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#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555

## NIAGARA MOHAWK POWER CORPORATION

#### - DOCKET NO. 50-220

## NINE MILE POINT NUCLEAR STATION UNIT NO. 1

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 136 License No. DPR-63

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated September 17, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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## (2) <u>Technical Specifications</u>

The Technical Specification's contained in Appendices A and B, as revised through Amendment No. 136, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented prior to startup from the next refueling outage.

**FOR THE NUCLEAR REGULATORY COMMISSION** 

Robert a. Capi

Robert A. Capra, Director Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 17, 1992

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# ATTACHMENT TO LICENSE AMENDMENT

# AMENDMENT NO. 136 TO FACILITY OPERATING LICENSE NO. DPR-63

# DOCKET\_NO. 50-220

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
iiia	iiia
241ii1	241ii1
241ii2	241ii2
241ii3	241ii3

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SECTION		DESCRIPTIC	NC	PAGE
3.6.5	Radioactive Material Sources	4.6.5	Radioactive Material Sources	241k
3.6.6	(Deleted)	4.6.6	(Deleted)	
3.6.7	(Deleted)	4.6.7	(Deleted)	-
3.6.8	(Deleted)	4.6.8	(Deleted)	
3.6.9	(Deleted)	4.6.9	(Deleted)	•
3.6.10	(Deleted)	4.6.10	(Deleted)	
3.6.11	Accident Monitoring Instrumentation	4.6.11	Accident Monitoring Instrumentation	241ee
3.6.12	Reactor Protection System and Reactor Trip System Power Supply Monitoring	4.6.12	Reactor Protection System and Reactor Trip System Power Supply Monitoring	241ii1
3.6.13	Remote Shutdown Panels	4.6.13	Remote Shutdown Panels	241ii4
3.6.14	Radioactive Effluent Instrumentation	4.6.14	Radioactive Effluent Instrumentation	241jj
3.6.15	Radioactive Effluents	4.6.15	Radioactive Effluents	241ww
3.6.16	Radioactive Effluent Treatment Systems	4.6.16	Radioactive Effluent Treatment Systems	241qqq
3.6.17	Explosive Gas Mixture	4.6.17	Explosive Gas Mixture	241ttt
3.6.18	Mark I Containment	4.6.18	Mark I Containment	241vvv
3.6.19	Liquid Waste Holdup Tanks	4.6.19	Liquid Waste Holdup Tanks	241xxx
3.6.20	Radiological Environmental Monitoring Program	4.6.20	Radiological Environmental Monitoring Program	241zzz
3.6.21	Interlaboratory Comparison Program	4.6.21	Interlaboratory Comparison Program ·	2411111
3.6.22	Land Use Census	4.6.22	Land Use Census	241nnnn
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Amendment No. 22,92,52,58,72,182, 136

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## LIMITING CONDITION FOR OPERATION

## 3.6.12 <u>REACTOR PROTECTION SYSTEM AND REACTOR</u> <u>TRIP SYSTEM POWER SUPPLY MONITORING</u>

## Applicability:

Applies to the operability of instrumentation that provides protection of the reactor protection system and reactor trip system.

#### Objective:

To assure the operability of the instrumentation monitoring the power to the reactor protection system and reactor trip system.

#### **Specification:**

a. Except as specified in specifications b and c below, two\_protective relay systems shall be operable for each power supply.

#### SURVEILLANCE REQUIREMENT

## 4.6.12 <u>REACTOR PROTECTION SYSTEM AND REACTOR</u> <u>TRIP SYSTEM POWER SUPPLY MONITORING</u>

## Applicability:

Applies to the surveillance of instrumentation that provides protection of the reactor protection system and reactor trip system.

#### **Objective:**

To verify the operability of protection instrumentation monitoring the power to the reactor protection and reactor trip buses.

## Specification:

a. At least once every six months

Demonstrate operability of the overvoltage, undervoltage and underfrequency protective instrumentation by performing an instrument channel test. This instrument channel test will consist of simulating abnormal power conditions by applying from a test source, an overvoltage signal, an undervoltage signal and an underfrequency signal to verify that the tripping logic up to but not including the output contactors functions properly.

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LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
3.6.12 <u>REACTOR PROTECTION SYSTEM AND REACTOR</u> <u>TRIP SYSTEM POWER SUPPLY MONITORING</u> (cont'd)	4.6.12 <u>REACTOR PROTECTION SYSTEM AND REACTOR</u> <u>TRIP SYSTEM POWER SUPPLY MONITORING</u> (cont <sup>*</sup> d)
Specification: (cont'd)	Specification: (cont'd)
b. With one protective relaying system inoperable, restore the inoperable system to an operable status within 72 hours or remove the power supply from service.	<ul> <li>b. At least once per refueling cycle Demonstrate operability of the overvoltage, undervoltage and underfrequency protective instrumentation by performing an instrument channel test. This instrument channel test will consist of simulating abnormal power conditions by applying from a test source an overvoltage signal, an undervoltage signal and an underfrequency signal to verify that the tripping logic including the output contactors functions properly at least once. In addition, a sensor calibration will be performed to verify the following setpoints.</li> </ul>
c. With both protective relaying systems inoperable, restore at least one to an operable status within 30 minutes or remove the power supply from service.	i. Overvoltage $\leq$ 132 volts, $\leq$ 4 seconds ii. Undervoltage $\geq$ 108 volts, $\leq$ 4 seconds iii. Underfrequency $\geq$ 57 hertz, $\leq$ 2 seconds

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# BASES FOR 3.6.12 AND 4.6.12 REACTOR PROTECTION SYSTEM AND REACTOR TRIP SYSTEM POWER SUPPLY MONITORING

To eliminate the potential for undetectable single component failure which could adversely affect the operability of the reactor protectionsystem and reactor trip system, protective relaying schemes are installed on Motor Generator Sets 131 and 141, Static Uninterruptible Power Supply Systems 162 and 172, and maintenance bus 130A. This provides for overvoltage, undervoltage and underfrequency protection.

Amendment No. 62, 136

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