

February 15, 1991

Docket No. 50-220

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

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Dear Mr. Sylvia:

SUBJECT: IMPLEMENTATION OF NRC BULLETINS AND GENERIC LETTERS AT NINE
MILE POINT NUCLEAR STATION, UNIT NO. 1

The NRC staff has recently been reviewing the implementation status of NRC bulletins and generic letters issued since the Three Mile Island Unit No. 2 accident in March 1979. This review has been coordinated with members of your staff. The enclosed tables provide the results of this review as applicable to Nine Mile Point Unit No. 1. Although we believe the enclosed tables accurately reflect the implementation status of these bulletins and generic letters, we request that you review the enclosed tables and confirm their accuracy, or provide any corrections, by February 25, 1991.

Please contact me on (301) 492-1402 if you have any questions regarding this matter.

This requirement affects one respondent, and, therefore, is not subject to Office of Management and Budget review under P. L. 96-511.

Sincerely,

ORIGINAL SIGNED BY:

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Key to Table Notations
2. Table 1 - NRC Bulletins
3. Table 2 - NRC Generic Letters

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

February 15, 1991

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Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

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Sincerely,

A handwritten signature in cursive script, reading "Donald S. Brinkman".

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

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2. Table 1 - NRC Bulletins
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cc w/enclosures:
See next page

Mr. B. Ralph Sylvia
Niagara Mohawk Power Corporation

Nine Mile Point Nuclear Station,
Unit No. 1

cc:

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Albany, New York 12223

Key To Table Notations On Tables 1 and 2

Tables 1 and 2 do not include NRC Bulletins and Generic Letters which were obviously not applicable to NMP-1 (e.g., Bulletins and Generic Letters addressed to only PWRs, etc.). The column entitled, "Applicable to NMP-1" indicates whether or not upon evaluation, the bulletin or generic letter was determined applicable to NMP-1. The column entitled, "Response Required" indicates whether or not NMPC was required by the bulletin or generic letter to submit a written response to the NRC; although in some cases a response was not required, NMPC did submit a response and in these cases a response date is indicated. The column entitled, "Response Date" provides the date a written response was submitted to the NRC if a response was required. The column entitled, "Implementation Date" provides the date NMPC fully implemented the requirements of the bulletin or generic letter.

TABLE 1
NRC BULLETINS

ENCLOSURE 2

BL-#	TITLE	APPLICABLE TO NMP1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
*****	*****	*****	*****	*****	*****	*****
** YEAR 79						
79	06 Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	NO	NO	/ /	/ /	
79	06A Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	NO	NO	/ /	/ /	
79	06B Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	NO	NO	/ /	/ /	
79	06C Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	NO	NO	/ /	/ /	
79	07 Seismic Stress Analysis of Safety-Related Piping	YES	YES	06/12/79	06/19/79	06/19/79 CLOSED BY NRC MORE THAN ONE RESPONSE CLOSED BY NRC 5/21/80 SE MORE THAN ONE RESPONSE DATE LICENSEE SUBMITTED RESPONSE
79	08 Events Relevant to Boiling Water Reactors Identified During Three Mile Island Incident	YES	YES	05/14/79	05/21/80	
79	09 Failures of GE Type AK-2 Circuit Breaker in Safety Related Systems	NO	NO	05/07/79	/ /	
79	10 Requalification Training Program Statistics	YES	YES	05/21/79	05/21/79	
79	11 Faulty Overcurrent Trip Device in Circuit Breakers for Engineered Safety Systems	NO	NO	07/30/79	05/21/79	
79	12 Short Period Scrams at BWR Facilities	YES	YES	07/30/79	05/27/81	
79	13 Cracking in Feedwater System Piping	NO	NO	/ /	/ /	
79	14 Seismic Analyses for As-Built Safety-Related Piping Systems	YES	YES	08/17/79	04/25/89	ADDITIONAL WALKDOWNS TO BE PERFORMED DURING ENGINEERING PROGRAM INTEGRATION NRC CLOSED 10/18/90
79	15 Deep Draft Pump Deficiencies -	YES	YES	12/05/89	12/12/89	
79	18 Audibility Problems Encountered on Evacuation of Personnel from High-Noise Areas	YES	YES	09/19/79	09/19/79	
79	19 Packaging of Low-Level Radioactive Waste for Transport and Burial	YES	YES	01/04/80	04/17/81	NO FORMAL NRC RESPONSE BACK
79	21 Temperature Effects on Level Measurements	NO	NO	/ /	/ /	
79	23 Potential Failure of Emergency Diesel Generator Field Exciter Transformer	YES	YES	10/26/79	12/31/79	NO FORMAL NRC RESPONSE BACK
79	24 Frozen Lines	YES	YES	10/24/79	/ /	NO IMPLEMENTATION DATE REQUIRED LICENSEE SUBMITTED RESPONSE
79	25 Failures of Westinghouse BFD Relays in Safety-Related Systems	NO	NO	11/26/79	/ /	
79	26 Boron Loss from BWR Control Blades	YES	YES	12/17/79	12/17/79	
79	27 Loss of Non-Class-1-E Instrumentation and Control Power System Bus During Operation	YES	YES	02/29/80	02/29/80	
79	28 Possible Malfunction of NAMCO Model EA180 Limit Switches at Elevated Temperatures	NO	NO	01/04/80	/ /	LICENSEE SUBMITTED RESPONSE

TABLE 1
NRC BULLETINS

BL-#	TITLE	APPLICABLE TO NMP1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====	=====	=====	=====	=====	=====	=====
** YEAR 80						
80 01	Operability of ADS Valve Pneumatic Supply	NO	NO	01/17/80	/ /	LICENSEE SUBMITTED RESPONSE
80 02	Inadequate Quality Assurance for Nuclear Supplied Equipment	NO	NO	02/19/80	/ /	LICENSEE SUBMITTED RESPONSE
80 03	Loss of Charcoal from Standard Type II, 2 Inch, Tray Adsorber Cells	YES	YES	03/19/80	03/19/80	
80 06	Engineered Safety Feature (ESF) Reset Controls	YES	YES	05/12/80	06/ /81	UNCERTAIN OF THE EXACT DAY OF IMPLEMENTATION (6/81)
80 08	Examination of Containment Liner Penetration Welds	YES	YES	06/09/80	06/09/80	
80 09	Hydramotor Actuator Deficiencies	NO	NO	06/09/80	/ /	LICENSEE SUBMITTED RESPONSE
80 10	Contamination of Nonradioactive System and Resulting Potential for Unmonitored, Uncontrolled Release to Environment	YES	YES	07/02/80	07/02/80	
80 11	Masonry Wall Design	YES	YES	10/25/84	10/25/84	
80 12	Decay Heat Removal System Operability	NO	NO	/ /	/ /	
80 13	Cracking in Core Spray Spargers	YES	YES	05/16/80	05/16/80	PROGRAM ONGOING FOR INSPECTION AT EACH REFUELING OUTAGE
80 14	Degradation of Scram Discharge Volume Capability	YES	YES	07/25/80	07/25/80	
80 15	Possible Loss of Emergency Notification System (ENS) with Loss of Offsite Power	YES	YES	09/03/80	09/03/80	NO FORMAL RESPONSE BACK
80 16	Potential Misapplication of Rosemount...Models 1151 and 1152 Pressure Transmitters with Either "A" or "D" Output Codes	NO	NO	07/28/80	/ /	LICENSEE SUBMITTED RESPONSE
80 17	Failure of 76 of 185 Control Rods to Fully Insert During a Scram at a BWR	YES	YES	09/02/80	12/27/80	SUBMITTED T.S. CHANGE
80 19	Failures of Mercury-Wetted Matrix Relays in Reactor Protective Systems of...Plants Designed by Combustion Engineering	NO	NO	08/27/80	/ /	LICENSEE SUBMITTED RESPONSE
80 20	Failures of Westinghouse Type W-2 Spring Return to Neutral Control Switches	NO	NO	09/11/80	/ /	LICENSEE SUBMITTED RESPONSE
80 21	Valve Yokes Supplied by Malcolm Foundry Company, Inc.	NO	NO	01/14/81	/ /	LICENSEE SUBMITTED RESPONSE
80 23	Failures of Solenoid Valves Manufactured by Valcor Engineering Corporation	NO	NO	12/03/80	/ /	LICENSEE SUBMITTED RESPONSE
80 24	Prevention of Damage Due to Water Leakage Inside Containment (October 17, 1980 Indian Point 2 Event)	YES	YES	01/05/81	01/05/81	
80 25	Operating Problems with Target Rock Safety-Relief Valves at BWRs	YES	YES	10/14/87	10/14/87	

