB 8D	1 1
	NRR/DST/SICB 7E	1 1	NRR/DST/SPLB8D1	1 1
	NRR/DST/SRXB 8E	1 1	REG FILE 02	1 1
	RES/DSIR/EIB	1 1	RGN1 FILE 01	1 1
EXTERNAL:	EG&G BRYCE, J.H	3 3	EG&G STUART, V.A	4 4
	L ST LOBBY WARD	1 1	LPDR	1 1
	NRC PDR	1 1	NSIC MAYS, G	1 1
	NSIC. MURPHY, G.A	1 1	NUDOCS FULL TXT	1 1

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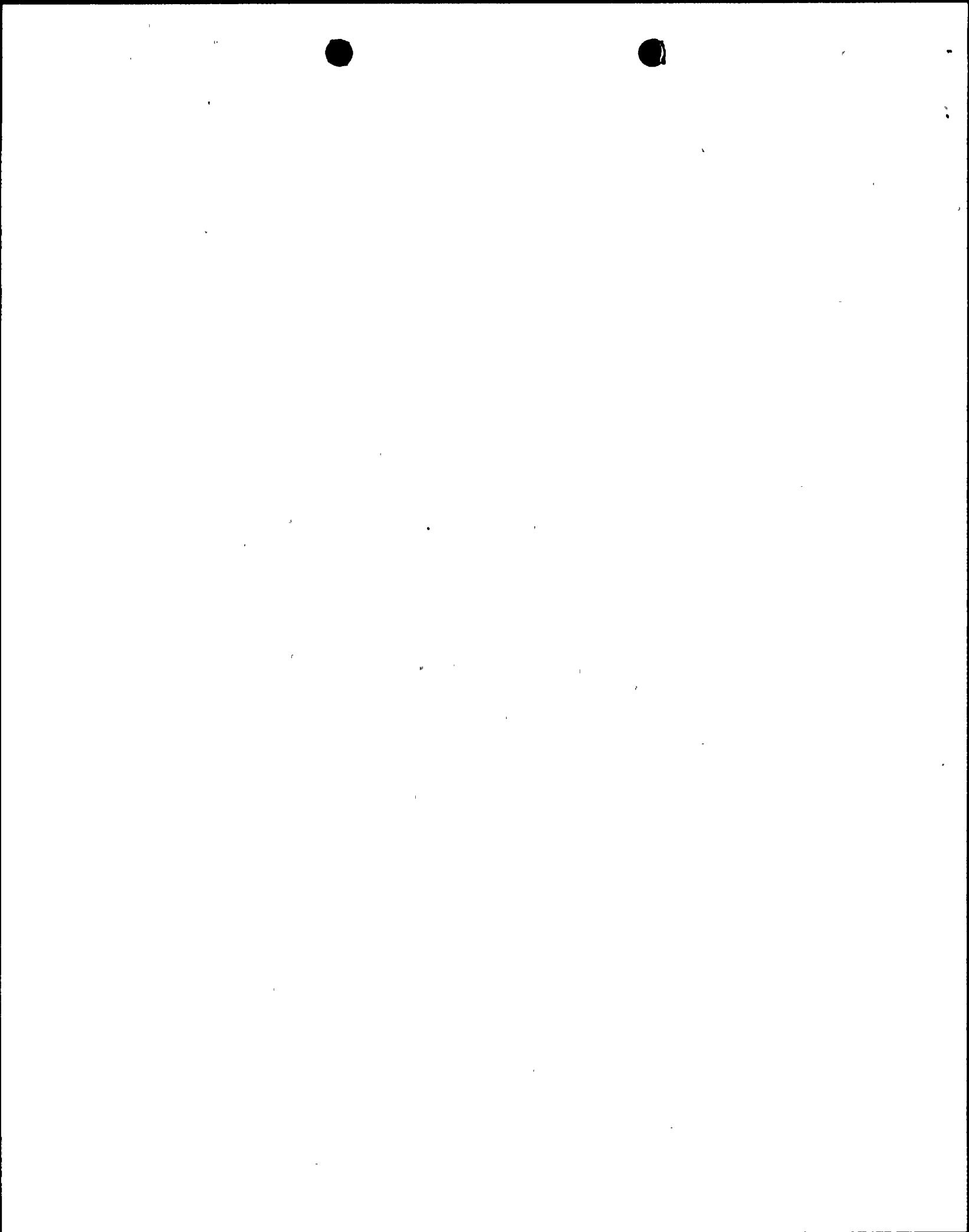
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**NIAGARA
MOHAWK**

