CELERATED DISTRIBUTION DEMONSTRATION SYSTEM 1 REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS) DOCKET # DOC.DATE: 90/11/21 NOTARIZED: NO ACCESSION NBR:9012120169 FACIL: 50-410 Nine Mile Point Nuclear Station, Unit 2, Niagara Moha 05000410 AUTHOR AFFILIATION AUTH.NAME CONWAY, J. Niagara Mohawk Power Corp. FIRLIT, J.F. Niagara Mohawk Power Corp. R RECIPIENT AFFILIATION RECIP.NAME I SUBJECT: LER 90-020-00:on 901027, ESF actuation due to procedural deficiency.Flow switch procedure revised.W/901121 ltr. D LENCL / Ь DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR SIZE: S TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc. NOTES:

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NINE MILE POINT NUCLEAR STATION/P.O. BOX 32, LYCOMING, N.Y. 13093/TELEPHONE (315) 343-2110

NMP73953

November 21, 1990

United States Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

RE: Docket No. 50-410 LER 90-20

Gentlemen:

In accordance with 10CFR50.73, we hereby submit the following Licensee Event Report:

LER 90-20 Is being submitted in accordance with 10CFR50.73 (a)(2)(iv), "Any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS)".

A telephone notification was made on October 27, 1990, at 0855 hours per 10CFR50.72 part (b)(2)(ii).

This report was completed in the format designated in NUREG-1022, Supplement 2, dated September 1985.

Very truly yours,

Joseph F. Firlit Vice President - Nuclear Generation

ELL

JFF/AC/lmc

ATTACHMENT

12120169

ADOČK OŠ

xc: Thomas T. Martin, Regional Administrator Region I William A. Cook, Sr. Resident Inspector ı · · · . . , ł

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INRC Form 386 (9-83) LICENSEE EVENT REPORT	U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 T (LER) EXPIRES: 8/31/88
FACILITY NAME (1) Nine Mile Point Unit 2	DOCKET NUMBER (2) PAGE (3) 0 15 0 0 0 4 1 1 0 1 0 5
Engineered Safety Feature Actuation Due To Procedural	Deficiency
EVENT DATE (5) LER NUMBER (6) REPORT DATE (7)	OTHER FACILITIES INVOLVED (8) FACILITY NAMES DOCKET NUMBER(S)
MONTH DAY TEAR TEAR TEAR STATE NUMBER STATE OF TEAR	A 0 5 0 0 0 1
1 0 2 7 9 0 9 0 - 0 2 0 - 0 0 1 1 2 1 9 0 N/	ΥΑ ··· 01510101011
OPERATING MODE (9) 5 20 402(1) 20 402(1)	: (Check one or more of the following) (11)
POWER 20.405(a)(1)(i) 50.36(c)(1)	50,73(a)(2)(v) 73,71(c)
LEVEL 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0	50,73(a)(2)(vii) OTHER (Specify in Abstract below and in Text, NRC Form
20,405(s)(1)(iii) 50,73(s)(2)(i)	50,73(e)(2)(viii)(A) 366A) 50,73(e)(2)(viii)(B)
20.405(a)(1)(v) 50.73(a)(2)(iii)	50,73(a)(2)(x)
LICENSEE CONTACT FOR THIS LER (12	2) TELEDHONE NIMBER
John Conway, Manager Technical Support	AREA CODE
com conway, Manager rechircar Support	3 1 5 3 4 9 - 2 6 9 8
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCR	IBED IN THIS REPORT (13)
CAUSE SYSTEM COMPONENT MANUFAC- TURER TO NPRDS	EM COMPONENT MANUFAC- REPORTABLE TURER TO NPROS
SUPPLEMENTAL REPORT EXPECTED (14)	EXPECTED MONTH DAY YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	OATE (15)
ABSTRACT (Limit to 1400 speces, i.e., approximately fifteen single-space typewritten lines) (16)	
On October 27, 1990, at 0549 hours, experienced an actuation of an Engineer Specifically, the Secondary Containment (and an automatic start of the Rea Recirculation Unit Coolers occurred, c Reactor Building Ventilation System (H event, the reactor mode switch was in " was in progress and the core was off-loa	Nine Mile Point Unit 2 red Safety Feature (ESF). (Reactor Building) isolated actor Building Emergency aused by low flow in the VR). At the time of the 'REFUEL", no fuel movement aded.
The root cause is a procedural deficiency calibration of the flow switches did not variations between the redundant fans. excessive flow variation between fans.	y in that the procedure for account for potential flow A contributing cause was
The immediate corrective action was to on ESF actuation, assess the plant for on conditions and restore the HVR Syste Additionally, the Reactor Building above flow switches were recalibrated, other setpoints were verified as correct and t	determine the cause of the other possible initiating em to normal operation. ve/below refuel floor low r applicable flow switch the procedure was revised.

