



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 139 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

DOCKET NO. 50-220

1.0 INTRODUCTION

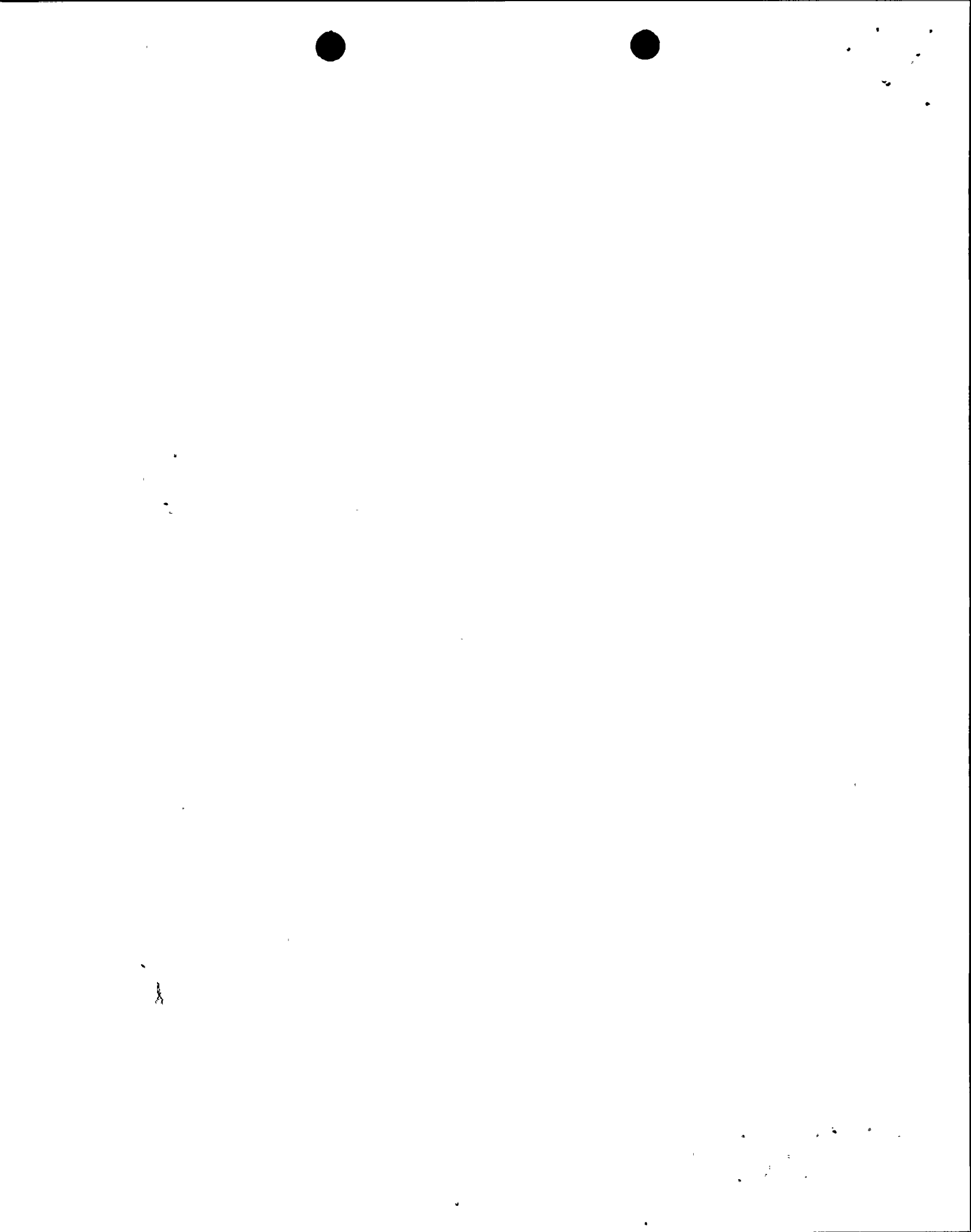
By letter dated December 4, 1992, as supplemented February 12, 1993, and February 17, 1993, Niagara Mohawk Power Corporation (the licensee or NMPC) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1 (NMP1), Technical Specifications (TS). The requested changes would revise TS 3.6.2, 4.6.2, 3.6.11, and 4.6.11 and associated Bases to increase the surveillance test intervals and add allowable out-of-service times for various instruments. The licensee stated in its request that the requested changes are consistent with the NRC staff's previous approvals of several General Electric Company (GE) Licensing Topical Reports (LTRs). The licensee's submittal also stated that the proposed out-of-service times are consistent with the guidance provided in NUREG-1433, "Standard Technical Specifications, General Electric Plants, BWR/4." The proposed changes would permit specified instrument channel functional tests to be performed once per 3 months rather than once per week or once per month. The proposed changes would include editorial corrections on TS pages 192, 199, and 210. The February 12, 1993, letter clarified the actions to be taken in the event of a loss of more than one instrument channel in a trip system so that a loss of function will not occur. The February 17, 1993, letter withdrew the original plant specific proprietary reports and affidavits requesting withholding of proprietary information. The February 17, 1993, letter also resubmitted the plant specific proprietary reports and affidavits to more specifically delineate General Electric's proprietary information. Neither the February 12, 1993, letter nor the February 17, 1993, letter change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The licensee has proposed changes to TS 3.6.2 and 4.6.2 (Protective Instrumentation) and TS 3.6.11 and 4.6.11 (Accident Monitoring Instrumentation). The proposed changes are based on the NRC staff's previous approvals of the following GE LTRs:

1. NEDC-30851P-A, "Technical Specification Improvement Analyses for BWR Reactor Protection System," dated March 1988. This LTR was approved by

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letter and enclosed safety evaluation dated July 15, 1987, from A. C. Thadani (NRC) to T. A. Pickens (BWR Owners Group).