NINE MILE POINT NUCLEAR STATION/P.O. BOX 32, LYCOMING, N.Y. 13093/TELEPHONE (315) 343-2110

NMP67594

June 28 , 1990

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

RE: Docket No. 50-410
LER 90-12

Gentlemen:

In accordance with 10CFR50.73, we hereby submit the following Licensee Event Report:

LER 90-12 Is being submitted in accordance with 10CFR50.73 (a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications".

This report was completed in the format designated in NUREG 1022, Supplement 2, dated September 1985.

Very truly yours,



J. L. Willis
General Superintendent
Nuclear Generation

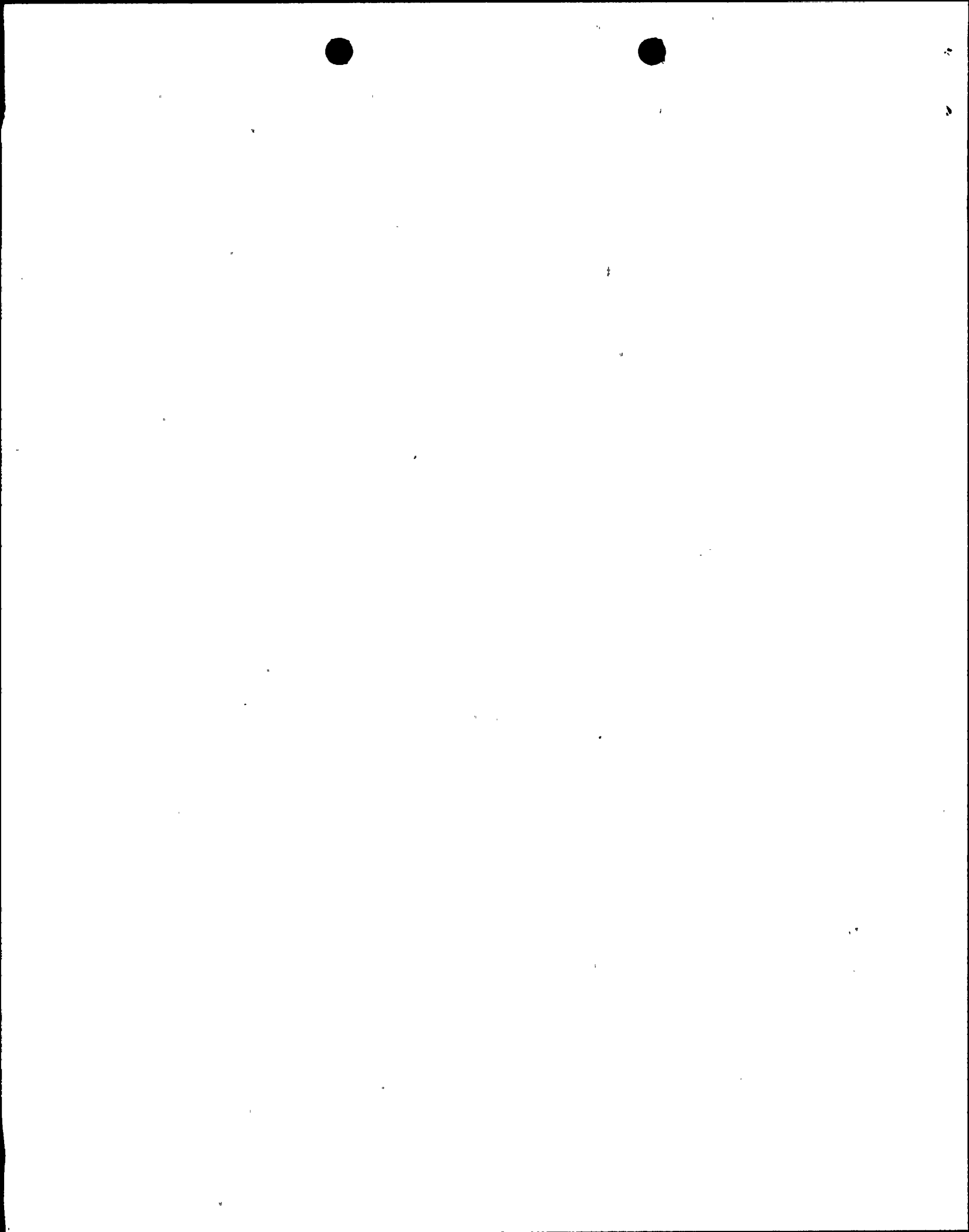
JLW/AC/lmc

ATTACHMENT

cc: Regional Administrator, Region I
Sr. Resident Inspector, W. A. Cook

9007090214 900628
PDR ADDCK 05000410
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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Nine Mile Point Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 4 1 1 0	PAGE (3) 1 OF 0 4
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TITLE (4)
Violation of Technical Specifications Due To Personnel Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 5	2 9	9 0	9 0	0 1 2	0 0	0 6	2 8	9 0	N/A	0 5 0 0 0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)										

OPERATING MODE (9) 1	POWER LEVEL (10) 0 3 1	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
		<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.38(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
		<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.38(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 365A)
		<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
		<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
		<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Michael J. Colomb, Operations Superintendent Unit 2	TELEPHONE NUMBER AREA CODE: 3 1 5 3 4 9 - 7 9 5 2
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On May 29, 1990, at 1700 hours, Nine Mile Point Unit 2 (NMP2) experienced a Technical Specification Violation for failure to complete Technical Specification required action for a control rod with an inoperable control rod position indication. At the time of the event, the mode switch was in "RUN" and the reactor was at 31% rated thermal power.

