



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 29, 1990

Docket No. 50-410

Mr. Lawrence Burkhardt III
Executive Vice President - Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Burkhardt:

SUBJECT: SAFETY EVALUATION OF THE INSERVICE TESTING (IST) PROGRAM FOR PUMPS
AND VALVES, NINE MILE POINT UNIT 2 (TAC NO. 63429)

The enclosed Safety Evaluation (SE) provides the results of the NRC staff's review of the Niagara Mohawk Power Corporation (NMPC) inservice testing program for pumps and valves at Nine Mile Point Unit 2.

The Code of Federal Regulations, paragraph 10 CFR 50.55a(g) requires certain Class 1, 2, and 3 pumps and valves in water-cooled nuclear reactor facilities to meet the inservice testing requirements stated in the ASME Boiler and Pressure Vessel Code, Section XI; specifically, Subsection IWP, Inservice Testing of Pumps in Nuclear Power Plants, and Subsection IWV, Inservice Testing of Valves in Nuclear Power Plants. Each facility is required to establish a program for the inservice testing of pumps and valves which is updated every ten years to meet the requirements in the latest approved edition and addenda to Section XI of the ASME Code. The program is submitted to the NRC for review and approval of the relief requests. The review entails verifying that the program is based on the applicable Code edition and addenda, verifying that the program covers testing of all appropriate pumps and valves, and verifying the acceptability of the requests for relief from the requirements of Subsections IWP and IWV.

The IST program addressed in this report covers the first ten-year interval from October 31, 1986 to October 31, 1996. The NMPC program was submitted in a letter dated March 31, 1988, which superseded all previous IST program submittals. Additional information was submitted by letters dated September 30, 1988 and February 8, 1989.

The NRC staff, with technical assistance from EG&G Idaho, Inc. (EG&G), has reviewed and evaluated the IST program and requests for relief submitted for Nine Mile Point Unit 2. The staff adopts the evaluations and conclusions contained in the enclosed Technical Evaluation Report (TER) as prepared by EG&G. The enclosed safety evaluation (SE) incorporated the TER and its findings. Relief is granted from the testing requirements which have been determined to be impractical to perform, where compliance would result in a hardship without a compensating increase in safety, and where the proposed alternative testing provides an acceptable level of quality and safety.

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Mr. Lawrence Burkhardt

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The NRC staff has determined that the IST program relief requests are acceptable for implementation provided that the items identified in Appendix B of the TER are addressed within the time frame specified in the SE.

The licensee is required to comply with the IST program defined in the letters of March 31, and September 30, 1988, and February 8, 1989, consistent with the relief which has been granted in the enclosed SE. New or revised relief requests may not be implemented without prior approval by NRC, unless they are relief requests meeting the positions in Generic Letter 89-04, Enclosure 1. Program changes that involve additions or deletions of components from the IST program should be provided to the NRC.

The requests for relief comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I. Accordingly, relief from certain provisions of Section XI of the ASME Boiler and Pressure Vessel Code and the applicable addenda is hereby granted, as described in the enclosed Safety Evaluation.

This completes our efforts in response to your submittal as listed above and its associated TAC No. 63429.

Sincerely,



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulations

Enclosures:

1. Safety Evaluation of Relief Requests
2. TER No. EGG-NTA-8974

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ENCLOSURE SIGNED BY:

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulations

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see next page

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