TABLE 1
NRC BULLETINS

BL-#	TITLE	APPLICABLE TO NMP1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====	=====	=====	=====	=====	=====	=====
** YEAR 81						
81	01 Surveillance of Mechanical Snubbers	YES	YES	03/24/81	06/ /81	LICENSEE UNCERTAIN OF THE EXACT IMPLEMENTATION DATE (6/81)
81	02 Failure of Gate Type Valves to Close Against Differential Pressure	NO	NO	09/10/81	/ /	LICENSEE SUBMITTED RESPONSE
81	03 Flow Blockage of Cooling Water to Safety System Components by Corbicula Sp. (Asiatic Clam) and Mytilus Sp. (Mussel)	YES	YES	05/22/81	05/22/81	
** YEAR 82						
82	01 Alteration of Radiographs of Welds in Piping Subassemblies	NO	NO	06/23/82	/ /	LICENSEE SUBMITTED RESPONSE
82	03 Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants	YES	NO	/ /	/ /	REPLACED BY GL 84-11. SEE JUNE 29, 1984 LETTER
82	04 Deficiencies in Primary Containment Electrical Penetration Assemblies	NO	NO	01/20/83	/ /	LICENSEE SUBMITTED RESPONSE
** YEAR 83						
83	01 Failure of Trip Breakers (Westinghouse DB-50) to Open on Automatic Trip Signal	NO	NO	/ /	/ /	
83	02 Stress Corrosion Cracking in Large-Diameter Stainless Steel Recirculation System Piping at BWR Plants	YES	YES	/ /	/ /	REPLACED BY GL 84-11. SEE JUNE 29, 1984 LETTER
83	03 Check Valve Failures in Raw Water Cooling Systems of Diesel Generators	YES	YES	06/14/83	12/ /83	LICENSEE UNCERTAIN OF THE EXACT DAY OF IMPLEMENTATION (12/83)
83	04 Failure of the Undervoltage Trip Function of Reactor Trip Breakers	NO	NO	/ /	/ /	
83	05 ASME Nuclear Code Pumps and Spare Parts Manufactured by the Hayward Tyler Pump Company	NO	NO	06/15/83	/ /	LICENSEE SUBMITTED RESPONSE
83	06 Nonconforming Materials Supplied by Tube-Line Facilities at Long Island City, NY, Houston, TX, & Carol Stream, IL	YES	YES	11/08/83	11/08/83	
83	07 Apparently Fraudulent Products Sold by Ray Miller, Inc.	YES	YES	03/23/84	03/23/84	
83	08 Elect. Circuit Breakers With Undervoltage Trip...in Safety-Related Applications other than the Reactor Trip System	NO	NO	03/01/84	/ /	LICENSEE SUBMITTED RESPONSE
** YEAR 84						
84	01 Cracks in BWR Mark I Containment Vent Headers	YES	YES	09/17/84	09/17/84	
84	02 Failure of GE Type HFA Relays In Use In Class 1E Safety Systems	YES	YES	07/12/84	04/26/86	

TABLE 1
NRC BULLETINS

BL-#		TITLE	APPLICABLE TO NMP1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
84	03	Refueling Cavity Water Seal	YES	YES	11/26/84	11/26/84	
** YEAR 85							
85	01	Steam Binding of Auxiliary Feedwater Pumps	NO	NO	/ /	/ /	
85	02	Undervoltage Trip Attachments of Westinghouse DB-50 Type Reactor Trip Breakers	NO	NO	/ /	/ /	
85	03	Motor-Operated Valve Common Mode Failures During Plant Transients Due to Improper Switch Settings	YES	YES	05/12/88	12/ /88	LICENSEE UNCERTAIN OF EXACT IMPLEMENTATION DAY
** YEAR 86							
86	01	Minimum Flow Logic Problems That Could Disable RHR Pumps	NO	NO	06/06/86	/ /	LICENSEE SUBMITTED RESPONSE
86	02	Static "O" Ring Differential Pressure Switches	NO	NO	07/29/86	/ /	LICENSEE SUBMITTED RESPONSE
86	03	Potential Failure of Multiple ECCS Pumps Due to Single Failure of Air-Operated Valve in Minimum Flow Recirculation Line	NO	NO	11/13/86	/ /	LICENSEE SUBMITTED RESPONSE
** YEAR 87							
87	01	Thinning of Pipe Walls in Nuclear Power Plants	YES	YES	09/14/87	06/ /90	ONGOING PROGRAM LICENSEE UNCERTAIN OF EXACT DAY OF IMPLEMENTATION (6/90)
87	02	Fastener Testing to Determine Conformance with Applicable Material Specifications	YES	YES	07/25/86	07/25/86	
** YEAR 88							
88	01	Defects in Westinghouse Circuit Breakers	NO	NO	03/28/88	/ /	LICENSEE SUBMITTED RESPONSE
88	03	Inadequate Latch Engagement in HFA Type Latching Relays Manufactured by General Electric (GE) Company	YES	YES	07/14/88	07/14/88	
88	04	Potential Safety-Related Pump Loss	YES	YES	05/09/90	/ /	OPEN IMPLEMENTATION DATE
88	05	Nonconforming Materials Supplied by Piping Supplies, Inc. ... and West Jersey Manufacturing Company ...	YES	YES	09/09/88	09/09/88	NRC CLOSEOUT NUREG-1402
88	07	Power Oscillations in Boiling Water Reactors (BWRs)	YES	YES	02/01/89	11/14/89	
88	08	Thermal Stresses in Piping Connected to Reactor Cooling Systems	YES	YES	09/29/88	06/11/89	LICENSEE PROVIDED THE DATE 12/14/89, BUT DID NOT REFERENCE IT.
88	10	Nonconforming Molded-Case Circuit Breakers	YES	YES	10/30/89	02/11/91	

TABLE 1
NRC BULLETINS

BL-#	TITLE	APPLICABLE TO NMP1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====	=====	=====	=====	=====	=====	=====
** YEAR 89						
89 02	Stress Corrosion Cracking of High-Hardness Type 410 .. Bolting .. Anchor Darling S350W .. Swing Check Valves .. Similar	NO	NO	01/18/90	/ /	LICENSEE SUBMITTED RESPONSE
** YEAR 90						
90 01	Loss of Fill-Oil in Transmitters Manufactured by Rosemount	YES	YES	07/20/90	/ /	OPEN IMPLEMENTATION DATE AWAITING NRC APPROVAL
90 02	Loss of Thermal Margin Caused by Channel Box Bow	YES	YES	04/20/90	12/12/90	

TABLE 2

ENCLOSURE 3

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
*****	*****	*****	*****	*****	*****	*****
** YEAR 79						
79 17	Reliability Of Onsite Diesel Generators At Light Water Reactors	YES	NO	/ /	/ /	
79 18	Westinghouse Two-Loop NSSS	NO		/ /	/ /	
79 19	NRC Staff Review Of Responses To I&E Bulletins 79-06 and 79-06a	YES	NO	/ /	/ /	
79 20	Cracking In Feedwater Lines	NO		/ /	/ /	
79 21	Enclosing NUREG/CR 0660, "Enhancement Of On Site Emergency Diesel Generator Reliability"	YES	NO	/ /	/ /	
79 22	Enclosing NUREG-0560, "Staff Report On The Generic Assessment Of Feedwater Transients In PWRs Designed By B&W	NO		/ /	/ /	
79 23	NRC Staff Review Of Responses To I&E Bulletin 79-08	YES	NO	/ /	/ /	
79 24	Multiple Equipment Failures In Safety-Related Systems	NO		/ /	/ /	
79 25	Information Required To Review Corporate Capabilities	YES	YES	08/10/79	08/10/79	
79 26	Upgraded Standard Technical Specification Bases Program	YES	NO	/ /	/ /	
79 27	Operability Testing Of Relief And Safety Relief Valves	NO	NO	09/13/79	/ /	LICENSEE SUBMITTED RESPONSE
79 28	Evaluation Of Semi-Scale Small Break Experiment	NO		/ /	/ /	
79 29	Transmitting NUREG-0473, Revision 2, Draft Radiological Effluent Technical Specifications	YES	NO	/ /	/ /	
79 30	Transmitting NUREG-0472, Revision 2, Draft Radiological Technical Specifications	YES	NO	/ /	/ /	
79 31	Submittal Of Copies Of Response To 6/29/79 NRC Request	YES	NO	/ /	/ /	
79 32	Transmitting NUREG-0578, "TMI-2 Lessons Learned"	YES	YES	01/04/80	12/ /80	LICENSEE UNCERTAIN OF THE EXACT IMPLEMENTATION DATE
79 33	Transmitting NUREG-0576 - Security Training And Qualification Plans	YES	YES	08/ /79	06/09/82	LICENSEE UNCERTAIN OF THE EXACT RESPONSE DATE (8/79)
79 34	New Physical Security Plans (FR 43280-285)	NO		/ /	/ /	
79 35	Regional Meetings To Discuss Impacts On Emergency Planning	YES	NO	/ /	/ /	
79 36	Adequacy Of Station Electric Distribution Systems Voltages	YES	YES	10/03/79	06/ /81	LICENSEE UNCERTAIN OF THE EXACT IMPLEMENTATION DATE (6/81)
79 37	Amendment To 10 CFR 73.55 Deferral From 8/1/79 to 11/1/79	YES	NO	/ /	/ /	
79 38	BWR Off-Gas Systems - Enclosing NUREG/CR-0727	YES	NO	/ /	/ /	