The root cause for this event was determined to be personnel error.

The corrective actions taken for this event included:

The affected control rod was restored to operable status; the licensed Reactor Operator and Reactor Analyst Supervisor involved completed remedial training on control rod operability, verbal communications and personnel responsibilities during control rod movement; prior to resuming power ascension, the Operations Superintendent and Assistant Operations Superintendent reviewed the event and corrective actions with all of the operating shifts; a Lessons Learned Transmittal (LLT) was developed; and Operating Procedure N2-OP-95A, "Rod Worth Minimizer", Section H.1.0 was revised.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Nine Mile Point Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 4 1 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 0	- 0 1 2	- 0 0	0 2	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. DESCRIPTION OF EVENT

On May 29, 1990, at 1700 hours, Nine Mile Point Unit 2 (NMP2) discovered a Technical Specification Violation for failure to complete a Technical Specification required action (3.1.3.7 Action 2) for a control rod with an inoperable control rod position indication. At the time of the event, the mode switch was in "RUN" and the reactor was at 31% rated thermal power.

Beginning at approximately 0800 hours on the day of the event, reactor power was being raised from 24% to 35% rated thermal power via control rod withdrawal. The rod worth minimizer (RWM) was inoperable (bypassed) and the Reactor Analyst Supervisor was performing a control rod sequence verification.

During the course of control rod withdrawal, the licensed Reactor Operator at the controls withdrew control rod 26-55 to the prescribed position 36. The 4-rod display on CEC-PNL603 did not properly indicate (i.e. was blank) for this position. The Reactor Operator observed this lack of position indication and then made a general statement concerning the lack of proper position indication to the people behind him. He then resumed control rod withdrawal for the power ascension.