2. NEDC-30851P-A (Supplement 2), "Technical Specification Improvement Analysis for BWR Isolation Instrumentation Common to RPS and ECCS Instrumentation," dated March 1989. This LTR was approved by letter and enclosed safety evaluation dated January 6, 1989, from C. E. Rossi (NRC) to D. N. Grace (BWR Owners Group).
3. NEDC-31677P-A, "Technical Specification Improvement Analysis for BWR Isolation Actuation Instrumentation," dated July 1990. This LTR was approved by letter and enclosed safety evaluation dated June 18, 1990, from C. E. Rossi (NRC) to S. D. Floyd (BWR Owners Group).
4. NEDC-30936P-A, "BWR Owners' Group Technical Specification Improvement Methodology (With Demonstration for BWR ECCS Actuation Instrumentation) Part 1," dated December 1988. This LTR was approved by letter and enclosed safety evaluation dated December 9, 1988, from A. C. Thadani (NRC) to D. N. Grace (BWR Owners Group).
5. NEDC-30936P-A, "BWR Owners' Group Technical Specification Improvement Methodology (With Demonstration for BWR ECCS Actuation Instrumentation) Part 2," dated December 1988. This LTR was approved by letter and enclosed safety evaluation dated December 9, 1988, from C. E. Rossi (NRC) to D. N. Grace (BWR Owners Group).
6. NEDC-30851P-A (Supplement 1), "Technical Specification Improvement Analysis for BWR Control Rod Block Instrumentation," dated October 1988. This LTR was approved by letter and enclosed safety evaluation dated September 22, 1988, from C. E. Rossi (NRC) to D. N. Grace (BWR Owners Group).
7. GENE-770-06-1, "Bases for Changes to Surveillance Test Intervals and Allowed Out-of-Service Times for Selected Instrumentation Technical Specifications," dated February 1991. This LTR was approved by letter and enclosed safety evaluation dated July 21, 1992, from C. E. Rossi (NRC) to R. D. Bing IV (BWR Owners Group).

Each of the above LTRs was prepared and approved on a generic basis with requirements for individual licensees to perform plant-specific evaluations to demonstrate that the LTRs are applicable to plant-specific license amendment requests. NMPC has performed the required plant-specific evaluations for NMP1. These evaluations are discussed below:

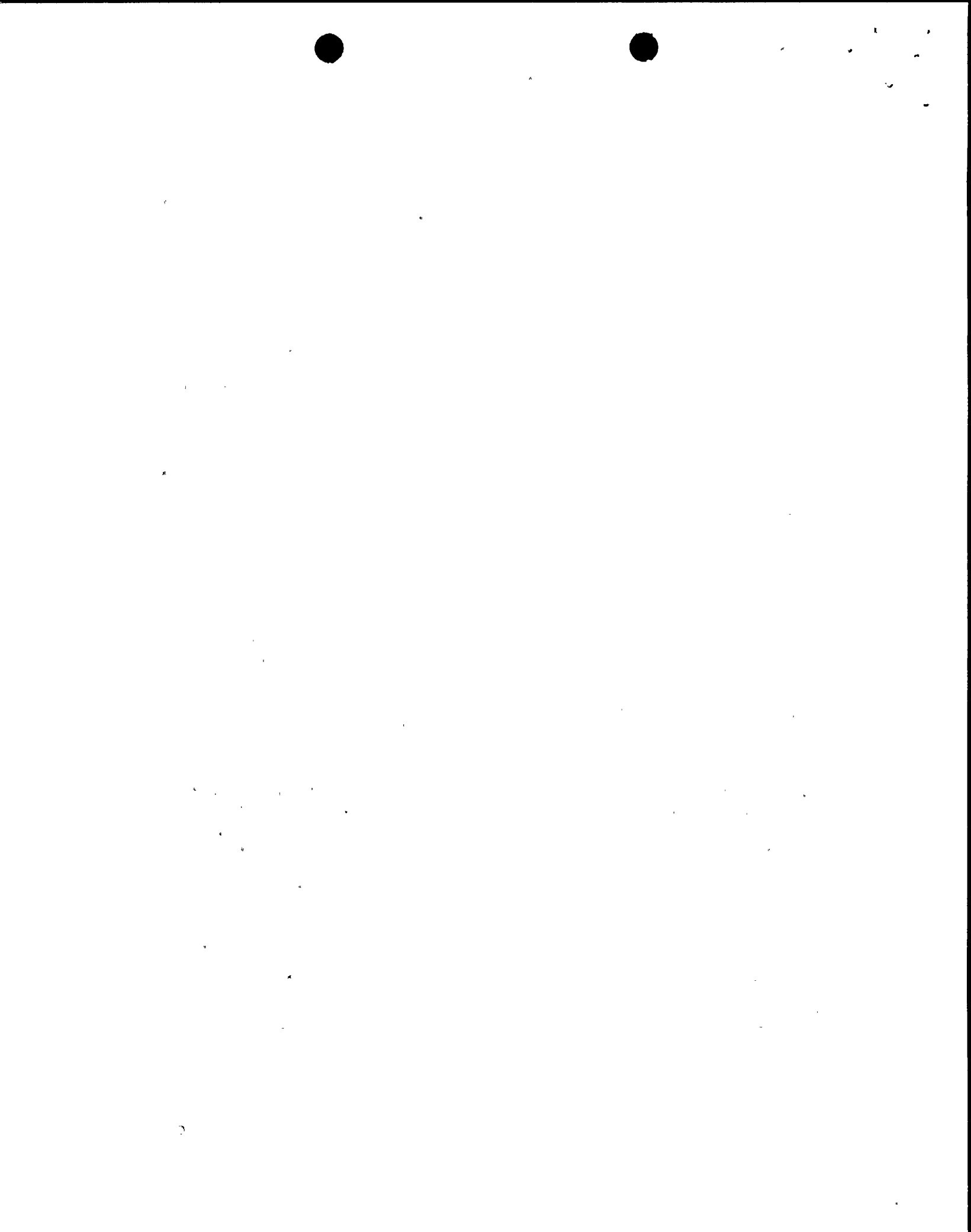
1. Appendix L of NEDC-30851P-A identifies NMP1, a GE BWR-2 product line reactor with a Mark I containment, as a participating utility in the development of this LTR. Section 7.4, "Conclusions of Plant Specific Applications," of NEDC-30851P-A concluded that the generic results of this



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LTR can be applied to NMP1. Furthermore, NMPC's December 4, 1992, submittal included a copy of GE Report MDE-77-0485 DRF A00-02119-D, April 1985 (Proprietary), "Technical Specification Improvement Analysis for the Reactor Protection System for Nine Mile Point Nuclear Station, Unit 1," which concludes in Section 4, "Summary and Conclusions," that the generic analyses in NEDC-30851P-A are applicable to NMP1. Therefore, we have concluded that NEDC-30851P-A is applicable to NMP1.