TABLE 2

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
79 39	Transmitting Division 5 Draft Regulatory Guide & Value Impact Statement	NO		/ /	/ /	
79 40	Follow-up Actions Resulting From The NRC Staff Reviews Regarding The TMI-2 Accident	YES	YES	10/18/79	01/18/82	NO IMPLEMENTATION DATE REQUIRED
79 41	Compliance With 40 CFR 190, EPA Uranium Fuel Cycle Standard	YES	YES	10/23/79	10/23/79	
79 42	Potential Unreviewed Question On Interaction Between Non-Safety Grade Systems And Safety Grade Systems	YES	YES	10/09/79	10/09/79	
79 43	Reactor Cavity Seal Ring Generic Issue	NO		/ /	/ /	
79 44	Referencing 6/29/79 Letter Re Multiple Equipment Failures	YES	NO	/ /	/ /	
79 45	Transmittal Of Reports Regarding Foreign Reactor Operating Experiences	YES	NO	/ /	/ /	
79 46	Containment Purging And Venting During Normal Operation - Guidelines For Valve Operability	YES	YES	11/01/90	05/01/80	RECENT 11/90 RESPONSE SHOULD CLOSE OUT ITEM LICENSEE UNCERTAIN OF DAYS
79 47	Radiation Training	YES	YES	11/05/79	11/05/79	
79 48	Confirmatory Requirements Relating to Condensation Oscillation Loads For The Mark I Containment Long Term Program	YES	YES	11/30/79	01/22/85	
79 49	Summary Of Meetings Held On 9/18-20/79 To Discuss Potential Unreviewed Safety Question On Systems Interaction For B&W P1	YES	NO	/ /	/ /	
79 50	Emergency Plans Submittal Dates	YES	NO	/ /	/ /	
79 51	Follow-up Actions Resulting From The NRC Staff Reviews Regarding The TMI-2 Accident	YES	NO	/ /	/ /	
79 52	Radioactive Release At North Anna Unit 1 And Lessons Learned	YES	NO	/ /	/ /	
79 53	ATWS	YES	NO	/ /	/ /	
79 54	Containment Purging And Venting During Normal Operation	YES	YES	08/08/85	07/24/84	
79 55	Summary Of Meeting Held On October 12, 1979 To Discuss Responses To IE Bulletins 79-05c And HPI Termination Criteria	YES	NO	/ /	/ /	
79 56	Discussion Of Lessons Learned Short Term Requirements	YES	YES	07/06/81	10/01/81	
79 57	Acceptance Criteria For The Mark I Containment Long Term Program	YES	YES	11/30/79	01/22/85	SEE GL 79-48
79 58	ECCS Calculations On Fuel Cladding	YES	YES	11/16/79	09/ /80	LICENSEE UNCERTAIN OF THE EXACT IMPLEMENTATION DATE (9/80)

TABLE 2

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
*****	*****	*****	*****	*****	*****	*****
79 59	GENERIC LETTER NEVER ISSUED	NO		/ /	/ /	
79 60	Discussion Of Lessons Learned Short Term Requirements	YES	NO	/ /	/ /	
79 61	Discussion Of Lessons Learned Short Term Requirements	NO		/ /	/ /	
79 62	EECS Calculations On Fuel Cladding	YES	YES	11/16/79	09/ /80	LICENSEE UNCERTAIN OF THE EXACT IMPLEMENTATION DAY
79 63	Upgraded Emergency Plans	YES	NO	/ /	/ /	
79 64	Suspension Of All Operating Licenses	YES	NO	/ /	/ /	
79 65	Radiological Environmental Monitoring Program Requirements - Enclosing Branch Technical Position, Revision 1	YES	NO	/ /	/ /	
79 66	Additional Information RE 11/09/79 Letter On ECCS Calculations	YES	NO	/ /	/ /	
79 67	Estimates For Evacuation Of Various Areas Around Nuclear Power Reactors	YES	YES	02/01/80	02/01/80	
79 68	Audit Of Small Break LOCA Guidelines	YES	NO	/ /	/ /	
79 69	Cladding Rupture, Swelling & Coolant Blockage As A Result Of A Reactor Accident	YES	NO	/ /	/ /	
79 70	Environmental Monitoring For Direct Radiation	YES	NO	/ /	/ /	
** YEAR 80						
80 001	NUREG-0630 " Cladding, Swelling And Rupture - Models For LOCA Analysis"	YES	NO	/ /	/ /	
80 002	Quality Assurance Requirements Regarding Diesel Generator Fuel Oil	YES	YES	02/22/80	02/22/80	
80 003	BWR Control Rod Failures	YES	NO	/ /	/ /	
80 004	IEB 80-01 Operability Of ADS Valve Pneumatic Supply	YES	NO	/ /	/ /	
80 005	IEB 79-01b Environmental Qualification Of Class 1E Equipment	YES	NO	/ /	/ /	
80 006	Issuance Of NUREG-0313, Rev 1, "Technical Report On Material Selection And Processing Guidelines For BWR Coolant Pressur	YES	NO	/ /	/ /	
80 007	Information Regarding The Program For Environmental Qualification Of Safety-Related Electrical Equipment	YES	NO	/ /	/ /	
80 008	IEB 80-02 Inadequate Quality Assurance For Nuclear Supplied Equipment	YES	NO	/ /	/ /	
80 009	Low Level Radioactive Waste Disposal	YES	NO	/ /	/ /	
80 010	Issuance Of NUREG-0588, "Interim Staff Position On Equipment Qualifications Of Safety-Related Electrical Equipment	YES	NO	/ /	/ /	
80 011	IEB 80-03 Loss Of Charcoal From Standard Type II, 2 Inch, Tray Absorber Cells	YES	NO	/ /	/ /	

