UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 98
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated January 29, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 98 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical Specifications

Date of Issuance: May 23, 1988

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 98 TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

Remove Pages	Insert Pages
92	92
93	93
94	94

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
3.2.6 INSERVICE INSPECTION AND TESTING	4.2.6 INSERVICE INSPECTION AND TESTING
Applicability:	Applicability:
Applies to components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.	Applies to periodic inspection and testing of components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.
Objective:	Objective:
To assure the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.	To verify the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.
Specification:	Specification:
a. Inservice Inspection	a. Inservice Inspection
1. To be considered operable, Quality Group A, B and C components shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2 and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i).	1. Inservice inspection of Quality Group A, B and C components shall be performed in accordance with the requirements for ASME Code Class 1, 2 and 3 components, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10CFR part 50 Section 50.55a(g)(6)(i).

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
	2. An augmented inservice inspection program shall be performed in accordance with the schedules contained in NUREG-0313 Revision 1. The augmented inservice inspection program shall be performed on all nonconforming components. The systems containing nonconforming components shall be identified in the Inservice Inspection Program.
b. Inservice Testing	b. Inservice Testing
1. To be considered operable, Quality Group A, B and C pumps and valves shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2 and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i).	1. Inservice testing of Quality Group A, B and C pumps and valves shall be performed in accordance with the requirements for ASME Code Class 1, 2 and 3 components contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10CFR Part 50 Section 50.55a(g)(6)(i).

The inservice inspection and testing programs[1][2] for the Nine Mile Point Unit 1 plant conform to the requirements of 10CFR50, Section 50.55a(g). Where practical, the inspection of components, pumps and valves classified into NRC Quality Groups A, B and C conforms to the requirements of ASME Code Class 1, 2 and 3 components, pumps and valves, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code. If a Code required inspection is impractical for the Nine Mile Point Unit 1 facility, a request for relief from that requirement is submitted to the Commission in accordance with 10CFR50, Section 50.55a(g)(6)(i).

Request for relief from the requirements of Section XI of the ASME Code and applicable Addenda will be submitted to the Commission prior to the beginning of each 10-year inspection interval if they are known to be required at the time. Requests for relief which are identified during the course of inspection will be submitted quarterly throughout the inspection interval.

The augmented inservice inspection program for the Nine Mile Point Unit 1 plant conforms to the schedules contained in NUREG 0313 Revision 1. It is performed in order to detect and survey intergranular stress corrosion cracking of ASME Code Class 1, 2 and 3 pressure boundary piping. Inspections shall be performed by individuals qualified to: (A) the ASME Boiler and Pressure Vessel Code, Section XI, and (B) Ultrasonic Testing Operator Training for the Detection of Intergranular Stress Corrosion Cracking developed by the EPRI Non-Destructive Examination Center. As an alternate, Niagara Mohawk may use other qualification programs approved by the NRC.

References

(1) Letter from the Nuclear Regulatory Commission (D. B. Vassallo) to Niagara Mohawk Power Corporation (G. K. Rhode), dated September 19, 1983.

(2) Letter from Niagara Mohawk Power Corporation (D. P. Dise) to the Nuclear Regulatory Commission (T. A. Ippolito), dated August 7, 1981.