2. Appendix A of NEDC-30851P-A (Supplement 2) identifies NMP1 as a participating utility in the development of this LTR. Section 3.4, "BWR-2 Plants," specifically analyzes BWR-2 plants. Therefore, we have concluded that NEDC-30851P-A (Supplement 2) is applicable to NMP1.
3. Appendix E of NEDC-31677P-A identifies NMP1 as a participating utility in the development of this LTR. Section 5.4, "BWR-2 Plants," and Appendix C3, "BWR-2 Isolation Actuation Instrumentation Technical Specification Requirements," specifically analyze BWR-2 plants. Therefore, we have concluded that NEDC-31677P-A is applicable to NMP1.
4. Appendix N of Part 1 of NEDC-30936P-A identifies NMP1 as a participating utility in the development of this LTR. NMPC's December 4, 1992, submittal included a copy of GE Report RE-003 DRF A00-02558E, January 1987 (Proprietary), "Technical Specification Improvement Analysis for the Emergency Core Cooling System Actuation Instrumentation for Nine Mile Point Nuclear Station, Unit 1," which concludes in Section 4, "Summary and Conclusions," that the generic analyses in NEDC-30936P-A, Parts 1 and 2, are applicable to NMP1. Therefore, we have concluded that NEDC-30936P-A, Part 1, is applicable to NMP1.
5. Appendix B of Part 2 of NEDC-30936P-A identifies NMP1 as a participating utility in the development of this LTR. Section 5.3, "Plant Specific Analysis Results of BWR-2 Plants," of NEDC-30936P-A, Part 2, specifically analyzes BWR-2 plants. Furthermore, as noted above, GE Report RE-003 also concludes that the generic analyses in NEDC-30936P-A are applicable to NMP1. Therefore, we have concluded that NEDC-30936P-A, Part 2, is applicable to NMP1.
6. Appendix B of NEDC-30851P-A (Supplement 1) identifies NMP1 as a participating utility in the development of this LTR. Section 4, "Plant Technical Specifications," specifically addresses BWR-2 plants. Therefore, we have concluded that NEDC-30851P-A (Supplement 1) is applicable to NMP1.
7. GENE-770-06-1 identifies application of changes to surveillance test intervals and allowable out-of-service times for selected instrumentation. The instrumentation included in GENE-770-06-1 is either the same or similar instrumentation to that analyzed in the above referenced LTRs. The changes in surveillance test intervals and allowable out-of-service



times covered by GENE-770-06-1 are consistent with the guidance provided in NUREG-1433, "Standard Technical Specifications, General Electric Plants, BWR/4." Therefore, we have concluded that GENE-770-06-1 is applicable to NMP1.

Each of the above LTRs also contain requirements for licensees to demonstrate that the drift characteristics for the applicable instrumentation are bounded by the assumptions used in the LTRs when the functional test interval is extended from once per month to once per 3 months. In response to these requirements, the NRC staff noted that the proposed changes in NMP1 surveillance test intervals are applicable only to the Instrument Channel Tests. For NMP1, Instrument Channel Tests are defined in TS Definition 1.6 as: "Instrument channel test means injection of a simulated signal into the channel to verify its proper response including, where applicable, alarm and/or trip initiating action." During our review of the proposed changes, the NRC staff determined that during performance of Instrument Channel Tests at NMP1, the instrument functions, e.g., alarms and/or trips, are verified but NMPC does not verify instrument set points during these tests. Instrument set points are verified during Instrument Channel Calibrations which the current TS require to be performed on a quarterly frequency. The proposed license amendment does not propose any changes in the frequency for performing the Instrument Channel Calibration surveillance tests. Therefore, in accordance with the clarification regarding instrument drift allowances provided in a letter dated April 27, 1988, from C. E. Rossi (NRC) to R. F. Janecek (BWR Owners Group), no further action on this issue is required since the surveillance test intervals for performing Instrument Channel Calibrations are unchanged.

Our evaluations of specific proposed changes are as follows:

Tables 3.6.2a and 4.6.2a - Instrumentation That Initiates Scram

Note (o) of current TS Tables 3.6.2a and 4.6.2a provides that an affected instrument channel may be placed in an inoperable status for up to 2 hours (up to 5 hours for the High Radiation Main-Steam Line Instrument Channel Calibration surveillance) for required surveillance without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter. The proposed change would revise Note (o) to read:

- (o) A channel may be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one Operable Instrument Channel in the same trip system is monitoring that parameter.

With one channel required by Table 3.6.2a inoperable in one or more Parameters, place the inoperable channel and/or that trip system in the tripped condition* within 12 hours.



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With two or more channels required by Table 3.6.2a inoperable in one or more Parameters:

1. Within 1 hour, verify sufficient channels remain Operable or tripped* to maintain trip capability for the Parameter, and
2. Within 6 hours, place the inoperable channel(s) in one trip system and/or that trip system** in the tripped condition*, and
3. Within 12 hours, restore the inoperable channels in the other trip system to an Operable status or tripped*.

Otherwise, take the ACTION required by Specification 3.6.2a for that Parameter.

* An inoperable channel or trip system need not be placed in the tripped condition where this would cause the Trip Function to occur. In these cases, if the inoperable channel is not restored to Operable status within the required time, the ACTION required by Specification 3.6.2a for the parameter shall be taken.

** This ACTION applies to that trip system with the most inoperable channels; if both trip systems have the same number of inoperable channels, the ACTION can be applied to either trip system.

The proposed change would increase the allowable out-of-service time for performing required surveillances for the affected instrument channels (Parameters (2) High Reactor Pressure, (3) High Drywell Pressure, (4) Low Reactor Water Level, (5) High Water Level Scram Discharge Volume, (6) Main-Steam-Line Isolation Valve Position, (9)(a)(i) IRM Upscale, (9)(a)(ii) IRM Inoperative, (9)(b)(i) APRM Upscale, (9)(b)(ii) APRM Inoperative, (9)(b)(iii) APRM Downscale, (10) Turbine Stop Valve Closure, and (11) Generator Load Rejection) from 2 hours to 6 hours. The allowable out-of-service time for Parameter (7) High Radiation Main-Steam-Line would be increased from 5 hours to 6 hours. These proposed changes in allowable out-of-service times are consistent with the provisions of NEDC-30851P-A and are, therefore, acceptable.

The proposed change to Note (o) includes actions to be taken when one or more channels are inoperable. The actions to be taken if channels are inoperable are consistent with current NRC staff positions and with the guidance provided in NUREG-1433, ensure that a loss of function will not exist if two or more channels are inoperable, and are, therefore, acceptable.

The proposed change would correct an error of omission on TS page 192. The word "Upscale" would be added to Parameter (9)(a)(i) in Table 3.6.2a. This change corrects an editorial error of omission and is, therefore, acceptable.