TABLE 2

Page No. 4
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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====	=====	=====	=====	=====	=====	=====
80	012 IEB 80-04 Analysis Of A PWR Main Steam Sine Break With Continued Feedwater Addition	NO		/ /	/ /	
80	013 Qualification Of Safety Related Electrical Equipment	YES	NO	/ /	/ /	
80	014 LWR Primary Coolant System Pressure Isolation Valves	YES	YES	02/23/80	12/ /81	LICENSEE UNCERTAIN OF THE EXACT DAY OF IMPLEMENTATION LICENSEE SUBMITTED RESPONSE
80	015 Request For Additional Management And Technical Resources Information	YES	NO	/ /	/ /	
80	016 IEB 79-01b Environmental Qualification Of Class 1E Equipment	YES	NO	/ /	/ /	
80	017 Modifications To Boiling Water Reactor Control Rod Drive Systems	YES	NO	/ /	/ /	
80	018 Crystal River 3 Reactor Trip From Approximately 100% Full Power	NO		/ /	/ /	
80	019 Resolution Of Enhanced Fission Gas Release Concern	YES	NO	/ /	/ /	
80	020 Actions Req.. From OL Applicants Of NSSS Designs By W and CE Resulting From NRC B&O Task Force Review Of TMI2 Accident	NO		/ /	/ /	
80	021 IEB 80-05 Vacuum Condition Resulting In Damage To Chemical Volume Control System Holdup Tanks	NO		/ /	/ /	
80	022 Transmittal Of NUREG-0654 "Criteria For Preparation And Evaluation Of Radiological Emergency Response Plans And Prepared	YES	NO	/ /	/ /	
80	023 Change Of Submittal Date For Evaluation Time Estimates	YES	NO	/ /	/ /	
80	024 Transmittal Of Information On NRC "Nuclear Data Link Specifications"	YES	NO	/ /	/ /	
80	025 IEB 80-06 Engineering Safety Feature (ESF) Reset Controls	YES	NO	/ /	/ /	
80	026 Qualifications Of Reactor Operators	YES	NO	/ /	/ /	
80	027 IEB 80-07 BWR Jet Pump Assembly Failure	YES	NO	/ /	/ /	
80	028 IEB 80-08 Examination Of Containment Liner Penetration Welds	YES	NO	/ /	/ /	
80	029 Modifications To Boiling Water Reactor Control Rod Drive Systems	YES	NO	/ /	/ /	
80	030 Clarification Of The Term "Operable" As It Applies To Single Failure Criterion For Safety Systems Required By TS	YES	YES	05/20/80	02/02/84	
80	031 IEB 80-09 Hydramotor Actuator Deficiencies	YES	NO	/ /	/ /	
80	032 Information Request On Category I Masonry Walls Employed By Plants Under CP and OL Review	NO		/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
*****	*****	*****	*****	*****	*****	*****
80	033 Actions Req. From OL Applicants Of B&W Designed NSSS Resulting From NRC B&O Task Force Review Of TMI2 Accident	NO		/ /	/ /	
80	034 Clarification Of NRC Requirements For Emergency Response Facilities At Each Site	YES	NO	/ /	/ /	
80	035 Effect Of A DC Power Supply Failure On ECCS Performances	NO		/ /	/ /	
80	036 IEB 80-10 Contamination Of Non-Radioactive System And Resulting Potential For Unmonitored, Uncontrolled Release To Envir	YES	NO	/ /	/ /	
80	037 Five Additional TMI-2 Related Requirements To Operating Reactors	YES	YES	06/20/80	03/ /81	MOST RECENT RESPONSE DATE USED LICENSEE UNCERTAIN OF DAY OF IMPLEMENTATION
80	038 Summary Of Certain Non-Power Reactor Physical Protection Requirements	NO		/ /	/ /	
80	039 IEB 80-11 Masonry Wall Design	YES	NO	/ /	/ /	SEE RESPONSE TO IEB 80-11
80	040 Transmittal Of NUREG-0654 "Report Of The B&O Task Force" And Appropriate NUREG-0626 "Generic Eval Of FW Transient And Sbl	YES	NO	/ /	/ /	
80	041 Summary Of Meetings Held On April 22 & 23, 1980 With Representatives Of The Mark I Owners Group	YES	NO	/ /	/ /	
80	042 IEB 80-12 Decay Heat Removal System Operability	NO		/ /	/ /	
80	043 IEB 80-13 Cracking In Core Spray Spargers	YES	NO	/ /	/ /	
80	044 Reorganization Of Functions And Assignments Within ONRR/SSPB	NO		/ /	/ /	
80	045 Fire Protection Rule	YES	NO	/ /	/ /	
80	046 Generic Technical Activity A-12 Fracture Toughness	YES	NO	/ /	/ /	
80	047 Additional Guidance On "Potential For Low Fracture Toughness And Laminar Tearing On PWR SG & Reactor Coolant Pump Suprts	YES	NO	/ /	/ /	
80	048 Revision To 5/19/80 Letter On Fire Protection	YES	NO	/ /	/ /	
80	049 Nuclear Safeguards Problems	YES	NO	/ /	/ /	
80	050 Generic Activity A-10 BWR Cracks	YES	NO	/ /	/ /	
80	051 On-Site Storage Of Low-Level Waste	YES	NO	/ /	/ /	
80	052 Five Additional TMI-2 Related Requirements - Erata Sheets To 5/7/80 Letter	YES	NO	/ /	/ /	
80	053 Decay Heat Removal Capability	NO		/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====	=====	=====	=====	=====	=====	=====
80	054 IEB 80-14 Degradation Of Scram Discharge Volume Capability	NO		/ /	/ /	
80	055 IEB 80-15 Possible Loss Of Hotline With Loss Of Off-Site Power	YES	NO	/ /	/ /	SEE RESPONSE TO IEB 80-15
80	056 Commission Memorandum And Order On Equipment Qualification	YES	NO	/ /	/ /	
80	057 Further Commission Guidance For Power Reactor Operating Licenses NUREG-0660 And NUREG-0694	YES	NO	/ /	/ /	
80	058 IEB 80-16 Potential Misapplication Of Rosemount Inc. Models 1151/1152 Pressure Transmitters With "A" Or "D" Output Codes	YES	NO	/ /	/ /	
80	059 Transmittal Of Federal Register Notice RE Regional Meetings To Discuss Environmental Qualification Of Elec. Equipment	YES	NO	/ /	/ /	
80	060 Request For Information Regarding Evacuation Times	YES	YES	02/01/80	02/01/80	
80	061 TMI-2 Lessons Learned	NO		/ /	/ /	
80	062 TMI-2 Lessons Learned	YES	YES	09/17/80	04/13/81	
80	063 IEB 80-17 Failure Of Control Rods To Insert During A Scram At A BWR	YES	NO	/ /	/ /	
80	064 Scram Discharge Volume Designs	YES	YES	01/30/81	05/13/81	
80	065 Request For Estimated Construction Completion And Fuel Load Schedules	YES	NO	/ /	/ /	
80	066 IEB 80-17 Supplement 1 Failure Of Control Rods To Insert During A Scram At A BWR	YES	YES	09/02/80	12/01/80	
80	067 Scram Discharge Volume	YES	NO	/ /	/ /	
80	068 IEB 80-17 Supplement 2 Failures Revealed By Testing Subsequent To Failure Of Control Rods To Insert During A Scram At A	YES	YES	07/25/80	07/25/80	
80	069 IEB 80-18 Maintenance Of Adequate Minimum Flow Through Centrifugal Charging Pumps Following Secondary Side Helb	NO		/ /	/ /	
80	070 IEB 80-19 Failures Of Mercury-Wetted Matrix Relays In RPS Of Operating Nuclear Power Plants Designed By GE	YES	NO	/ /	/ /	
80	071 IEB 80-20 Failures Of Westinghouse Type W-2 Spring Return To Neutral Control Switches	YES	NO	/ /	/ /	
80	072 Interim Criteria For Shift Staffing	YES	NO	/ /	/ /	
80	073 "Functional Criteria For Emergency Response Facilities", NUREG-0696	YES	NO	/ /	/ /	
80	074 Notice Of Forthcoming Meeting With Representatives Of EPRI To Discuss Program For Resolution Of USI A-12, Fracture Tough	YES	NO	/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
80	075 Lessons Learned Tech Specs	YES	NO	/ /	/ /	
80	076 Notice Of Forthcoming Meeting With GE To Discussed Proposed BWR Feedwater Nozzle Leakage Detection System	YES	NO	/ /	/ /	
80	077 Refueling Water Level	NO		/ /	/ /	
80	078 Mark I Containment Long-Term Program	YES	NO	/ /	/ /	
80	079 IEB 80-17 Supplement 3 Failures Revealed By Testing Subsequent To Failure Of Control Rods To Insert During A Scram At A	YES	YES	08/29/80	08/29/80	
80	080 Preliminary Clarification Of TMI Action Plan Requirements	YES	NO	/ /	/ /	
80	081 Preliminary Clarification Of TMI Action Plan Requirements - Addendum To 9/5/80 Letter	YES	NO	/ /	/ /	
80	082 IEB 79-01b Supplement 2 Environmental Qualification Of Class 1E Equipment	YES	NO	/ /	/ /	SEE NRC BL79-01B SUPPLEMENT 2
80	083 Environmental Qualification Of Safety-Related Equipment	YES	NO	/ /	/ /	
80	084 BWR Scram System	YES	YES	07/10/81	07/10/81	
80	085 Implementation Of Guidance From USI A-12 "Potential For LOW Fracture Toughness And Lamellar Tearing On Component Support	YES	NO	/ /	/ /	
80	086 Notice Of Meeting To Discuss Final Resolution Of USI A-12	YES	NO	/ /	/ /	
80	087 Notice Of Meeting To Discuss Status Of EPRI-Proposed Resolution Of The USI A-12 Fracture Toughness Issue	YES	NO	/ /	/ /	
80	088 Seismic Qualification Of Auxiliary Feedwater Systems	NO		/ /	/ /	
80	089 IEB 79-01b Supplement 3 Environmental Qualification Of Class 1E Equipment	YES	NO	/ /	/ /	SAVED AS NRC BL79-01B SUPPLEMENT 3
80	090 Post TMI Requirements, NUREG-0737	YES	YES	02/01/82	11/01/90	SEE GLs 82-85, 82-10, AND 89-06
80	091 Odyn Code Calculation	YES	NO	/ /	/ /	
80	092 IEB 80-21 Valve Yokes Supplied By Malcolm Foundry Company, Inc.	YES	NO	/ /	/ /	
80	093 Emergency Preparedness	NO		/ /	/ /	
80	094 Emergency Plan	YES	NO	/ /	/ /	
80	095 Generic Activity A-10	YES	YES	09/08/81	04/01/86	
80	096 Fire Protection	YES	NO	/ /	/ /	
80	097 IEB 80-23 Failures Of Solenoid Valves Manufactured By Valcor Engineering Corporation	YES	NO	/ /	/ /	
80	098 IEB 80-24 Prevention Of Damage Due To Water Leakage Inside Containment	YES	NO	/ /	/ /	
80	099 Technical Specification Revisions For Snubber Surveillance	YES	YES	03/13/85	09/01/85	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
80	100 Appendix R to 10 CFR 50 Regarding Fire Protection-Federal Register Notice	YES	NO	/ /	/ /	
80	101 Inservice Inspection Programs	YES	NO	/ /	/ /	
80	102 Commission Memorandum And Order Of May 23, 1980 (Referencing IEB 79-01b Supplement 2 - q.2 & 3 - Sept 30, 1980)	NO		/ /	/ /	
80	103 Fire Protection - Revised Federal Register Notice	YES	NO	/ /	/ /	
80	104 Orders On Environmental Qualification Of Safety Related Electrical Equipment	YES	NO	/ /	/ /	
80	105 Implementation Of Guidance For USI A-12, Potential For Low Fracture Toughness And Lamellar Tearing On Component Supports	YES	NO	/ /	/ /	
80	106 Report On ECCS Cladding Models, NUREG-0630	NO		/ /	/ /	
80	107 BWR Scram Discharge System	YES	NO	/ /	/ /	
80	108 Emergency Planning	YES	NO	/ /	/ /	
80	109 Guidelines For SEP Soil Structure Interaction Reviews	YES	NO	/ /	/ /	
80	110 Periodic Updating Of FSARS	YES	NO	/ /	/ /	
80	111 IEB 80-17 Supplement 4, Failure Of Control Rods To Insert During A Scram At A BWR	YES	NO	/ /	/ /	
80	112 IEB 80-25 Operating Problems With Target Rock Safety Relief Valves	YES	NO	/ /	/ /	
80	113 Control Of Heavy Loads	YES	YES	07/26/84	03/05/85	
** YEAR 81						
81	01 Qualification of Inspection, Examination, Testing and Audit Personnel	YES	YES	07/30/81	07/31/81	
81	02 Analysis, Conclusions and Recommendations Concerning Operator Licensing	YES	NO	/ /	/ /	
81	03 Implementation of NUREG-0313, Technical Report on Material Selection & Processing GL For BWR Coolant Press Boundary Piping	YES	YES	07/06/81	07/07/81	SEE GL 88-01
81	04 Emergency Procedures And Training For Station Blackout Events	YES	YES	06/02/81	12/ /81	LICENSEE UNCERTAIN OF THE EXACT DAY OF IMPLEMENTATION
81	05 Information Regarding The Program For Environmental Qualification Of Safety-Related Electrical Equipment	YES	NO	/ /	/ /	
81	06 Periodic Updating of Final Safety Analysis Reports (FSARS)	YES	NO	/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====	=====	=====	=====	=====	=====	=====
81	07 Control of Heavy Loads	YES	NO	/ /	/ /	
81	08 Odyn Code	YES	NO	/ /	/ /	
81	09 BWR Scram Discharge System	YES	NO	/ /	/ /	
81	10 Post-TMI Requirements For The Emergency Operations Facility	YES	YES	11/29/82	03/01/83	
81	11 BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking (NUREG-0619)	YES	NO	/ /	/ /	
81	12 Fire Protection Rule (45 FR 76602, 11/19/80)	YES	YES	10/22/84	04/01/85	
81	13 SER For GEXL Correlation For 8X8R Fuel Reload Applications For Appendix D Submittals of The GE Topical Report	YES	NO	/ /	/ /	
81	14 Seismic Qualifications for Auxiliary Feedwater Systems	NO		/ /	/ /	
81	15 Environmental Qualification of Class 1E Electrical Equipment-Clarification of Staffs Handling of Proprietary Information	YES	NO	/ /	/ /	
81	16 NUREG-0737 Item I.C.1 SER on Abnormal Transient Operating Guidelines (ATOG)	NO		/ /	/ /	
81	17 Functional Criteria for Emergency Response Facilities	YES	NO	/ /	/ /	
81	18 BWR Scram Discharge System-Clarification of Diverse Instrumentation Requirements	YES	NO	/ /	/ /	
81	19 Thermal Shock to Reactor Pressure Vessels	NO		/ /	/ /	
81	20 Safety Concerns Associated With Pipe Breaks in The BWR Scram System	YES	YES	12/31/81	01/03/86	
81	21 Natural Circulation Cooldown	NO		/ /	/ /	
81	22 Engineering Evaluation of The H.B. Robinson Reactor Coolant System Leak on 1/29/81	YES	NO	/ /	/ /	
81	23 INPO Plant Specific Evaluation Reports	YES	NO	/ /	/ /	
81	23A INPO Evaluation Reports	YES	NO	/ /	/ /	
81	24 Multi-Plant Issue B-56 Control Rods Fail To Fully Insert	YES	NO	/ /	/ /	
81	25 Change in Implementing Schedule For Submission and Evaluation of Upgraded Emergency Plans	YES	NO	/ /	/ /	
81	26 Licensing Requirements for Pending Construction Permit and Manufacturing License Applications	YES	NO	/ /	/ /	
81	27 Privacy and Proprietary Material in Emergency Plans	YES	NO	/ /	/ /	
81	28 Steam Generator Overfill	NO		/ /	/ /	
81	29 Simulator Examinations	YES	NO	02/23/82	/ /	LICENSEE SUBMITTED RESPONSE