NRC Form 366 (9-83)

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URC FORM 366A (6-89) LICENSEE EVENT F J TEXT CONTINU	U.S. NUCLEAR REGULATORY COMMISSION REPORT (LER)	APPROVED OMB NO. 3150-0104 EXPIRES: 4/30/92 ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS, FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 2055, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.		
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)	
Nine Mile Point Unit 2	0. 5 0 0 0 4 1 0	YEAR SEQUENTIAL REVISION 90 - 01 20 - 00	0 2 OF 0 5	
TEXT (If more space is required, use additional NRC Form 366A's) (17)	•			

I. DESCRIPTION OF EVENT

On October 27, 1990, at 0549 hours, Nine Mile Point Unit 2 experienced an actuation of an Engineered Safety Feature (ESF). Specifically, the Secondary Containment (Reactor Building) isolated and an automatic start of the Reactor Building Emergency Recirculation Unit Coolers occurred, caused by low flow in the Reactor Building Ventilation System (HVR). At the time of the event, the reactor mode switch was in "REFUEL", no fuel movement was in progress and the core was off-loaded.

A low flow signal was detected by the Division II flow switch in the below refuel floor exhaust duct, which initiated, per its design, a secondary containment isolation and an auto start of the reactor building emergency recirculation unit coolers. The Standby Gas Treatment System received an initiation signal, but both trains were in pull-to-lock to prevent operation of the system due to other maintenance activities.

Secondary Containment integrity was not required during this event as the core was off-loaded and fuel was not being moved.

Subsequent to the ESF initiations, control room personnel investigated the cause of the event and found no indication of a high radiation or Loss of Coolant Accident (LOCA) signal (the other signals capable of causing the initiation). The Control Room alarm printer indicated a DIV II below refuel floor exhaust air low flow (HVR FC11) at 05:49:36 and the ventilation panels at the rear of the control room (CEC PNL 870, 871) indicated that the building had isolated and the unit coolers had initiated. When no other trip indications were found, the operators placed the above and below refuel floor low flow override switches in the "override" position, restarted the reactor building ventilation system, and secured the Emergency Recirculation Unit Coolers. The ventilation system started up normally and all four of the low flow switches reset, indicating normal flow. The low flow override switches were left in "override" until the cause of the trip could be determined.

The Reactor Building Ventilation System has four exhaust fans. Exhaust below the refuel floor is controlled by two 70,000 CFM exhaust fans 2HVR-FN2A, 2B, each capable of 100% of the exhaust flow with the other fan remaining in standby. The fans draw air through a common duct. The air flow through the duct is protected against low flow by redundant air flow switches 2HVR*FS37A (Div I) and 2HVR*FS37B (Div II) which initiate secondary containment isolation and other ESF actuations on low flow.

