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SUBJECT: Provides results of visual exam of drain lines,per 870513 response to GL 87-05.

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June 26, 1990
NMP1L 0510

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Gentlemen:

Your Generic Letter 87-05, dated March 12, 1987, requested information from Boiling Water Reactor Mark I Licensees regarding measures to identify and/or mitigate potential degradation of Mark I drywells. In our response of May 13, 1987 (NMP1L 0152) we committed to the following future actions:

- Visual examination of the drain lines in the sand cushion area will be performed using remote video equipment during the next refueling outage scheduled to begin in March, 1988.
- To assure that the reactor head cavity seal bypass leakage drain is properly functioning, a visual inspection of the drain line will be made using remote video equipment during the Spring 1988 refueling outage. In addition, drain line flow switch FS 89-01 will be checked for proper operation.
- After the initial inspections described above are performed, Niagara Mohawk will re-evaluate its position regarding the performance of additional inspections.

This letter provides the results of these inspections and our position regarding future additional inspections.

The results of the remote visual examination of each of the ten lines in the sand cushion area indicate that each drain line is open to the gap between the sand cushion and the drywell shell and each would pass drainage water to the torus room. The sand cushion area drain line interiors were not heavily corroded and there was no evidence of drainage from the air gap to the torus room. During the inspection of the drain lines, the drywell shell surface adjacent to the two sand cushion area drain lines was also inspected. The drywell shell surface viewed did not exhibit signs of heavy corrosion or water staining.

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A remote visual inspection of the reactor head cavity seal leakage drain area was conducted including the entire circumference of the concrete shelf and shelf drain scupper. In addition, remote visual inspection of water leakage from the reactor cavity wall into the concrete shelf drain was conducted while an approximate three gallon-per-minute leak existed. The results of these inspections show that the reactor head cavity seal bypass leakage drain line functions properly. There was no evidence of leakage from the concrete shelf beneath the reactor head cavity into the air gap between the drywell shell and the surrounding concrete.

Leakage water from the reactor head cavity flowed into the concrete shelf drain via the drain scupper. A check of the reactor head cavity drain line flow switch (FS 89-01) revealed that the existing switch was not adequately sensitive for the low flows associated with this line. Niagara Mohawk is currently modifying the drain line flow switch to provide a more reliable system for detecting and monitoring leakage. This modification is scheduled for completion during the next refueling outage.

Based on the results of our inspections, additional periodic examination of the sand cushion area and reactor head cavity drain lines is not warranted. All drains were found to be operable, and factors that could cause a change in their condition are not present. In addition, examination for evidence of moisture in the air gap between the outer surface of the drywell and surrounding concrete indicate that mechanisms which could cause aggravated corrosive attack on the drywell wall are not present.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION

C. D. Terry
Vice President
Nuclear Engineering and Licensing

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