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====	=====	=====	=====	=====	=====	=====
81	30 Safety Concerns Associated With Pipe Breaks in the BWR Scram System	YES	NO	/ /	/ /	
81	31 Small Break LOCA Confirmatory Integral Systems Experiments for B&W Designed Plants	NO		/ /	/ /	
81	32 NUREG-0737, Item II.K.3.44-Evaluation of Anticipated Transients Combined With Single Failure	YES	YES	11/12/81	11/12/81	12/15/82 NRC EVALUATION
81	33 Technical Specification for Station Batteries Multiplant Action	NO		/ /	/ /	
81	34 Safety Concerns Associated With Pipe Breaks in the BWR Scram System	YES	NO	/ /	/ /	
81	35 Safety Concerns Associated With Pipe Breaks in the BWR Scram System	YES	NO	/ /	/ /	
81	36 Revised Schedule for Completion of TMI Action Plan Item II.D.1, Relief and Safety Valve Testing	YES	NO	/ /	/ /	
81	37 ODYN Code Reanalysis Requirements	YES	YES	01/25/82	01/25/82	
81	38 Storage of Low Level Radioactive Wastes at Power Reactor Sites	YES	NO	/ /	/ /	
81	39 NRC Volume Reduction Policy	YES	NO	/ /	/ /	
81	40 Qualifications of Reactor Operators	YES	NO	/ /	/ /	
** YEAR 82						
82	01 New Applications Survey	YES	NO	/ /	/ /	
82	02 Commission Policy on Overtime	YES	YES	02/01/82	02/01/82	SEE GLs 82-12 AND 83-02
82	03 High Burnup MAPLHGR Limits	YES	NO	/ /	/ /	
82	04 Use of INPO See-in Program	YES	NO	/ /	/ /	
82	05 Post-TMI Requirements	YES	YES	04/16/82	04/16/82	CONFIRM ORDER 08/31/82 TEL
82	06 RTD Response Time Determination	NO		/ /	/ /	
82	07 Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	NO		/ /	/ /	
82	08 Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	NO		/ /	/ /	
82	09 Environmental Qualification of Safety Related Electrical Equipment	YES	NO	/ /	/ /	
82	10 Post-TMI Requirements	YES	YES	06/07/82	06/07/82	
82	11 Transmittal of NUREG-0916 Relative to the Restart of R.E. Ginna Nuclear Power Plant	NO		/ /	/ /	
82	12 Nuclear Power Plant Staff Working Hours	YES	YES	04/25/83	06/07/82	SEE GL 83-02, 04/25/83 RESPONSE
82	13 Reactor Operator and Senior Reactor Operator Examinations	YES	NO	/ /	/ /	
82	14 Submittal of Documents to the NRC	YES	NO	/ /	/ /	
82	15 Nuclear Plant Staff Working Hours	NO		/ /	/ /	
82	16 NUREG-0737 Technical Specifications	NO		/ /	/ /	
82	17 Inconsistency of Requirements Between 50.54(T) and 50.15	YES	YES	04/21/82	03/20/84	TECHNICAL SPECIFICATION 03/20/84

