



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 17 TO FACILITY OPERATING LICENSE NO. NPF-69
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR POWER STATION, UNIT NO. 2
DOCKET NO. 50-410

INTRODUCTION

By letter dated April 10, 1990, Niagra Mohawk Power Corporation (the licensee) proposed changes to the Technical Specifications (TS) for Nine Mile Point Unit 2. This application superseded in its entirety an earlier submittal dated November 9, 1989. The proposed changes would modify specifications having cycle-specific parameter limits by replacing the values of those limits with a reference to the Core Operating Limits Report (COLR) for the values of those limits. The proposed changes also include the addition of the COLR to the Definitions section and to the reporting requirements of the Administrative Controls section of the TS. Guidance on the proposed changes was developed by NRC on the basis of the review of a lead-plant proposal submitted on the Oconee plant docket by Duke Power Company. This guidance was provided to all power reactor licensees and applicants by Generic Letter 88-16, dated October 4, 1988.

The proposed revisions in the April 10, 1990 application to Technical Specification (TS) Section 5.3 which describe the fuel assemblies are not included in this amendment. This does not impact the proposed changes to the remaining TS in the interim period until the beginning of operation in the second fuel cycle. The proposed changes to TS section 5.3 will be dealt with as a separate action.

EVALUATION

The licensee's proposed changes to the TS are in accordance with the guidance provided by Generic Letter 88-16 and are addressed below.

- (1) The Definition section of the TS was modified to include a definition of the Core Operating Limits Report that requires cycle/reload-specific parameter limits to be established on a unit-specific basis in accordance with NRC-approved methodologies that maintain the limits of the safety analysis. The definition notes that plant operation within these limits is addressed by individual specifications.

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- (2) The following specifications were revised to replace the values of cycle-specific parameter limits with a reference to the COLR that provides these limits.

(a) Specification 3/4.2.1

The Average Planar Linear Heat Generation Rate (APLHGR) limits for this specification are specified in the COLR.

(b) Specification 3/4.2.2

The Average Power Range Monitor (APRM) flow-biased simulated thermal power upscale scram trip setpoint (S) and flow-biased neutron flux upscale control rod block trip setpoint (S_{RB}) for this specification are specified in the COLR. The definition of the setdown factor T for the surveillance requirement is provided in the COLR.

(c) Specification 3/4.2.3

The Minimum Critical Power Ratio (MCPR) limits and the K_f flow adjustment factors for this specification are specified in the COLR.

(d) Specification 3.2.4

The Linear Heat Generation Rate (LHGR) limits for this specification are specified in the COLR.

(e) Specification 3.3.6

The Control Rod Block Instrumentation Setpoints for the Rod Block Monitor upscale and APRM flow biased neutron flux upscale trip for this specification are specified in the COLR.

These changes to the specifications also required changes to the Bases to include appropriate reference to the COLR. Based on our review, we conclude that the changes to these Bases are acceptable.

- (3) Specification 6.9.1.9 was added to the reporting requirements of the Administrative Controls section of the TS. This specification requires that the COLR be submitted, upon issuance, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector. The report provides the values of cycle-specific parameter limits that are applicable for the current fuel cycle. Furthermore, these specifications require that the values of these limits be established using NRC-approved methodologies and be consistent with all applicable limits of the safety analysis. The approved methodologies are the following:

- (a) "General Electric Company Analytical Model for Loss-of-Coolant Analysis in Accordance with 10 CFR 50, Appendix K," NEDE-20566 (latest approved revision).



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- (b) "General Electric Standard Application for Reactor Fuel,"
NEDE-24011-P-A-9-US (latest approved revision).

Finally, the specification requires that all changes in cycle-specific parameter limits be documented in the COLR before each reload cycle or remaining part of a reload cycle and submitted upon issuance to NRC, prior to operation with the new parameter limits.

On the basis of the review of the above items, the NRC staff concludes that the licensee provided an acceptable response to those items as addressed in the NRC guidance in Generic Letter 88-16 on modifying cycle-specific parameter limits in TS. Because plant operation continues to be limited in accordance with the values of cycle-specific parameter limits that are established using NRC-approved methodologies, the NRC staff concludes that this change is administrative in nature and there is no impact on plant safety as a consequence. Accordingly, the staff finds that the proposed changes are acceptable.

As part of the implementation of Generic Letter 88-16, the staff has also reviewed a sample COLR that was provided by the licensee. On the basis of this review, the staff concludes that the format and content of the sample COLR are acceptable.

In addition, Index page xxiii, Administrative Controls, will be corrected to remove a reference to Figure 6.2.1-1 and Figure 6.2.2-1. The deletion of these two figures for Amendment No. 12 issued on June 30, 1989, was not reflected in the Index when the amendment was processed. These change are administrative in nature and are, therefore, acceptable.

SUMMARY

We have reviewed the request by the Niagra Mohawk Power Company to modify the Technical Specifications of Nine Mile Point Unit 2 that would remove the specific values of some cycle-dependent parameters from the specifications and place the values in a Core Operating Limits Report that would be referenced by the specifications. Based on this review, we conclude that these Technical Specification modifications are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in a requirement with respect to the installation or use of the facility components located within the restricted areas as defined in 10 CFR Part 20, changes to the surveillance requirements and changes to recordkeeping or reporting. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public



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comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9) and 10 CFR Sec 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 19, 1990

PRINCIPAL CONTRIBUTOR:

D. Fieno