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The proposed changes in Table 4.6.2a would decrease the Instrument Channel Test surveillance frequency of Parameter (1), Manual Scram, from once per 3 months to once per week and would increase the Instrument Channel Test surveillance frequencies of the following Parameters from once per week or once per month to once per 3 months:

- a. High Reactor Pressure - Parameter (2)
- b. High Drywell Pressure - Parameter (3)
- c. Low Reactor Water Level - Parameter (4)
- d. High Water Level Scram Discharge Volume - Parameter (5)
- e. Main-Steam-Line Isolation Valve Position - Parameter (6)
- f. High Radiation Main-Steam-Line - Parameter (7)
- g. APRM Upscale - Parameter (9)(b)(i)
APRM Inoperative - Parameter (9)(b)(ii)
APRM Downscale - Parameter (9)(b)(iii)
- h. Generator Load Rejection - Parameter (11)

These proposed changes in surveillance test frequencies are consistent with NEDC-30851P-A and are, therefore, acceptable.

Tables 3.6.2b and 4.6.2b - Instrumentation That Initiates Primary Coolant System or Containment Isolation

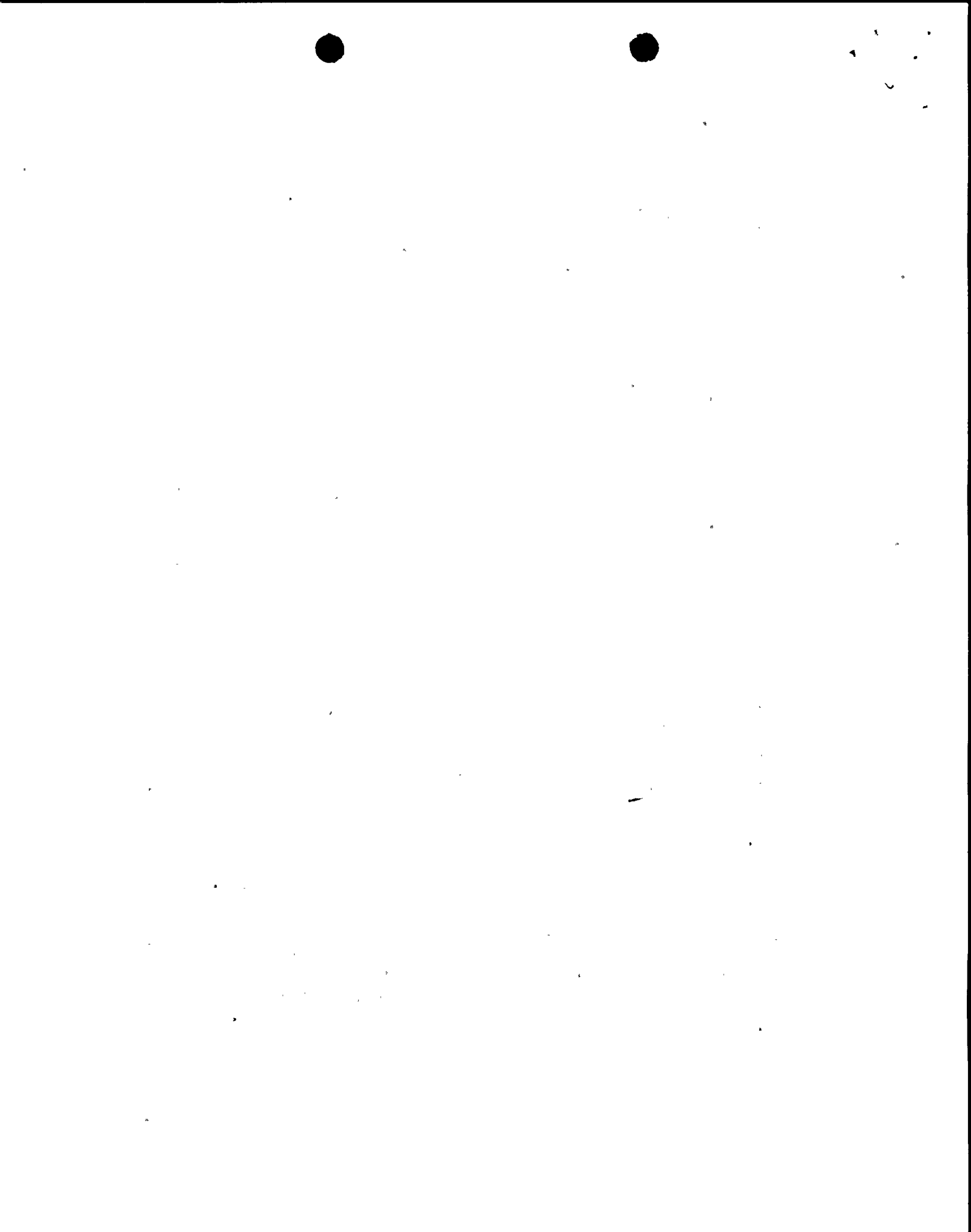
Note (f) of current TS Tables 3.6.2b and 4.6.2b provides that an affected instrument channel may be placed in an inoperable status for up to 2 hours (up to 5 hours for the High Radiation Main-Steam Line Instrument Channel Calibration surveillance) for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter. The proposed change would revise Note (f) and add Note (g) to Tables 3.6.2b and 4.6.2b to read:

- (f) A channel may be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one Operable Instrument Channel in the same Trip System is monitoring that Parameter.

With the number of Operable Channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement for one trip system, either

1. Place the inoperable channel(s) in the tripped condition within
 - a. 12 hours for Parameters common to SCRAM Instrumentation, and
 - b. 24 hours for Parameters not common to SCRAM Instrumentation.

or



2. Take the ACTION required by Specification 3.6.2a for that Parameter.

With the number of Operable Channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement for both trip systems,

1. Place the inoperable channel(s) in one trip system in the tripped condition within 1 hour,

and

2. a. Place the inoperable channel(s) in the remaining trip system in the tripped condition within
 - (1) 12 hours for Parameters common to SCRAM Instrumentation, and
 - (2) 24 hours for Parameters not common to SCRAM Instrumentation.

or

- b. take the ACTION required by Specification 3.6.2a for that Parameter.

- (g) A channel may be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one Operable Instrument Channel in the same Trip System is monitoring that Parameter.