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
*****	*****	*****	*****	*****	*****	*****
82 18	Reactor Operator and Senior Reactor Operator Requalification Examinations	YES	NO	/ /	/ /	
82 19	Submittal of Copies of Documentation to NRC-Copy Requirements for Emergency Plans and Physical Security Plans	NO		/ /	/ /	
82 20	Guidance for Implementing the Standard Review Plan Rule	YES	NO	/ /	/ /	
82 21	Fire Protection Audits	YES	NO	/ /	/ /	
82 22	Congressional Request for Information Concerning Steam Generator Tube Integrity	NO		/ /	/ /	
82 23	Inconsistency Between Requirements of 10CFR 73.40(D) and Standard Tech Specs For Performing Audits of Safeguards Contin	YES	YES	04/21/83	03/20/84	
82 24	Safety Relief Valve Quencher Loads: BWR MARK II and III Containments	YES	NO	/ /	/ /	
82 25	Integrated IAEA Exercise for Physical Inventory at LWRS	YES	NO	/ /	/ /	
82 26	NUREG-0744, REV. 1 - Pressure Vessel Material Fracture Toughness	YES	NO	/ /	/ /	
82 27	Transmittal of NUREG-0763 - Guidelines For Confirmatory In-Plant Tests of Safety-Relief Valve Discharge for BWR Plants	YES	NO	/ /	/ /	
82 28	Inadequate Core Cooling Instrumentation System	NO		/ /	/ /	
82 29	NUREG/CR-2980	NO		/ /	/ /	
82 30	Filings Related to 10 CFR 50 Production and Utilization Facilities	YES	NO	/ /	/ /	
82 31	NUREG-0737 Technical Specifications	NO		/ /	/ /	
82 32	Draft Steam Generator Report (SAI)	NO		/ /	/ /	
82 33	Supplement 1 to NUREG-0737 - Emergency Response Capabilities	YES	YES	12/06/85	11/19/86	
82 34	SECY 82-111 Modification of Vacuum Breakers & Mark I Containments	NO		/ /	/ /	
82 35		NO		/ /	/ /	
82 36	Assignment of Authority to Regions for Nonpower Reactors Amendments and Licensing Activities	NO		/ /	/ /	
82 37	NUREG-0737 Tech Specs	NO		/ /	/ /	
82 38	Meeting to Discuss Developments for Operator Licensing Examinations	YES	NO	/ /	/ /	
82 39	Problems With Submittals of Subsequent Information of CURT 73.21 For Licensing Reviews	YES	NO	/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
*****	*****	*****	*****	*****	*****	*****
** YEAR 83						
83 01	Operator Licensing Examination Site Visit	YES	NO	/ /	/ /	
83 02	NUREG-0737 Technical Specifications	YES	YES	01/31/84	01/31/84	
83 03	Regulatory Guide 1.150, "Ultrasonic Testing of Reactor Vessel Welds During Pre-Service and Inservice Examinations"	NO		/ /	/ /	
83 04	Regional Workshops Regarding Supplement 1 to NUREG-0737, Requirements For Emergency Response Capability	YES	NO	/ /	/ /	
83 05	Safety Evaluation of "Emergency Procedure Guidelines, Revision 2," June 1982	YES	NO	/ /	/ /	
83 06	Certificates and Revised Format For Reactor Operator and Senior Reactor Operator Licenses	YES	NO	/ /	/ /	
83 07	The Nuclear Waste Policy Act of 1982	YES	NO	/ /	/ /	
83 08	Modification of Vacuum Breakers on Mark I Containments	YES	YES	03/07/86	03/07/86	
83 09	Review of Combustion Engineering Owners' Group Emergency Procedures Guideline Program	NO		/ /	/ /	
83 10		NO		/ /	/ /	
83 10A	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NO		/ /	/ /	
83 10B	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NO		/ /	/ /	
83 10C	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NO		/ /	/ /	
83 10D	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NO		/ /	/ /	
83 10E	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NO		/ /	/ /	
83 10F	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NO		/ /	/ /	
83 11	Licensee Qualification for Performing Safety Analyses in Support of Licensing Actions	YES	NO	/ /	/ /	
83 12	Issuance of NRC FORM 398 - Personal Qualifications Statement - Licensee	YES	NO	/ /	/ /	
83 12A	Issuance of NRC FORM 398 - Personal Qualifications Statement - Licensee	NO		/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====	=====	=====	=====	=====	=====	=====
83	13 Clarification of Surveill. Req's for HEPA Filters and Charcoal Absorber Units In STD. Tech Specs on ESF Cleanup Systems	NO	NO	/ /	/ /	
83	14 Definition of "Key Maintenance Personnel," (Clarification of Generic Letter 82-12)	YES	NO	/ /	/ /	
83	15 Implement. of Reg. Guide 1.150, "Ultrasonic Testing of RX Vessel Welds During Preservice & Inservice Examinations, REV.1	YES	NO	/ /	/ /	
83	16 Transmittal of NUREG-0977 Relative to the ATWS Events at Salem Generating Station, Unit NO. 1	YES	NO	/ /	/ /	
83	16A Transmittal of NUREG-0977 Relative to the ATWS Events at Salem Generating Station, Unit NO. 1	YES	NO	/ /	/ /	
83	17 Integrity of Requalification Examinations for Renewal of Reactor Operator and Senior Reactor Operator Licenses	YES	NO	/ /	/ /	
83	18 NRC Staff Review of the BWR Owners' Group (BWROG) Control Room Survey Program	YES	YES	09/30/83	09/30/83	
83	19 New Procedures for Providing Public Notice Concerning Issuance of Amendments To Operating Licenses	YES	NO	/ /	/ /	
83	20 Integrated Scheduling for Implementation of Plant Modifications	YES	NO	/ /	/ /	
83	21 Clarification of Access Control Procedures for Law Enforcement Visits	YES	NO	/ /	/ /	
83	22 Safety Evaluation of "Emergency Response Guidelines"	YES	NO	/ /	/ /	
83	23 Safety Evaluation of "Emergency Procedure Guidelines"	NO		/ /	/ /	
83	24 TMI Task Action Plan Item I.G.1, "Special Low Power Testing and Training," Recommendations for BWRS	NO	NO	/ /	/ /	
83	25 Issuance of NRC Form 398 Personal Qualifications Statement - Licensee	NO		/ /	/ /	
83	26 Clarification Of Surveillance Requirements For Diesel Fuel Impurity Level Tests	NO	NO	/ /	/ /	
83	27 Surveillance Intervals in Standard Technical Specifications	YES	NO	/ /	/ /	
83	28 Required Actions Based on Generic Implications of Salem ATWS Events	YES	YES	03/01/89	06/01/89	
83	29 This Generic Communication was not issued	NO		/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
*****	*****	*****	*****	*****	*****	*****
83	30 Deletion of STD. Tech Spec Surveillance Requirement 4.8.1.1.2.d.6 For Diesel Generator Testing	NO	NO	/ /	/ /	
83	31 Safety Evaluation of "Abnormal Transient Operating Guidelines"	NO		/ /	/ /	
83	32 NRC Staff Recommendations Regarding Operator Action for Reactor Trip and ATWS	YES	NO	/ /	/ /	
83	33 NRC Positions on Certain Requirements of Appendix R to 10 CFR 50	YES	NO	/ /	/ /	
83	34 Important to Safety	NO		/ /	/ /	
83	35 Clarification of TMI Action Plan Item II.K.3.31	YES	NO	/ /	/ /	
83	36 NUREG-0737 Technical Specifications	YES	YES	08/ /86	12/ /86	LICENSEE UNCERTAIN OF EXACT DAYS
83	37 NUREG-0737 Technical Specifications	YES	NO	/ /	/ /	
83	38 NUREG-0965, "NRC Inventory of Dams"	YES	NO	/ /	/ /	
83	39 Voluntary Survey of Licensed Operators	YES	NO	/ /	/ /	
83	40 Operator Licensing Examination	YES	NO	/ /	/ /	
83	41 Fast Cold Starts of Diesel Generators	YES	NO	/ /	/ /	
83	42 Clarification to GL 81-07 Regarding Response to NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants"	YES	NO	/ /	/ /	
83	43 Reporting Requirements of 10 CFR 50, Sections 50.72 and 50.73, and Standard Technical Specifications	YES	YES	01/31/84	10/29/84	
83	44 Availability of NUREG-1021, "Operator Licensing Examiner Standards"	NO		/ /	/ /	
** YEAR 84						
84	01 NRC Use Of The Terms "Important To Safety" and "Safety Related"	YES	NO	/ /	/ /	
84	02 Notice of Meeting Regarding Facility Staffing	YES	NO	/ /	/ /	
84	03 Availability of NUREG-0933, "A Prioritization of Generic Safety Issues"	YES	NO	/ /	/ /	
84	04 Safety Evaluation of W Topical Reports On Elimination of Postulated Pipe Breaks in PWR Primary Main Loops	NO		/ /	/ /	
84	05 Change to NUREG-1021, "Operator Licensing Examiner Standards"	YES	NO	/ /	/ /	
84	06 Operator and Senior Operator License Examination Criteria For Passing Grade	NO		/ /	/ /	
84	07 Procedural Guidance for Pipe Replacement at BWRs	NO	NO	/ /	/ /	