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U.S. NUCLEAR REGULATORY COMMISSION			APPROVED OMB NO. 3150-0104			
•	LICENSEE EVENT REPORT (TEXT CONTINUATION	LER)	EXPIRES: 4/30/92 ESTIMATED BURDEN PER RESPONSE T INFORMATION COLLECTION REQUEST: COMMENTS REGARDING BURDEN ESTIM AND REPORTS MANAGEMENT BRANCH REGULATORY COMMISSION, WASHINGT THE PAPERWORK REDUCTION PROJEC OF MANAGEMENT AND BUDGET, WASHIN	O COMPLY WTH THIS 50.0 HRS. FORWARD ATE TO THE RECORDS (P-530), U.S. NUCLEAR ON, DC 20555, AND TO T (3150-0104), OFFICE NGTON, DC 20503.		
FACILITY NAME (1)		DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)		
Nine Mile I	Point Unit 2	0 5 0 0 0 4 1 0	YEAR SEQUENTIAL REVISION 9 0 0 2 0	0 3 OF 0 5		
TEXT (If more space is re	quired, use additional NRC Form 366A's) (17)	┟╾╍┙╾╸┦╾╌╸╏╼╸╸╸╸╸╸╸╸╸╸╸╸╸		(K,,,,,,		
<u>I.</u> Test disc 1.	DESCRIPTION OF EVENT (Continued of fans and flow switched the following: Fan HVR-FN2B has a flow (i.e. outside the 10)	ont.) itches for the be w of approximatel	elow refuel floor d Ly 115% of design f flow balance) du	luct flow cing		
2.	normal operation and a operating with an adequ point and the low flow Fan HVR-FN2A has a flo the 100% \pm 10% normal and flow switches 2HV reduced margin between flow setpoint when thi	flow switches 2H ate margin betwee setpoint when th w of 95% of desi flow balance) du R*FS37A and 37B the normal opera s fan is in servi	VR*FS37A and 37B we on the normal operations fan is in servi- oring flow (i.e., with aring normal operations were operating we ting point and the ice.	vere ting ice. thin tion with low		
The selo 2HV abo flo	low flow setpoint (appro ected based on the norma R-FN2A was placed in ser ve the low flow trip set w (cause unknown) result	ximately 20% less l`flow using fan vice, its operat ting. A small d ed in the low flo	s than normal flow) A 2HVR-FN2B. When ing flow was sligh listurbance of exha ow trip.	was fan ntly aust		
<u> </u>	CAUSE OF EVENT					
A ro Pro of t air the TPM	oot cause investigation w cedure NDP-16.01, "Root (this event was a low air flow signal was caused k operating point. The se -GEN-4003, "Calibration	as performed util Cause Evaluation" flow signal in th by the low flow so tpoint was establ of Fluid Compone	izing Nuclear Divis . The immediate ca le HVR System. The etpoint being close ished by procedure ents Incorporated 1	sion ause low e to N2- Flow		

Therefore, the root cause is a procedural deficiency in that the procedure for calibration of the flow switches did not account for potential flow variations 'between the redundant fans. A contributing cause was excessive. flow variation between fans.

Switches". However, the procedure did not address possible flow

III. ANALYSIS OF EVENT

variations between the two fans.

This event is reportable per 10CFR50.73, part (a)(2)(iv), "Any event or condition that results in manual or automatic actuation of any Engineered Safety Feature (ESF)".

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NRC FORM 366A (6-89)	U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) TEXT CONTINUATION		APPROVED OMB NO. 3150-0104 EXPIRES: 4/30/92 ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS, FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104). OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.		
FACILITY NAME (1)		DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)	
Nine Mile Point	Unit 2	0 5 0 0 0 4 1 0	YEAR SEQUENTIAL REVISION 910 0 2 0 0 0	0 4 OF 0 5	

EXT (If more space is required, use additional NRC Form 366A's) (17)

III. ANALYSIS OF EVENT (cont.)

The Secondary Containment isolation and Reactor Building unit cooler starts were conservative actions, and actuation of these systems have no adverse safety consequences to the general public or plant at any reactor power level. The event in no way adversely affected any other safety system nor the operators' ability to achieve safe plant conditions.