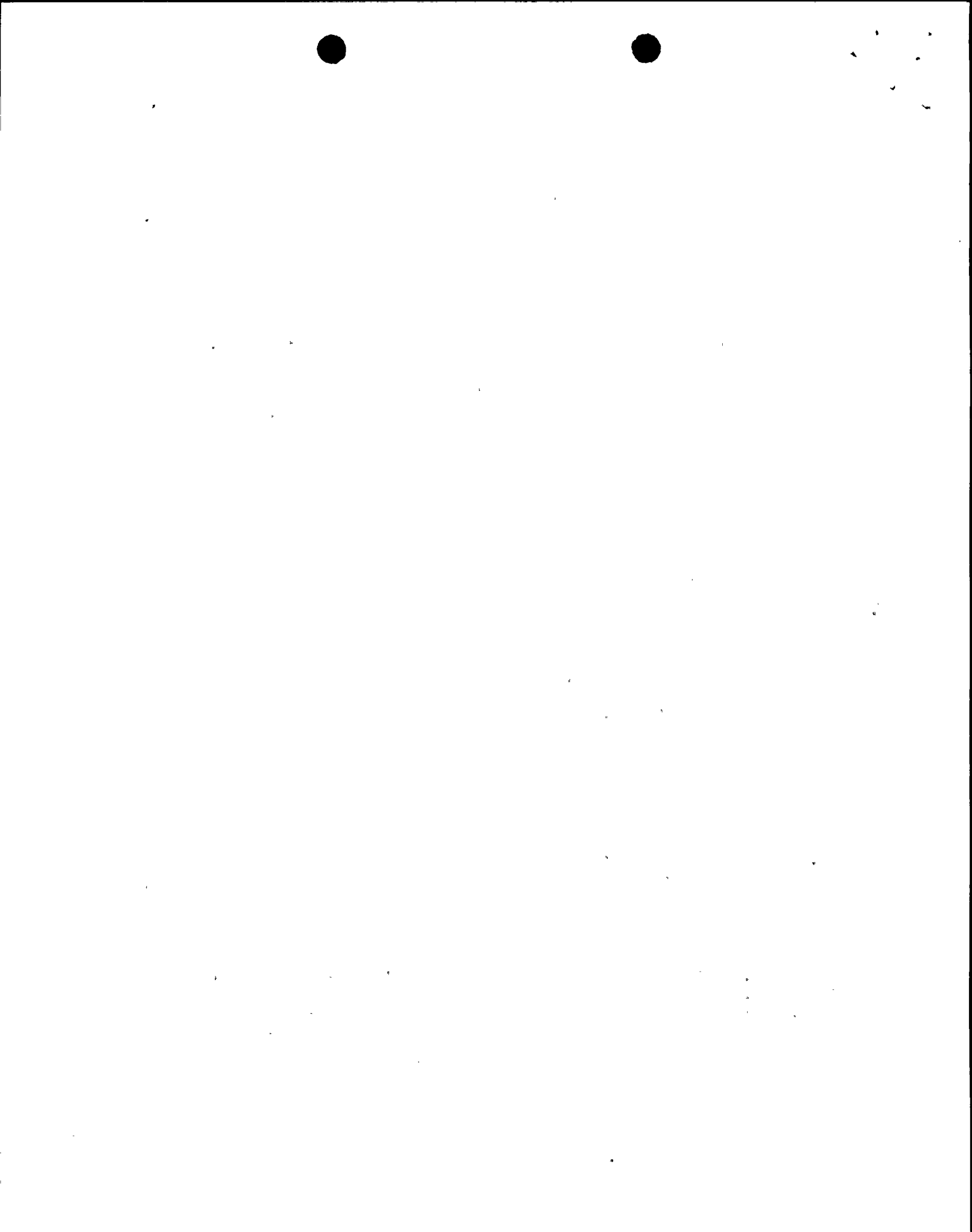
With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels for the Operable Trip System, either

1. Place the inoperable channel(s) in the tripped condition within 24 hours.

or

2. Take the ACTION required by Specification 3.6.2a for that Parameter.

Revised Note (f) and added Note (g) would increase or provide allowable out-of-service times for performing required surveillances for the affected instrument channels (Parameters (1) Low-Low Reactor Water Level, (3) High Steam Flow Main-Steam Line, (5) Low Reactor Pressure, (6) Low-Low-Low Condenser Vacuum, (7) High Temperature Main Steam Line Tunnel, (8) High Area Temperature, (10) Low-Low Reactor Water, and (11) High Drywell Pressure) from 2 hours to 6 hours. The allowable out-of-service time for Parameter (4) High Radiation Main Steam Line would be increased from 5 hours to 6 hours. These



proposed changes in allowable out-of-service times are consistent with the provisions of NEDC-30851P-A (Supplement 2) and NEDC-31677P-A and are, therefore, acceptable.

The proposed change to Note (f) and the addition of Note (g) specify actions to be taken when one or more channels are inoperable. The actions to be taken if channels are inoperable are consistent with current NRC staff positions and with the guidance provided in NUREG-1433, ensure that a loss of function will not exist if two or more channels are inoperable, and are, therefore, acceptable.

The proposed change would make two editorial changes in Table 3.6.2b. The units "°F" would be added in the Set Point column to Parameters (8) and (9) High Area Temperature. These changes are purely editorial in nature, do not change any requirements, and are, therefore, acceptable.

The proposed changes in Table 4.6.2b would increase the Instrument Channel Test surveillance frequencies of the following Parameters from once per week or once per month to once per 3 months:

a. PRIMARY COOLANT ISOLATION

- 1) Low-Low Reactor Water Level - Parameter (1)

b. MAIN-STEAM-LINE ISOLATION

- 1) High Steam Flow Main Steam Line - Parameter (3)
- 2) High Radiation Main Steam Line - Parameter (4)
- 3) Low Reactor Pressure - Parameter (5)

c. CONTAINMENT ISOLATION

- 1) Low-Low Reactor Water Level - Parameter (10)
- 2) High Drywell Pressure - Parameter (11)

These proposed changes in surveillance test frequencies are consistent with NEDC-30851P-A (Supplement 2) and NEDC-31677P-A and are, therefore, acceptable.

Tables 3.6.2c and 4.6.2c - Instrumentation That Initiates Or Isolates Emergency Cooling

Note (d) of current TS Tables 3.6.2c and 4.6.2c provides that an affected instrument channel (Parameters (1) High-Reactor Pressure, (2) Low-Low Reactor Water Level, and (3) High Steam Flow Emergency Cooling System) may be placed in an inoperable status for up to 2 hours for required surveillances without placing the Trip System in the tripped condition provided at least one



operable channel in the same Trip System is monitoring that parameter. The proposed change would revise the 2 hours to 6 hours. The proposed change would also add Notes (e) and (f) to TS Tables 3.6.2c and 4.6.2c to read:

- (e) With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement:
 - 1. For one channel inoperable, place the inoperable channel in the tripped condition within 24 hours or take the action required by Specification 3.6.2a for that Parameter.
 - 2. With more than one channel inoperable, take the ACTION required by Specification 3.6.2a for that Parameter.

- (f) With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement for one trip system, either
 - 1. Place the inoperable channel(s) in the tripped condition within 24 hours.