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NRC GENERIC LETTERS

GL-N	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
*****	*****	*****	*****	*****	*****	*****
84	08 Interim Procedures for NRC Management of Plant-Specific Backfitting	YES	NO	/ /	/ /	
84	09 Recombiner Capability Requirements of 10 CFR 50.44(c)(3)(ii)	YES	YES	07/31/84	03/01/84	
84	10 Administration of Operating Tests Prior to Initial Criticality	YES	NO	/ /	/ /	
84	11 Inspections of BWR Stainless Steel Piping	YES	YES	10/18/88	07/09/89	
84	12 Compliance W 10CFR61 and Implementation of Rad Effluent Tech Specs, Attendant Process Control Program	YES	NO	/ /	/ /	
84	13 Technical Specification for Snubbers	YES	YES	07/13/85	09/01/85	
84	14 Replacement and Requalification Training Program	YES	YES	10/28/86	06/30/87	
84	15 Proposed Staff Actions to Improve and Maintain Diesel Generator Reliability	YES	YES	10/13/84	10/13/84	
84	16 Adequacy of On-Shift Operating Experience for Near Term Operating License Applicants	YES	NO	/ /	/ /	
84	17 Annual Meeting to Discuss Recent Developments Regarding Operator Training, Qualifications, and Examinations	YES	NO	/ /	/ /	
84	18 Filing of Applications for Licenses and Amendments	NO		/ /	/ /	
84	19 Availability of Supplement 1 to NUREG-0933, "A Prioritization of Generic Safety Issues"	YES	NO	/ /	/ /	
84	20 Scheduling Guidance for Licensee Submittals of Reloads That Involve Unreviewed Safety Questions	YES	NO	/ /	/ /	
84	21 Long Term Low Power Operation in Pressurized Water Reactors	NO		/ /	/ /	
84	22 10 CFR 20.408 Termination Reports - Format	NO		/ /	/ /	
84	23 Reactor Vessel Water Level Instrumentation in BWRs	YES	YES	03/10/89	03/10/89	
84	24 Cert of Compl to 10CFR50.49, Eq of Electric Equipment Important to Safety for Npps	YES	YES	01/23/85	03/31/85	
** YEAR 85						
85	01 Fire Protection Policy Steering Committee Report	YES	NO	/ /	/ /	
85	02 Staff Recommended Actions ... Regarding Steam Generator Tube Integrity	NO		/ /	/ /	
85	03 Clarification of Equivalent Control Capacity for Standby Liquid Control Systems	YES	NO	/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
*****	*****	*****	*****	*****	*****	*****
85	04 Operating Licensing Examinations	YES	NO	/ /	/ /	LICENSEE SUBMITTED RESPONSE IMPLEMENTED THROUGH REQUIREMENTS OF 10CFR50.62
85	05 Inadvertent Boron Dilution Events	NO		/ /	/ /	
85	06 Quality Assurance Guidance for ATWS Equipment That Is Not Safety-Related	NO	NO	10/10/85	/ /	
85	07 Implementation of Integrated Schedules for Plant Modifications	YES	NO	/ /	/ /	
85	08 10 CFR 20.408 Termination Reports - Format	YES	NO	/ /	/ /	
85	09 Technical Specifications For Generic Letter 83-28, Item 4.3	NO		/ /	/ /	
85	10 Technical Specification For Generic Letter 83-28, Items 4.3 and 4.4	NO		/ /	/ /	
85	11 Completion of Phase II of "Control Of Heavy Loads At Nuclear Power Plants" NUREG-0612	YES	NO	/ /	/ /	
85	12 Implementation Of TMI Action Item II.K.3.5, "Automatic Trip Of Reactor Coolant Pumps	YES	NO	/ /	/ /	
85	13 Transmittal Of NUREG-1154 Regarding The Davis-Besse Loss Of Main And Auxiliary Feedwater Event	YES	NO	/ /	/ /	
85	14 Commercial Storage At Power Reactor Sites Of Low Level Radioactive Waste Not Generated By The Utility	YES	NO	/ /	/ /	
85	15 Information On Deadlines For 10CFR50.49, "EO Of Electric Equipment Important To Safety At Npps"	YES	NO	/ /	/ /	
85	16 High Boron Concentrations	YES	NO	/ /	/ /	
85	17 Availability Of Supplements 2 and 3 Yo NUREG-0933, "A Prioritization Of Generic Safety Issues"	YES	NO	/ /	/ /	
85	18 Operator Licensing Examinations	YES	NO	/ /	/ /	
85	19 Reporting Requirements On Primary Coolant Iodine Spikes	NO	NO	/ /	/ /	
85	20 Resolution Of Generic Issue 69: High Pressure Injection/Make-up Nozzle Cracking In Babcock And Wilcox Plants	NO		/ /	/ /	
85	21 This Generic Communication was not issued	NO		/ /	/ /	
85	22 Potential For Loss Of Post-LOCA Recirculation Capability Due To Insulation Debris Blockage	YES	NO	/ /	/ /	
** YEAR 86						
86	01 Safety Concerns Associated With Pipe Breaks In The BWR Scram System	YES	NO	/ /	/ /	
86	02 Technical Resolution of Generic Issue B-19 - Thermal Hydraulic Stability	YES	NO	/ /	/ /	



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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
86	03 Applications For License Amendments	YES	NO	/ /	/ /	
86	04 Policy Statement On Engineering Expertise On Shift	YES	NO	/ /	/ /	
86	05 Implementation Of TMI Action Item II.K.3.5, "Automatic Trip Of Reactor Coolant Pumps"	NO		/ /	/ /	
86	06 Implementation Of TMI Action Item II.K.3.5, "Automatic Trip Of Reactor Coolant Pumps"	NO		/ /	/ /	
86	07 Transmittal of NUREG-1190 Regarding The San Onofre Unit 1 Loss of Power And Water Hammer Event	YES	NO	/ /	/ /	
86	08 Availability of Supplement 4 to NUREG-0933, "A Prioritization of Generic Safety Issues"	YES	NO	/ /	/ /	
86	09 Technical Resolution of Generic Issue B-59, (n-1) Loop Operation in BWRs and PWRs	YES	NO	/ /	/ /	
86	10 Implementation of Fire Protection Requirements	YES	YES	06/01/88	06/01/88	
86	11 Distribution of Products Irradiated in Research Reactors	NO		/ /	/ /	
86	12 Criteria for Unique Purpose Exemption From Conversion From The Use of Heu Fuel	NO		/ /	/ /	
86	13 Potential Inconsistency Between Plant Safety Analyses and Technical Specifications	NO		/ /	/ /	
86	14 Operator Licensing Examinations	YES	NO	/ /	/ /	
86	15 Info ... Compliance W 10CFR50.49 "Eq of Electric Equipment Important to Safety For Nuclear Power Plants"	YES	NO	/ /	/ /	
86	16 Westinghouse ECCS Evaluation Models	NO		/ /	/ /	
86	17 Availability of NUREG-1169, "Technical Findings Related to Generic Issue C-8, BWR MSIC Leakage And Treatment Methods"	YES	NO	/ /	/ /	
** YEAR 87						
87	01 Public Availability Of The NRC Operator Licensing Examination Question Bank	YES	NO	/ /	/ /	
87	02 Verification of Seismic Adequacy of Mechanical and Electrical Equipment In Operating Reactors (USI A-46)	YES	YES	11/30/87	/ /	SEE 13/19/90 NRC LTR OPEN IMPLEMENTATION DATE
87	03 Verification of Seismic Adequacy of Mechanical and Electrical Equipment In Operating Reactors (USI A-46)	YES	NO	/ /	/ /	
87	04 Temporary Exemption From Provisions Of The FBI Criminal History Rule For Temporary Workers	YES	NO	/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
87	05 Rqst Add'l Info Assessment ... Degradation Of Mark I Drywells	YES	YES	06/26/90	06/26/90	
87	06 Periodic Verification Of Leak Tight Integrity Of Pressure Isolation Valves	YES	YES	04/26/87	06/27/87	
87	07 Information Transmittal of Final Rulemaking For Revisions To Operator Licensing - 10 CFR 55 And Confirming Amendments	YES	NO	/ /	/ /	
87	08 Implementation of 10 CFR 73.55 Miscellaneous Amendments and Search Requirements	YES	NO	/ /	/ /	
87	09 Sections 3.0 And 4.0 of Standard Tech Specs on Limiting Conditions For Operation And Surveillance Requirements	YES	NO	/ /	/ /	
87	10 Implementation of 10 CFR 73.57, Requirements For FBI Criminal History Checks	YES	NO	/ /	/ /	
87	11 Relaxation in Arbitrary Intermediate Pipe Rupture Requirements	YES	NO	/ /	/ /	
87	12 Loss of Residual Heat Removal While The Reactor Coolant System is Partially Filled	NO		/ /	/ /	
87	13 Integrity of Requalification Examinations At Non-Power Reactors	NO		/ /	/ /	
87	14 Operator Licensing Examinations	YES	NO	/ /	/ /	
87	15 Policy Statement On Deferred Plants	YES	NO	/ /	/ /	
87	16 Transmittal of NUREG-1262, "Answers To Questions On Implementation Of 10 CFR 55 On Operators' Licenses"	YES	NO	/ /	/ /	
** YEAR 88						
88	01 NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping	YES	YES	11/02/90	09/04/90	
88	02 Integrated Safety Assessment Program II (ISAP II)	YES	NO	/ /	/ /	
88	03 Resolution of Generic Safety Issue 93, "Steam Binding of Auxiliary Feedwater Pumps"	NO		/ /	/ /	
88	04 Distribution of Gems Irradiated In Research Reactors	NO		/ /	/ /	
88	05 Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components In PWR plants	NO		/ /	/ /	
88	06 Removal Of Organization Charts From Technical Specification	YES	NO	/ /	/ /	
88	07 Modified Enforcement Policy Relating to 10 CFR 50.49, "Environmental Qualification Of Electrical Equipment Important To	YES	NO	/ /	/ /	