The duration of the event was 17 minutes.

IV. CORRECTIVE ACTIONS

The immediate action was to evaluate the cause of the event. It was determined that the ESF actuation was <u>NOT</u> the result of a LOCA or a Reactor Building high radiation signal, but had in fact been initiated by a Reactor Building exhaust ventilation low flow signal. Subsequent to this determination, the low flow signal was overridden in accordance with N2-OP-52 and the reactor building ventilation was restored to normal operation. Other corrective actions for this event include:

- 1. Procedure N2-IPM-GEN-@003, "Calibration of Fluid Components Incorporated Flow Switches", has been revised to include duplicate flow measurements for each of the alternate fans for the above (2HVR-FN5A and 5B with flow switches 2HVR*FS36A and 36B) and below (2HVR-FN2A and 2B with flow switches 2HVR*FS37A and 37B) refuel floor exhaust fans.
- 2. Flow switches 2HVR*FS36A and 36B setpoints were verified per the revised procedure and found to be calibrated correctly (FS36A and B are the only other application of this type of flow switch having the possibility of flow induced by alternate fans).
- 3. Flow switches 2HVR*FS37A and 37B have been recalibrated to a new lower setpoint based on revised procedure N2-IPM-GEN-@003.
- 4. Other Fluid Component Incorporated flow switches to which this procedure applies have been evaluated for impact and no other applications involving two fans have been found.

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ENSEE EVENT REPORT	LER)	ESTIMATED BURDEN PER RESPONS	5 TO COMPLY WITH THIS		
TEXT CONTINUATION		INFORMATION COLLECTION REQUE COMMENTS REGARDING BURDEN ES AND REPORTS MANAGEMENT BRAN REGULATORY COMMISSION, WASHIN THE PAPERWORK REDUCTION PRO OF MANAGEMENT AND BUDGET, WA	ST: 500 HRS, FORWARD TIMATE TO THE RECORDS CH (P530), U.S. NUCLEAR (GTON, DC 20555, AND TO JECT (3150-0104), OFFICE SHINGTON, DC 20503.		
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ional NRC Form 366A's) (17)					
DNAL_INFORMATION					
Failed Components:	: None.				
B. Previous Similar Events: None. There have been other ESF actuations, however, no previous event associated with the Reactor Building Ventilation System had the same root cause.					
Identification of	components refer	rred to in this L	ER.		
DONENT	IEEE EIIS	803 IEEE FUNCTION SYSTE	805 M ID.		
Building) Hilding Unit Cooler Hilding Ventilation Solution Switch Hilding Ventilation Ans	rs Cl n (HVR) N/ 80 n System F7	/A N LR V /A V AN V AN V	G A H A A A		
	<pre>iit 2 konw/NRC Form 3884%/(IT) NAL INFORMATION Failed Components: Previous Similar : ESF actuations, h with the Reactor B root cause. Identification of PONENT Containment Building) iilding Unit Cooles uilding Ventilation as Treatment (GTS) ch Switch uilding Ventilation ans</pre>	DOCKET NUMBER (2) 1it 2 0 5 0 0 0 4 1 0 Kond MRC Form 3554%/ 1170 DNAL INFORMATION Failed Components: None. Previous Similar Events: None. ESF actuations, however, no prev with the Reactor Building Ventilat: root cause. Identification of components refer IEEE PONENT EIIS Containment Building) 11 and Ventilation (HVR) 12 and Ventilation System ans	OPENAMETER 127 LER NUMBER 107 LER NUMBER 107 LER NUMBER 107 VNAL INFORMATION Failed Components: None. Previous Similar Events: None. There have been ESF actuations, however, no previous event assoc with the Reactor Building Ventilation System had the root cause. Identification of components referred to in this L IEEE 803 IEEE PONENT Containment Building) N/A N/A N Vas Streatment (GTS) Switch N/A V alding Ventilation System Switch N/A V N/A V ans FAN V		

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