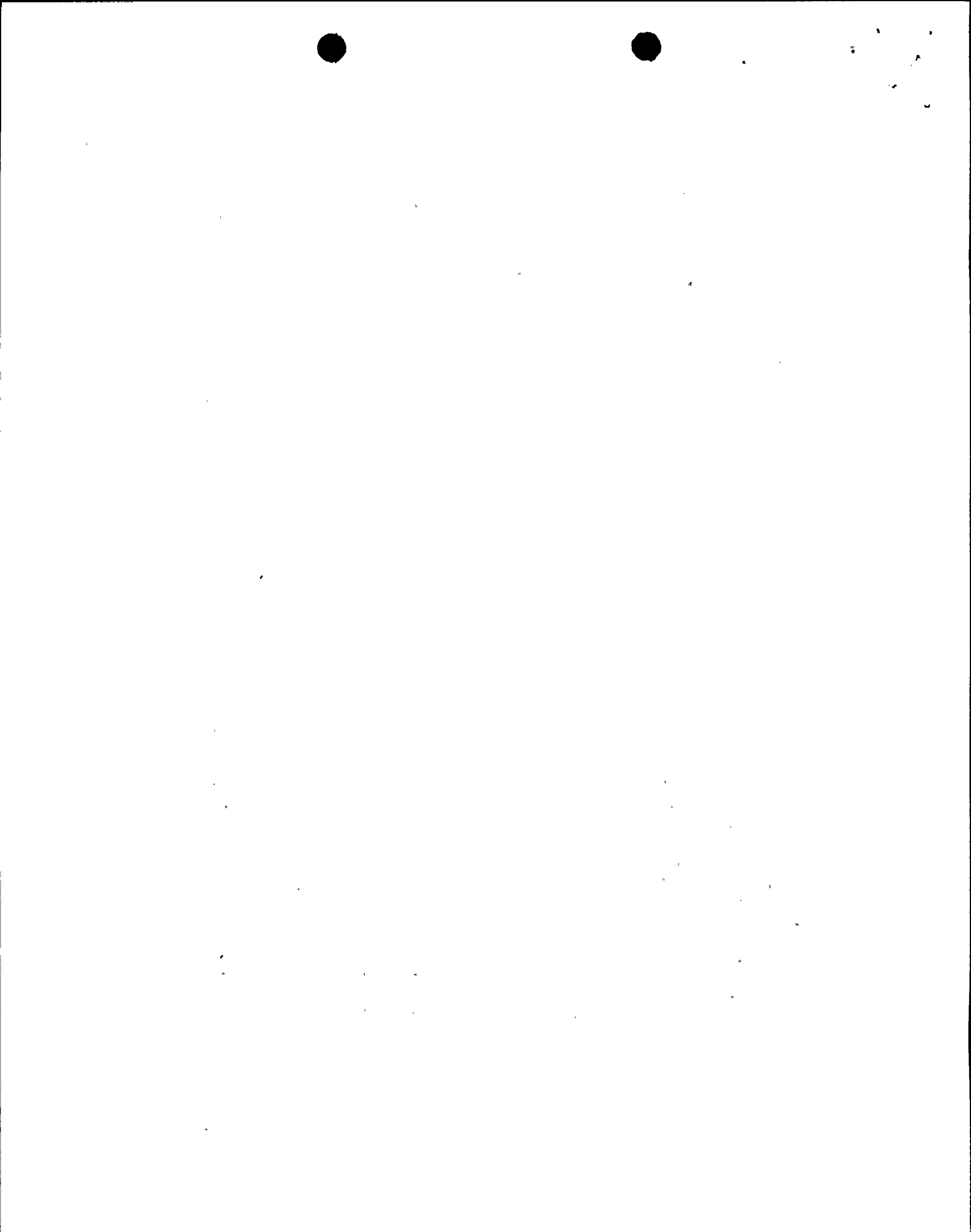
or

 - 2. Take the ACTION required by Specification 3.6.2a for that Parameter.

With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement for both trip systems,

- 1. Place the inoperable channel(s) in one trip system in the tripped condition within one hour
- and
- 2. a. Place the inoperable channel(s) in the remaining trip system in the tripped condition within 24 hours.
- or
- b. Take the ACTION required by Specification 3.6.2a for that Parameter.

The revision to Note (d) would increase the allowable out-of-service times for performing required surveillances for the affected instrument channels (Parameters (1) High Reactor Pressure, (2) Low-Low Reactor Water Level, and



(3) High Steam Flow Emergency Cooling System) from 2 hours to 6 hours. The proposed changes in allowable out-of-service times are consistent with the provisions of NEDC-30936P-A, NEDC-31677P-A, and RE-003 and are, therefore, acceptable.

The proposed addition of Notes (e) and (f) would specify actions to be taken when one or more channels are inoperable. The actions to be taken if channels are inoperable are consistent with current NRC staff positions and with the guidance provided in NUREG-1433, ensure that a loss of function will not exist if two or more channels are inoperable, and are, therefore, acceptable.

The proposed changes in Table 4.6.2c would increase the Instrument Channel Test surveillance frequencies of the following Parameters from once per month to once per 3 months:

EMERGENCY COOLING INITIATION

- 1) High Reactor Pressure - Parameter (1)
- 2) Low-Low Reactor Water Level - Parameter (2)

The proposed changes in surveillance test frequencies are consistent with NEDC-30936P-A (Parts 1 and 2) and NEDC-31677P-A and are, therefore, acceptable.

Tables 3.6.2d and 4.6.2d - Instrumentation That Initiates Core Spray

Tables 3.6.2d and 4.6.2d of the current TS contain a Note (f) on TS page 210 that was applicable only during the spring 1986 refueling outage. The proposed change would delete this note. Deletion of this note is purely an administrative change since the note is no longer applicable and therefore, its deletion is acceptable.

Note (g) of current TS Tables 3.6.2d and 4.6.2d provides that an affected instrument channel (Parameter (1) High Drywell Pressure, (2) Low-Low Reactor Water Level, and (3) Reactor Pressure and either (1) or (2) above) may be placed in an inoperable status for up to 2 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter. The proposed change would revise the 2 hours to 6 hours, redesignate Note (g) as Note (f), and expand the redesignated Note (f) to read:

- (f) A channel may be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one Operable Instrument Channel in the same Trip System is monitoring that parameter.



With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement:

1. With one channel inoperable, place the inoperable channel in the tripped condition within 24 hours or take the ACTION required by Specification 3.6.2a for that Parameter.
2. With more than one channel inoperable, take the ACTION required by Specification 3.6.2a for that Parameter.