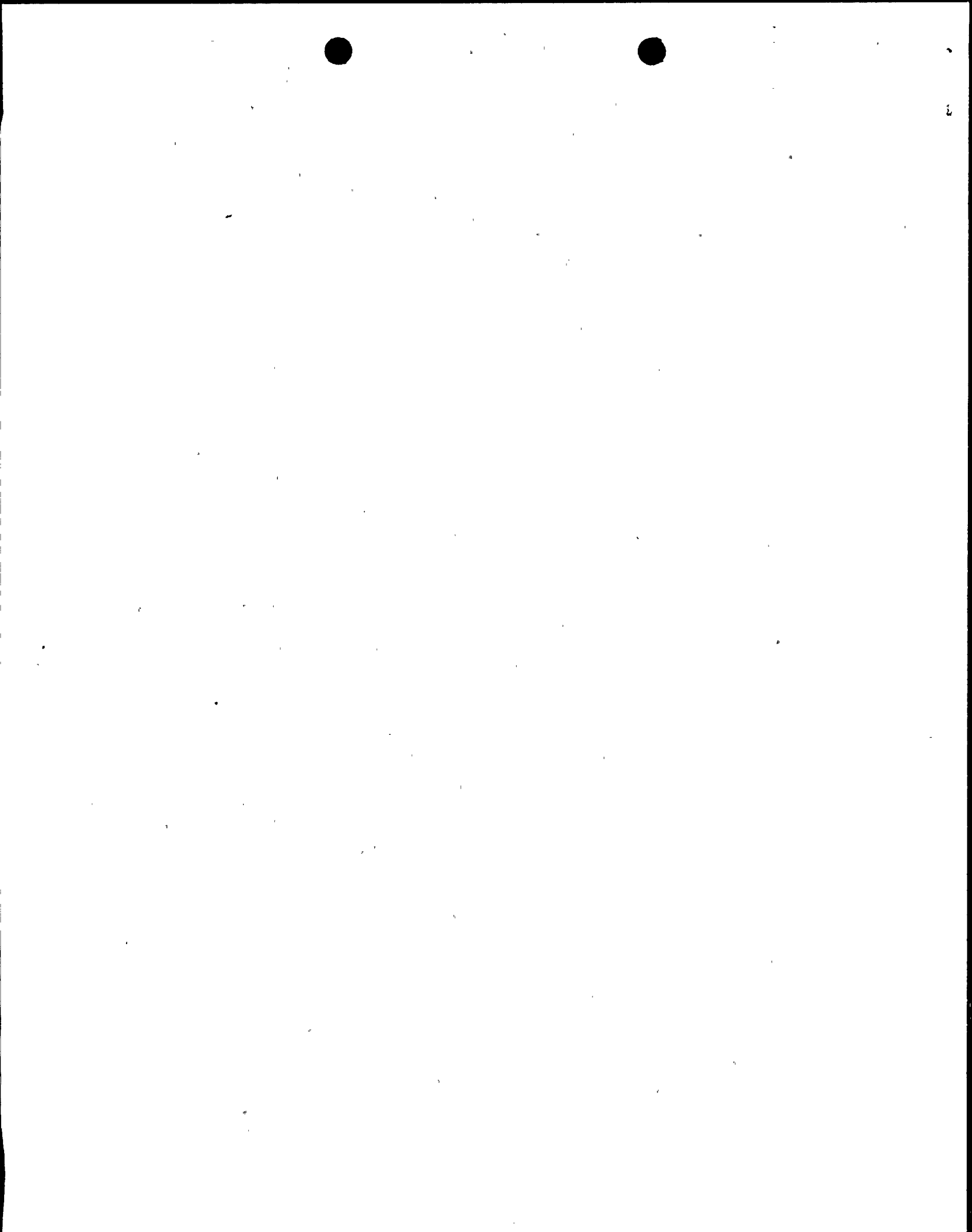
The subsequent shift discovered the inoperable control rod position soon after assuming shift duties. The Station Shift Supervisor (SSS) directed that the required Technical Specification action (3.1.3.7 Action 2) be performed and control rod 26-55 was inserted to the operable position indicator 34. This action restored the control rod to an operable status. The elapsed time the control rod was considered inoperable was approximately 7 hours.

