

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. NPF-69

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT 2

DOCKET NO. 50-410

1.0 INTRODUCTION

By letter dated January 29, 1992, which superseded the November 6, 1991, submittal, Niagara Mohawk Power Corporation (the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station, Unit 2, Technical Specifications (TS). The proposed amendment would delete the TS tables that include lists of components referenced in individual specifications. In addition, the TS requirements would be modified such that all references to these tables would be removed. Finally, the TS would be modified to state requirements in general terms that include the components listed in the tables. Guidance on the proposed TS changes was provided by Generic Letter 91-08, dated May 6, 1991.

2.0 <u>EVALUATION</u>

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The licensee has proposed the removal of Table 3.6.3-1, "Primary Containment Isolation Valves," that is referenced in TS 3/4.6.3. With the removal of this table, the licensee has proposed to include the following statement of the LCO under TS 3.6.3:

Each primary containment isolation valve and reactor instrumentation line excess flow check valve shall be OPERABLE.

In addition, the licensee proposes to remove all references to Table 3.6.3-1 by revising the following: the definition of Containment Integrity (TS 1.31); TS 4.6.1.1; the action requirements under TS 3.6.3; TS 4.6.3.1 through 4.6.3.4; TS 3.6.12; and TS 4.6.12. The definition of Containment Integrity and TS 4.6.1.1 refer to TS 3.6.3 for an exception that is now covered by a footnote to the LCO rather than by the table removed from the TS. With the removal of the reference to Table 3.6.3-1, the licensee has proposed to state this exception as:

..., except as provided in Specification 3.6.3.

The surveillance requirements of TS 4.6.3.1 through 4.6.3.4 would be revised to state: "Each primary containment isolation valve shall...," "Each primary containment automatic isolation valve shall...," "... each primary containment power operated or automatic valve shall...," and "each reactor instrumentation



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. . line excess flow check valve shall..." rather than stating the requirements in relation to the valves listed in Table 3.6.3-1.

With the deletion of Table 3.6.3-1, the operability requirements would be stated in general terms that apply to all containment isolation valves including those that are locked or sealed closed. These valves are locked or sealed closed consistent with the regulatory requirements for manuallyoperated valves that are used as containment isolation valves. Because opening these valves would be contrary to the operability requirements of these valves, the following footnote to the LCO has been proposed.

Locked or sealed closed valves may be opened on an intermittent basis under administrative control.

The licensee has proposed the removal of Table 3.8.4.1-1, "Primary Containment AC Circuits Deenergized" that is referenced in TS 3.8.4.1. With the removal of this table, the licensee has proposed to include the following statement of the LCO under TS 3.8.4.1:

The AC circuits inside primary containment that are not provided with primary and backup containment penetration conductor overcurrent protective devices shall be deenergized:

In addition, the licensee has proposed to revise the action requirements under TS 3.8.4.1 to remove the reference to Table 3.8.4.1-1 as follows:

With any of the above required circuits energized, trip the associated circuit breaker(s) within 1 hour.

The licensee has proposed to add the following statement to the footnote for LCO 3.8.4.1 to further define which circuits shall be deenergized:

Excluded from this specification are those penetration assemblies that are capable of withstanding the maximum current available because of an electrical fault inside containment.

The licensee has proposed the removal of Table 3.8.4.3-1, "Overcurrent Protective Devices For Non-Class 1E Lighting Fixtures On 1E Emergency System" that is referenced in TS 3.8.4.3. With removal of this table, the licensee has proposed to include the following statement of the LCO under TS 3.8.4.3:

The emergency lighting system overcurrent protection devices shall be OPERABLE.

In addition, the licensee has proposed to revise the ACTION statement for LCO 3.8.4.3 to remove the reference to the table as follows:

With one or more of the overcurrent protective devices inoperable,....

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The licensee has proposed changes to the above TS that are consistent with the guidance provided in Generic Letter 91-08. In addition, the licensee has provided an updated copy of Bases Section of TS 3/4.6.3 that addresses appropriate considerations for opening locked or sealed closed valves on an intermittent basis. Also, changes were made to Sections 3/4.6.3 and 3/4.8.4 in the Bases section to reference the respective administrative procedures where the removed equipment lists are now located. Finally, the licensee has confirmed that component lists removed from the TS have been updated to identify all components for which the TS requirements apply and are located in controlled plant procedures.

The staff has reviewed the proposed changes and we find them to be acceptable because the removal of the equipment lists and references to such was done in accordance with Generic Letter 91-08.

3.0 <u>STATE_CONSULTATION</u>

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 6038). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Thomas G. Dunning Richard A. Laura

Date: March 24, 1992

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March 24, 1992 .

Docket No. 50-410 Mr. B. Ralph Sylvia Executive Vice President, Nuclear Niagara Mohawk Power Corporation 301 Plainfield Road Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION, UNIT 2 (TAC NO. M82635)

The Commission has issued the enclosed Amendment No. ³⁷ to Facility Operating License No. NPF-69 for the Nine Mile Point Nuclear Station Unit 2 (NMP-2). The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated January 29, 1992.

The amendment revises Technical Specifications 1.31, "Primary Containment Integrity;" 3/4.6.1, "Primary Containment;" 3/4.6.3, "Primary Containment Isolation Valves;" 3/4.8.4, Electrical Equipment Protective Devices;" and deletes associated Tables 3.6.3-1, 3.8.4.1-1, and 3.8.4.3-1. The removal of the equipment lists contained in the tables allows for administrative control of any future changes to the lists without processing a license amendment. This is in accordance with Generic Letter 91-08.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly <u>Federal Register</u> notice.

Sincerely,

Original Signed By Donald Brinkman for Richard Laura Richard A. Laura, Acting Project Manager Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 37 to NPF-69

2. Safety Evaluation cc w/enclosures:

See next page

OFFICE	LA:PDI-1	PM:PDI-1	OGC*	D:PDI-1 Roc	
NAME	CVogan 🕖	RLaura: av 7/m	JHul 1	RACapra	
DATE	3/11/92	3 /12/92	2 /26/92	3 /24/92	

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DATED: <u>March 24, 1992</u>

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AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. NPF-69-NINE MILE POINT UNIT 2

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Docket_File_ NRC & Local PDRs PDI-1 Reading S. Varga, 14/E/4 J. Calvo, 14/A/4 R. Capra C. Vogan R. Laura C. Cowgill OGC-WF D. Hagan, 3302 MNBB G. Hill (4), P-137 Wanda Jones, P-130A C. Grimes, 11/F/23 T. Dunning, 11/E/22 ACRS (10) OPA OC/LFMB PD Plant-specific file cc: Plant Service list · • .

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Mr. B. Ralph Sylvia

August 11, 1993

Copies of the Exemption and the NRC staff's supporting safety evaluation are enclosed. The Exemption has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Robert a. Coper

Robert A. Capra, Director Project Directorate I-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosure:

1. Exemption

2. Safety Evaluation

cc w/enclosures: See next page

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Mr. B. Ralph Sylvia Niagara Mohawk Power Corporation

cc: 、

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