The revision to redesignated Note (f) to increase the allowable out-of-service times for performing required surveillances for the affected instrument channels from 2 hours to 6 hours is consistent with the provisions of NEDC-30936P-A, Part 2, and is, therefore, acceptable.

The proposed addition to redesignated Note (f) would specify actions to be taken when one or more channels are inoperable. The actions to be taken if channels are inoperable are consistent with current NRC staff positions and with the guidance provided in NUREG-1433, ensure that a loss of function will not exist if two or more channels are inoperable, and are, therefore, acceptable.

The proposed changes in Table 4.6.2d would increase the Instrument Channel Test surveillance frequencies of the following Parameters from once per month to once per 3 months:

- a. START CORE SPRAY PUMPS
 - 1) High Drywell Pressure - Parameter (1)
 - 2) Low-Low Reactor Water Level - Parameter (2)
- b. OPEN CORE SPRAY DISCHARGE VALVES
 - 1) Reactor Pressure and either (1) or (2) above - Parameter (3)

The proposed changes in surveillance test frequencies are consistent with NEDC-30936P-A (Parts 1 and 2) and RE-003 and are, therefore, acceptable.

Tables 3.6.2e and 4.6.2e - Instrumentation That Initiates Containment Spray

Note (c) of current TS Tables 3.6.2e and 4.6.2e provides that an affected instrument channel (Parameter (1)a. High Drywell Pressure and b. Low-Low Reactor Water Level) may be placed in an inoperable status for up to 2 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter. The proposed change would revise the 2 hours to 6 hours and expand Note (c) to read:



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- (c) A channel may be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip system in the tripped condition provided at least one Operable Instrument Channel in the same Trip System is monitoring that parameter.

With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement:

1. With one channel inoperable, place the inoperable channel in the tripped condition within 24 hours or take the ACTION required by Specification 3.6.2a for that Parameter.
2. With more than one channel inoperable, take the ACTION required by Specification 3.6.2a for that Parameter.

The revision to Note (c) to increase the allowable out-of-service time for performing required surveillances for the affected instrument channels from 2 hours to 6 hours is consistent with the provisions of GENE-770-06-1 and is, therefore, acceptable.

The proposed addition to Note (c) would specify actions to be taken when one or more channels are inoperable. The actions to be taken if channels are inoperable are consistent with current NRC staff positions and with the guidance provided in NUREG-1433, ensure that a loss of function will not exist if two or more channels are inoperable, and are, therefore, acceptable.

The proposed changes in Table 4.6.2e would increase the Instrument Channel Test surveillance frequencies of the following Parameters from once per month to once per 3 months:

INITIATES CONTAINMENT SPRAY

- 1) High Drywell Pressure - Parameter (1)a.
- 2) Low-Low Reactor Water Level - Parameter (1)b.

The proposed changes in surveillance test frequencies are consistent with GENE-770-06-1 and are, therefore, acceptable.

Tables 3.6.2f and 4.6.2f - Instrumentation That Initiates Auto Depressurization

Note (d) of current TS Tables 3.6.2f and 4.6.2f provides that an affected instrument channel (Parameter (1) a. Low-Low-Low Reactor Water Level and b. High Drywell Pressure) may be placed in an inoperable status for up to 2 hours for required surveillances without placing the Trip System in the



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tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter. The proposed change would revise the 2 hours to 6 hours and expand Note (d) to read:

- (d) A channel may be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter.

With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement:

1. With one channel inoperable, place the inoperable channel in the tripped condition within 24 hours or take the ACTION required by Specification 3.6.2a for that Parameter.
2. With more than one channel inoperable, take the ACTION required by Specification 3.6.2a for that Parameter.

The revision to Note (d) to increase the allowable out-of-service time for performing required surveillances for the affected instrument channels from 2 hours to 6 hours is consistent with the provisions of NEDC-30936P-A and is, therefore, acceptable.

The proposed addition to Note (d) would specify actions to be taken when one or more channels are inoperable. The actions to be taken if one or more channels are inoperable are consistent with current NRC staff positions and with the guidance provided in NUREG-1433, ensure that a loss of function will not exist if two or more channels are inoperable, and are, therefore, acceptable.

The proposed changes in Table 4.6.2f would increase the Instrument Channel Test surveillance frequencies of the following Parameters from once per month to once per 3 months:

INITIATES AUTO DEPRESSURIZATION

- 1) Low-Low-Low Reactor Water - Parameter (1)a.
- 2) High Drywell Pressure - Parameter (1)b.

The proposed changes in surveillance test frequencies are consistent with NEDC-30936P-A (Parts 1 and 2) and RE-003 and are, therefore, acceptable.



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Tables 3.6.2g and 4.6.2g - Instrumentation That Initiates Control Rod Withdrawal Block

The proposed change would add Note (i) to TS Tables 3.6.2g and 4.6.2g. Note (i) would permit an affected instrument channel (Parameter (1) SRM a. Detector not in Startup Position, b. Inoperative, c. Upscale, (2) IRM a. Detector not in Startup Position, b. Inoperative, c. Downscale, d. Upscale, and (3) APRM a. Inoperative, b. Upscale (Biased By Recirculation Flow), c. Downscale) to be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one other operable channel in the same Trip System is monitoring that Parameter. The proposed addition of Note (i) is consistent with NEDC-30851P-A (Supplement 1) and GENE-770-06-1 and is, therefore, acceptable.

The proposed changes in Table 4.6.2g would increase the Instrument Channel Test surveillance frequencies of the following Parameters from once per month to once per 3 months:

a. APRM CONTROL ROD WITHDRAWAL BLOCK

- 1) Inoperative - Parameter (3)a.
- 2) Upscale (Biased by Recirculation Flow) - Parameter (3)b.
- 3) Downscale - Parameter (3)c.

b. RECIRCULATION FLOW CONTROL ROD WITHDRAWAL BLOCK

- 1) Comparator Off Normal - Parameter (4)a.
- 2) Flow Unit Inoperative - Parameter (4)b.
- 3) Flow Unit Upscale - Parameter (4)c.

The proposed changes in surveillance test frequencies are consistent with NEDC-30851P-A (Supplement 1) and GENE-770-06-1 and are, therefore, acceptable.

Tables 3.6.2h and 4.6.2h - Vacuum Pump Isolation

The proposed change would add Note (b) to TS Tables 3.6.2h and 4.6.2h. Proposed Note (b) would read as follows:

- (b) A channel may be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter.

With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement for one trip system, either:



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1. Place the inoperable channel(s) in the tripped condition within 12 hours.

or

2. Take the ACTION required by Specification 3.6.2a for that Parameter.

With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement for both trip systems,

1. Place the inoperable channel(s) in one trip system in the tripped condition within one hour.

and

2. a. Place the inoperable channel(s) in the remaining trip system in the tripped condition within 12 hours.

or

- b. Take the ACTION required by Specification 3.6.2a for that Parameter.