II. CAUSE OF EVENT

The cause of this event was personnel error due to failure on the part of the licensed Reactor Operator to recognize that immediate action (within 1 hour) was required to maintain operability of control rod 26-55.

Contributing factors to the cause of the event were:

1. The Reactor Operator did not directly communicate the inoperable position indication condition to the Senior Reactor Operator (SRO) in charge of plant startup activities.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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		YEAR 9 0	SEQUENTIAL NUMBER 0 1 2	REVISION NUMBER 0 0	0 3	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

2. The Reactor Operator resumed control rod withdrawal without ensuring his communication, on the inoperable control rod position indication, was received and acknowledged by the SRO, and without having resolution on this abnormal condition.
3. The Reactor Analyst Supervisor performing the control rod sequence verification function did not ensure that the control rod was withdrawn to its prescribed position, as was required by N2-OP-95A, "Rod Worth Minimizer", Section H.1.0.
4. The Reactor Analyst Supervisor did not verify the actual control rod pattern conformed to the Rod Pull Sheets upon completion of withdrawal of each control rod group, as was required by N2-OP-95A, "Rod Worth Minimizer", Section H.1.0.

III. ANALYSIS OF EVENT

This event is reportable in accordance with 10CFR50.73(a)(2)(i) because operation with a control rod positioned such that the indicated position is inoperable is prohibited by the plant's Technical Specifications.

The control rod was moved to a position with operable indication and was capable of performing its intended function through the whole event, therefore, there were no safety consequences to the plant or public as a result of this event.

