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February 26, 1993 NMP1L 0739

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555

Re: Nine Mile Point Unit 1

Docket No. 50-220 DPR-63

TAC No. M83486

SUBJECT: GENERIC LETTER 92-01, REVISION 1, REACTOR VESSEL

STRUCTURAL INTEGRITY, ELASTIC-PLASTIC FRACTURE MECHANICS ASSESSMENT FOR SERVICE LEVEL C AND D

LOADINGS

Gentlemen:

By letter dated August 12, 1992, the Staff recommended that Niagara Mohawk perform an analysis to demonstrate that two beltline plates with Charpy upper shelf energies (USE) less than 50 ft-lbs have margins of safety against fracture equivalent to those required by Appendix G of the ASME Code. Our letter of October 16, 1992 (NMP1L 0707), responded to the Staff's request and provided an analysis of the two beltline plates for Service Level A and B loadings. A revised analysis for the Level A and B loadings was submitted on December 17, 1992 (NMP1L 0723), in response to the Staff's request of November 13, 1992. Our letter of December 17, 1992, indicated that a separate report addressing Service Level C and D loading calculations would be submitted by March 1, 1993.

Enclosed is a copy of our report entitled, "Elastic-Plastic Fracture Mechanics Assessment of Nine Mile Point Unit 1 Beltline Plates for Service Level C and D Loadings." As we had anticipated, the Service Level A and B loadings (and not the Level C and D) are limiting in terms of ductile fracture. Based on the calculations previously submitted for Service Level A and B loadings and those contained herein, Niagara Mohawk has concluded that NMP1 plate G-8-1 is limiting from a ductile fracture perspective, and the USE must be maintained above 23 ft-lbs.

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In addition, our letter of December 17, 1992, indicated that a separate report on USE estimates for the beltline welds would be submitted by March 1, 1993. However, due to the Staff's recent request to provide a determination of the beltline plate orientation, the report on USE estimates for beltline welds will now be submitted by March 22, 1993. The Staff has agreed that this date is acceptable.

Very truly yours,

C. D. Terry Vice President Nuclear Engineering

AER/mls 003611GG Enclosure

xc: Regional Administrator, Region I

Mr. R. A. Capra, Director, Project Directorate I-1, NRR

Mr. D. S. Brinkman, Senior Project Manager, NRR

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