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AUTH.NAME AUTHOR AFFILIATION MANGAN,C.V. Niagara Mohawk Power Corp.
RECIP. NAME RECIPIENT AFFILIATION
Record Services Branch (Document Control Desk)
SUBJECT: Provides clarification of 861210 request for change to Tech Spec Surveillance 4.6.2.1.c re operator action in response to second high temp alarm.per 861216;request. Immediate reactor trip in response to alarm not required.
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NIAGARA MOHAWK POWER CORPORATION/301 PLAINFIELD ROAD, SYRACUSE, N.Y. 13212/TELEPHONE (315) 474-1511

January 15, 1987 (NMP2L 0972)

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555

> Re: Nine Mile Point Unit 2 <u>Docket No. 50-410</u>

Gentlemen:

8701270307

PDR

- ADOCK

870115

This letter provides clarification of our December 10, 1986 submittal that requested a change to Technical Specification Surveillance 4.6.2.1.c. This clarification was requested by Ms. M. Haughey of the Nuclear Regulatory Staff in a meeting on December 16, 1986.

The evaluation in NUREG 0783 assumes there are two high temperature alarms for the suppression pool with the second alarm set at TS3, which is defined on page 23 as "... the Technical Specification pool temperature limit for reactor scram." This setpoint is equivalent to the 110°F given in action statement 3.6.2.1.b.2.b'on page 3/4 6-16 of the Technical Specifications. To assure the alarm is actuated before exceeding 110°F, allowances are made for environmental effects and the accuracy of components and calibration equipment. This is in accordance with Niagara Mohawk's approved setpoint methodology. The result is a second Technical Specification pool temperature setpoint limit of 105°F, which is in addition to the original setpoint limit of 90°F.

The staff requested clarification of operator action in response to the second high temperature alarm. Immediate reactor trip in response to the alarm is not required because the alarm is for local temperature only. The Limiting Condition for Operation in the Technical Specifications provides the limit for average pool temperature. This information is provided to the operators by the ERF computer. In accordance with plant emergency procedures, operators would monitor average pool temperature whenever it exceeds 90°F and would place the mode switch in shutdown before average suppression pool temperature reaches 110°F. If the computer is unavailable, then the highest operable indication is assumed to be the average and the mode switch would be placed in shutdown before any operable pool temperature indication reaches 110°F. It should be noted that Niagara Mohawk has already committed to provide dedicated average pool temperature indication in the Control Room by the first refueling outage, as indicated in our submittal dated July 24, 1986, Mr. C. Mangan to Ms. E. Adensam, on Detailed Control Room Design Review, Item HED 131.00.

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The staff also requested additional information regarding the physical layout of the suppression pool water temperature instrumentation and the asterisk footnote on page 3/4 6-17 of the Technical Specifications. There are 20 suppression pool water temperature instrument channels with 10 temperature elements in Division I and 10 in Division II. Each of the 10 suppression pool sectors has a pair of temperature elements, one from each division. The pair in each sector has the same setpoint. The upper and lower setpoints alternate between adjacent sectors such that each Division has five upper and five lower alarms.

Very truly yours,

C. V. Mangan Senior Vice President

TF/pns 2378G

xc: Regional Administrator, Region I Ms. E. G. Adensam, Project Director Mr. W. A. Cook, Resident Inspector Project File (2) i. •

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UNITED STATES OF AMERICA NUCLEAR REGULATORY COMMISSION

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In the Matter of Niagara Mohawk Power Corporation (Nine Mile Point Unit 2)

Docket No. 50-410

AFFIDAVIT

<u>C. V. Mangan</u>, being duly sworn, states that he is Senior Vice President of Niagara Mohawk Power Corporation; that he is authorized on the part of said Corporation to sign and file with the Nuclear Regulatory Commission the documents attached hereto; and that all such documents are true and correct to the best of his knowledge, information and belief.

Centary

Subscribed and sworn to					
New York and County of	Onunde	rga	 this	15 th	day
of <u>January</u> ,	1987.	U			

a. Meret Notary Public in and for Omindaga_ County, New York,

My Commission expires: BETH A. MENIKHEIM Notary Public in the State of New York Qualified in Onondaga County No. 4804074 My Commission Expires August 31, 1984

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Docket No. 50-410

January 14, 1987

Mr. C. V. Mangan, Senior Vice President Niagara Mohawk Power Corporation 301 Plainfield Road Syracuse, New York 13212

DISTRIBUTION: Docket No. 50-410 NRC PDR Local PDR BWD-3 r/f RStevens JPartlow EAdensam MHaughey BGrimes EHylton RBernero EJordan AMarinos

Attorney, (OGC) ACRS (10)

Dear Mr. Mangan:

8701270209 870114 PDR ADDCK 05000410

Enclosure:

Subject: Main Steam Isolation Valve Logic Modifications at Nine Mile Point, Unit 2

On December 3, 1986, a full scram occurred at Nine Mile Point Unit 2 (NMP-2) as a result of a loss of power to both reactor protection system (RPS) scram sensor buses. This event was caused by an overcurrent resulting from the crossing of loads in the MSIV logic circuit.

Alerted by this event, the NRC requested NMPC to meet with them on December 18, 1986, to discuss the details of the logic modifications to the main steam isolation valves (MSIVs). After reviewing, the modifications performed on the MSIVs, and the additional proposed changes as a result of the December 3, 1986, event, at this meeting, the staff stated that the MSIV logic scheme was unacceptable and NMPC should propose additional changes. In a conference call on December 22, 1986, the staff further clarified its concern stating that because of the modified MSIV logic design, a fault could propagate back to both buses of the RPS. Therefore, the modified design does not meet the requirements of 10 CFR 50, Appendix A, General Design Criterion 21 and of IEEE Standard 279. At your request, the staff again met with NMPC on January 6, 1987 to discuss the MSIV logic design. After extensive discussion, the staff advised NMPC that it still could not conclude that NMPC's proposed design was acceptable. Additional details relating to the staff's review of the MSIV logic modifications discussed December 19, 1986, and January 6, 1987, are provided in the enclosed safety evaluation.

The issue of the MSIV logic must be resolved before initial criticality. Please contact the License Project Manager, Mary Haughey, at (301) 492-9422, if you have any questions concerning the enclosed safety evaluation or to discuss any proposed alternate MSIV logic arrangements.

Sincerely,

Robert M. Bernero, Director Division of BWR Licensing

As stated See next/page cc: m-Haugher BWD-37DBL 3:DBL **ABL** D:BWD-MHaughey/vag RBernero Glastnas EAdeàsam 87/ بوز/01 01/0 /87 01/(/ /87

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Mr. Troy B. Conner, Jr., Esq. Conner & Wetterhahn Suite 1050 1747 Pennsylvania Avenue, N.W. Washington, D.C. 20006

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Nine Mile Point Nuclear Station Unit 2

Regional Administrator, Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, Pennsylvania 19406

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Mr. Paul D. Eddy
New York State Public Serice
Commission
Nine Mile Point Nuclear Station Unit II
P.O. Box 63
Lycoming, New York 13093

Mr. Richard M. Kessel Chair and Executive Director State Consumer Protection Board 99 Washington Avenue Albany, New York 12210

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