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GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====	=====	=====	=====	=====	=====	=====
88	08 Mail Sent or Delivered To The Office Of Nuclear Reactor Regulation	YES	NO	/ /	/ /	
88	09 Pilot Testing of Fundamentals Examination	YES	NO	/ /	/ /	
88	10 Purchase of GSA Approved Security Containers	YES	NO	/ /	/ /	
88	11 NRC Position on Radiation Embrittlement Of Reactor Vessel Materials And Its Impact On Plant Operations	YES	YES	11/22/88	/ /	A T.S. CHANGE TO RESOLVE PLANNED REQUIRED FOR 1992 RO IMPEM. DATE OPEN
88	12 Removal of Fire Protection Requirements From Technical Specification	YES	NO	/ /	/ /	
88	13 Operator Licensing Examinations	YES	NO	/ /	/ /	
88	14 Instrument Air Supply System Problems Affecting Safety-Related Equipment	YES	YES	04/10/89	/ /	LICENSEE 4/1/89 RESPONSE PROVIDED SCHEDULE FOR COMPLETION OF WORK BY 1992 RO
88	15 Electric Power Systems - Inadequate Control Over Design Process	YES	NO	/ /	/ /	
88	16 Removal Of Cycle-Specific Parameter Limits From Technical Specifications	YES	NO	/ /	/ /	
88	17 Loss of Decay Heat Removal	NO		/ /	/ /	
88	18 Plant Record Storage On Optical Disks	YES	NO	/ /	/ /	
88	19 Use Of Deadly Force By Licensee Guards To Prevent Theft Of Special Nuclear Material	NO	NO	/ /	/ /	INCORPORATED IN SECURITY PLAN, NO IMPLEMENTATION DATE REQUIRED
88	20 Individual Plant Examination For Severe Accident Vulnerabilities	YES	YES	11/03/89	/ /	IPE/EPI - IMPLEMENTATION TIED TO DESIGN BASIS RECONSTITUTION PROGRAM
88	20 Initiation Of The Individual Plant Examination For Severe Accident Vulnerabilities - 10 CFR 50.54	YES	YES	11/03/89	/ /	IPE/EPI - IMPLEMENTATION TIED TO DESIGN BASIS RECONSTITUTION PROGRAM
88	20 Accident Management Strategies for Consideration in the Individual Plant Examination Process	YES	YES	11/03/89	/ /	IPE/EPI - IMPLEMENTATION TIED TO DESIGN BASIS RECONSTITUTION PROGRAM
88	20 Completion of Containment Performance Improvement Program and Forwarding of Insights for Use in the IPE for SAV	YES	YES	11/03/89	/ /	IPE/EPI - IMPLEMENTATION TIED TO DESIGN BASIS RECONSTITUTION PROGRAM
** YEAR 89						
89	01 Implementation Of Programmatic And Procedural Controls for Radiological Effluent Technical Specifications	YES	NO	/ /	/ /	
89	02 Actions To Improve The Detection Of Counterfeit And Fraudulently Marketed Products	YES	NO	/ /	/ /	
89	03 Operator Licensing National Examination Schedule	YES	NO	/ /	/ /	
89	04 Guidance On Developing Acceptable Inservice Testing Programs	YES	NO	/ /	/ /	

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NRC GENERIC LETTERS

GL-#		TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
=====		=====	=====	=====	=====	=====	=====
89	05	Pilot Testing Of The Fundamentals Examination	YES	NO	/ /	/ /	
89	06	Task Action Item I.D.2 - Safety Parameter Display System - 10 CFR 50.54 (f)	YES	YES	07/11/89	07/11/89	
89	07	Power Reactor Safeguards Contingency Planning For Surface Vehicle Bombs	YES	YES	11/07/89	11/07/89	
89	08	Erosion/Corrosion-Induced Pipe Wall Thinking	YES	YES	07/05/89	07/05/89	
89	09	ASME Section III Component Replacements	YES	NO	/ /	/ /	
89	10	Safety-Related Motor-Operated Valve Testing And Surveillance	YES	YES	10/10/90	/ /	NMPC DEVELOPING PROGRAM PLAN TO ADDRESS RECOMMEND OF GL IMPLEM. DATE OPEN
89	11	Resolution Of Generic Issue 101 "Boiling Water Reactor Water Level Redundancy"	YES	NO	/ /	/ /	
89	12	Operator Licensing Examination	YES	NO	/ /	/ /	
89	13	Service Water System Problems Affecting Safety-Related Equipment	YES	YES	12/07/90	/ /	ACTIONS REQUIRED TO BE COMPLETED BY 1992 REFUELING OUTAGE IMPLEM. DATE OPEN
89	14	Line-Item Improvements In Technical Specifications - Removal Of 3.25 Limit On Extending Surveillance Intervals	YES	NO	11/30/90	01/11/91	LICENSEE SUBMITTED RESPONSE
89	15	Emergency Response Data System	YES	NO	/ /	/ /	
89	16	Installation Of A Hardened Wetwell Vent	YES	YES	10/15/90	/ /	ACTIONS TO BE COMPLETED IN 1992 REFUELING OUTAGE IMPLEMENTATION DATE OPEN
89	17	Planned Administrative Changes To The NRC Operator Licensing Written Examination Process	NO		/ /	/ /	
89	18	Resolution Of Unresolved Safety Issues A-17, "Systems Interactions In Nuclear Power Plants"	YES	NO	/ /	/ /	
89	19	Request For Actions Related To Resolution Of Unresolved Safety Issue A-47, "....," Pursuant To 10 CFR 50.54(f)	YES	YES	05/04/90	05/04/90	
89	20	Protected Area Long-Term Housekeeping	NO		/ /	/ /	
89	21	... Status Of... Unresolved Safety Issue Requirements	YES	YES	11/25/89	08/28/89	
89	22	Potential For Increased Roof Loads...Due To Recent Change In Probable Maximum Precipitation Criteria...	YES	NO	/ /	/ /	
89	23	NRC Staff Responses To Questions Pertaining To Implementation Of 10 CFR Part 26	YES	NO	/ /	/ /	

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NRC GENERIC LETTERS

GL-#	TITLE	APPLICABLE TO NMP-1	RESPONSE REQUIRED	RESPONSE DATE	IMPLEMENTATION DATE	COMMENTS
** YEAR 90						
90 01	Request for Voluntary Participation in NRC Regulatory Impact Survey	YES	NO	/ /	/ /	
90 02	Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications	YES	NO	/ /	/ /	
90 03	Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2 "Vendor Interface for Safety-Related Components"	YES	YES	09/25/90	/ /	FORMAL VENDOR INTERFACE PROGRAM BEING DEVELOPED DURING 1991 IMPLEM. OPEN
90 04	Request for Information on the Status of Licensee Implementation of GSIs Resolved With Imposition of Requirements or CAs	YES	NO	/ /	/ /	
90 05	Guidance for Performing Temporary Non-Code Repair of ASME Code Class 1, 2, and 3 Piping	YES	NO	/ /	/ /	
90 06	Resolution of Generic Issues 70, "PORV and Block Valve Reliability," and 94, "Additional LTOP Protection for PWRs"	NO		/ /	/ /	
90 07	Operator Licensing National Examination Schedule	YES	NO	/ /	/ /	
90 08	Simulation Facility Exemptions	YES	NO	/ /	/ /	
90 09	Alternative Requirements for Snubber Visual Inspection Intervals and Corrective Actions	YES	NO	/ /	/ /	
** YEAR 91						
91 01	Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens From Technical Specifications	YES	NO	/ /	/ /	