Note (b) would permit an affected instrument channel (Parameter - High Radiation Main Steam Line) to be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that Parameter. Note (b) would also provide the actions to be taken if one or more channels are inoperable. Permission to place an instrument channel in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that Parameter is consistent with NEDC-31677P-A and is, therefore, acceptable. The actions to be taken if one or more channels are inoperable are consistent with current NRC staff positions and with the guidance provided in NUREG-1433, ensure that a loss of function will not exist if two or more channels are inoperable, and are, therefore, acceptable.

The proposed changes in Table 4.6.2h would increase the Instrument Channel Test surveillance frequency for the High Radiation Main Steam Line parameter from once per week to once per 3 months. This proposed change is consistent with NEDC-31677P-A and is, therefore, acceptable.

Tables 3.6.2j and 4.6.2j - Emergency Ventilation Initiation

The proposed change would add Note (d) to TS Tables 3.6.2j and 4.6.2j. Proposed Note (d) would read as follows:



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- (d) A channel may be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one Operable Instrument Channel in the same Trip system is monitoring that parameter.

With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels for the Operable Trip System, either

- 1) Place the inoperable channel(s) in the tripped condition within 24 hours.

or

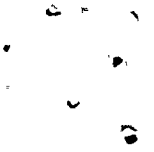
- 2) Take the ACTION required by Specification 3.6.2a for that Parameter.

Note (d) would permit an affected instrument channel (Parameter (1) High Radiation Reactor Building Ventilation Duct) to be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter. Note (b) would also provide the actions to be taken if one or more channels are inoperable. Permission to place an instrument channel in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter is consistent with NEDC-31677P-A and is, therefore, acceptable. The actions to be taken if one or more channels are inoperable are consistent with current NRC staff positions and with the guidance provided in NUREG-1433, ensure that a loss of function will not exist if two or more channels are inoperable, and are, therefore, acceptable.

Tables 3.6.2k and 4.6.2k - High Pressure Coolant Injection

Note (c) of current TS Tables 3.6.2k and 4.6.2k provides that an affected instrument channel (Parameter (1) Low Reactor Water Level) may be placed in an inoperable status for up to 2 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter. The proposed change would revise the 2 hours to 6 hours and expand Note (c) to read:

- (c) A channel may be placed in an inoperable status for up to 6 hours for required surveillances without placing the Trip System in the tripped condition provided at least one operable channel in the same Trip System is monitoring that parameter.



With the number of Operable channels less than required by the Minimum Number of Operable Instrument Channels per Operable Trip System requirement:

1. For one channel inoperable, place the inoperable channel in the tripped condition within 24 hours or take the ACTION required by Specification 3.6.2a for that Parameter.
2. With more than one channel inoperable, take the ACTION required by Specification 3.6.2a for that Parameter.

The revision to Note (c) to increase the allowable out-of-service time for performing required surveillances for the affected instrument channels from 2 hours to 6 hours is consistent with the provisions of NEDC-30936P-A (Parts 1 and 2) and RE-003 and is, therefore, acceptable.

The proposed addition to Note (c) would specify actions to be taken when one or more channels are inoperable. The actions to be taken if one or more channels are inoperable are consistent with current NRC staff positions and with the guidance provided in NUREG-1433, ensure that a loss of function will not exist if two or more channels are inoperable, and are, therefore, acceptable.

The proposed change in Table 4.6.2k would increase the Instrument Channel Test surveillance frequency for the Low Reactor Water Level parameter from once per month to once per 3 months. This proposed change is consistent with NEDC-30936P-A (Parts 1 and 2) and RE-003 and is, therefore, acceptable.

Tables 3.6.11-1 and 4.6.11 - Accident Monitoring Instrumentation

The proposed change would add footnote (*) to Table 3.6.11-1. Footnote (*) would permit one channel of the affected instrument channels (Parameters (1) Relief Valve Position Indication, (2) Safety Valve Position Indication, (3) Reactor Vessel Water Level, and (5) Suppression Chamber Water Level) to be placed in an inoperable status for up to 6 hours for required surveillances provided at least one operable channel is monitoring that Parameter. These proposed changes are consistent with GENE-770-06-1 and are, therefore, acceptable.

The proposed changes in Table 4.6.11 would increase the Instrument Channel Test surveillance frequencies of the following Parameters from once per month to once per quarter.

- a. Relief Valve Position Indication (Primary and Backup) - Parameter (1)
- b. Safety Valve Position Indication (Primary and Backup) - Parameter (2)
- c. Reactor Vessel Water Level - Parameter (3)
- d. Suppression Chamber Water Level - Parameter (5)

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The proposed changes in surveillance test frequencies are consistent with GENE-770-06-1 and are, therefore, acceptable.

The proposed change would also modify the Bases for TS 3.6.2, 4.6.2, 3.6.11, and 4.6.11 to reference the GE LTRs which justify the above proposed changes. The NRC staff offers no objection to the proposed Bases changes.

Proposed TS pages 191 and 196a were modified to incorporate changes approved by License Amendment No. 138 which was issued on January 8, 1993.

NMPC included TS pages 211, 213, 218, 219, and 224 in the December 4, 1992, submittal. However, no changes were proposed on these pages; therefore, they are not included in this amendment.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 61117). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Brinkman

Date: February 24, 1993



February 24, 1993

Docket No. 50-220

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT NO. 1
(TAC NO. M85109)

The Commission has issued the enclosed Amendment No.139 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated December 4, 1992, as supplemented February 12, 1993, and February 17, 1993.

The amendment revises TS 3.6.2, 4.6.2, 3.6.11, and 4.6.11 and associated Bases to increase the surveillance test intervals and add allowable out-of-service times for various instruments. The changes are in accordance with General Electric Company Licensing Topical Reports which have been previously reviewed and approved by the NRC staff. The allowable out-of-service times are consistent with the provisions of NUREG-1433, "Standard Technical Specifications, General Electric Plants, BWR/4." The changes permit specified instrument channel functional tests to be performed once per 3 months rather than once per week or once per month. The amendment also makes editorial corrections on TS pages 192, 199, and 210.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,
Original signed by:
Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 139 to DPR-63
2. Safety Evaluation
cc w/enclosures:
See next page

Distribution:
See next page

*See previous concurrence

PDI-1:LA	PDI-1:PM <i>A/B</i>	OTSB	HICB	OGC <i>AB</i>	PDI-1:D
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