

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

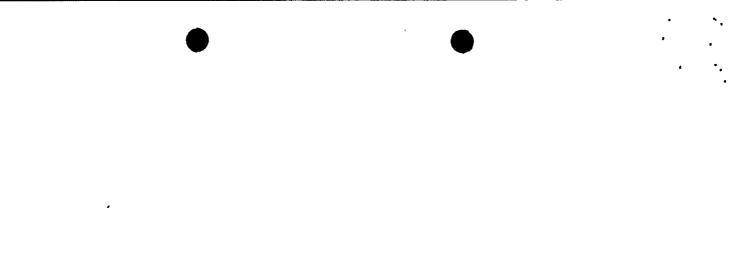
NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 99 License No. DPR-63

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated April 5, 1988, as supplemented April 8, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:





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(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No.99 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, and is to be implemented within 30 days.

FOB THE NUCLEAR REGULATORY COMMISSION Acting for Project Directorate I-1 Division of Reactor Projects, I/II

Attachment: Changes to the Technical Specifications

Date of Issuance: June 9, 1988

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ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

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SECTION	DESCRIPTION		PAGES	
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1.0 DEFINITIONS

1.1 <u>Reactor Operating Conditions</u>

The various reactor operating conditions are defined below. Individual technical specifications amplify these definitions when appropriate.

a. Shutdown Condition - Cold

- (1) The reactor mode switch is in the shutdown position or refuel position. *
- (2) No core alterations leading to an addition of reactivity are being performed.
- (3) Reactor coolant temperature is less than or equal to 212F.

b. Shutdown Condition - Hot

- (1) The reactor mode switch is in the shutdown position. **
- (2) No core alterations leading to an addition of reactivity are being performed.
- (3) Reactor coolant temperature is greater than 212F.

c. <u>Refueling Condition</u>

- (1) The reactor mode switch is in the refuel position.
- (2) The reactor coolant temperature is less than 212F.
- (3) Fuel may be loaded or unloaded.
- (4) No more than one operable control rod may be withdrawn.

d. Power Operating Condition

- (1) Reactor mode switch is in startup or run position.
- (2) Reactor is critical or criticality is possible due to control rod withdrawal.

e. <u>Major Maintenance Condition</u>

- (1) No fuel is in the reactor.
- * The reactor mode switch may be placed in the startup position to perform the shutdown margin demonstration. See Special Test Exception 3.7.1 .
- ** The reactor mode switch may be placed in the refuel position to perform reactor coolant system pressure testing, control rod scram time testing and scram recovery operations.

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LIMITING CONDITIONS FOR OPERATION

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3.7.1 <u>Special Test Exception-</u> <u>Shutdown Margin Demonstrations</u>

Applicability:

Applies to shutdown margin demonstration in the cold shutdown condition.

Objective:

To assure the capability of the control rod system to control core reactivity.

- a. The reactor mode switch may be placed in the startup position to allow more than one control rod to be withdrawn for shutdown margin demonstration, provided that at least the following requirements are satisfied.
 - The source range monitors are operable in the noncoincident condition.
 - (2) The rod worth minimizer is operable per Specification 3.1.1b(3)(b) and is programmed for the shutdown margin demonstration, or conformance with the shutdown margin demonstration procedure is verified by a second licensed operator or other technically qualified member of the unit technical staff.

SURVEILLANCE REQUIREMENTS

4.7.1 <u>Special Test Exception -</u> <u>Shutdown Margin Demonstrations</u>

Applicability:

Applies to periodic inspections required to perform shutdown margin demonstrations in the cold shutdown condition.

Objective:

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- To specify the inspections required to perform the shutdown margin demonstration in the cold shutdown condition.
- a. Within 30 minutes prior to and at least once per 12 hours during the performance of a shutdown margin demonstration, verify that:
 - (1) The source range monitors are operable per Specification 3.5.1.
 - (2) The rod worth minimizer is operable with the required program per Specification 3.1.1b(3)(b) or a second licensed operator or other technically qualified member of the unit technical staff is present and verifies compliance with the shutdown margin demonstration procedure.

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LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

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- (3) The continuous rod withdrawal control shall not be used during out-of-sequence movement of the control rods.
 - (4) No core alterations are in progress.
- b. With the requirements of the above specification not satisfied, immediately place the reactor mode switch in the shutdown or refuel position.

(3) No core alterations are in progress.

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BASES FOR 3.7.1 AND 4.7.1 SHUTDOWN MARGIN DEMONSTRATION

The shutdown margin demonstration has to be performed prior to power operation. However, the mode switch must be placed in the startup position to allow withdrawal of more than one control rod. Specifications 3.7.1 and 4.7.1 require certain restrictions in order to ensure that an inadvertent criticality does not occur while performing the shutdown margin demonstration.

The shutdown margin demonstration will be performed in the cold shutdown condition with the vessel head in place. The shutdown margin demonstration will be performed prior to the reactor coolant system pressure and control rod scram time tests following refueling outages when core alterations are performed. The shutdown margin demonstration is performed using the adjacent rod method.

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