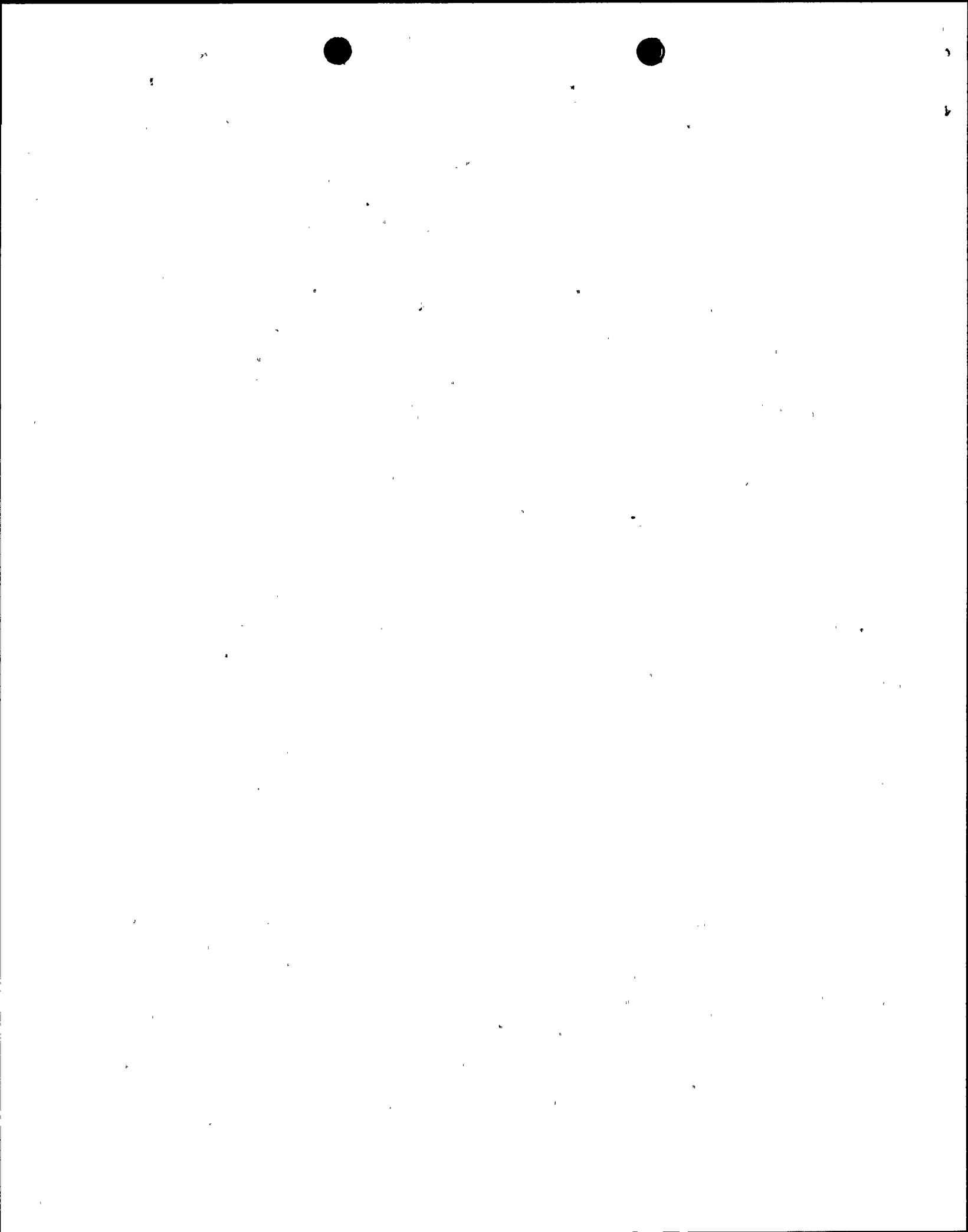
Had this event occurred at 100% power, the severity would not have increased.

IV. CORRECTIVE ACTIONS

The immediate corrective action taken was to insert control rod 26-55 to an operable position indication (Notch 34). This action restored control rod 26-55 to operable status. A Work Request (WR 176717) was initiated to repair the inoperable control rod position indicator.

Further corrective actions included:

1. The licensed Reactor Operator and Reactor Analyst Supervisor completed remedial training on Technical Specification requirements for control rod operability, verbal communications and personnel responsibilities during control rod movement.



LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Nine Mile Point Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 4 1 0 9 0	LER NUMBER (6)			PAGE (3)	
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		9 0	- 0 1 2	- 0 0	0 4	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

2. A requirement for the SSS to oversee all control movements (until control rod pull sheets with specific signoffs are developed) was put in place. This event and these corrective actions were discussed by the Superintendent Operations and Assistant Superintendent Operations with each operating shift prior to that shift being allowed to withdraw control rods for power ascension.
3. A Lessons Learned Transmittal emphasizing communications, control rod operability, notification of abnormal conditions and personnel responsibilities during control rod movement was developed.
4. Operating Procedure N2-OP-95A, "Rod Worth Minimizer", Section H.1.0, was revised to provide additional requirements for the person performing the verification function required by Technical Specifications when the Rod Worth Minimizer is not operable.
5. The Operations Superintendent and Assistant Operations Superintendent reviewed the event, control rod Technical Specifications, and Lessons Learned with each operating shift prior to that shift being allowed to withdraw control rods for power ascension.

V. ADDITIONAL INFORMATION

- A. Previous similar events--none.
- B. Failed components--none.
- C. Identification of components referred to in this LER:

COMPONENT	IEEE 803 EIIIS FUNCT.	IEEE 805 SYSTEM ID
Control Rod	ROD	AA
Control Rod Position Indication	ZI	AA
Rod Worth Minimizer	N/A	N/A



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