

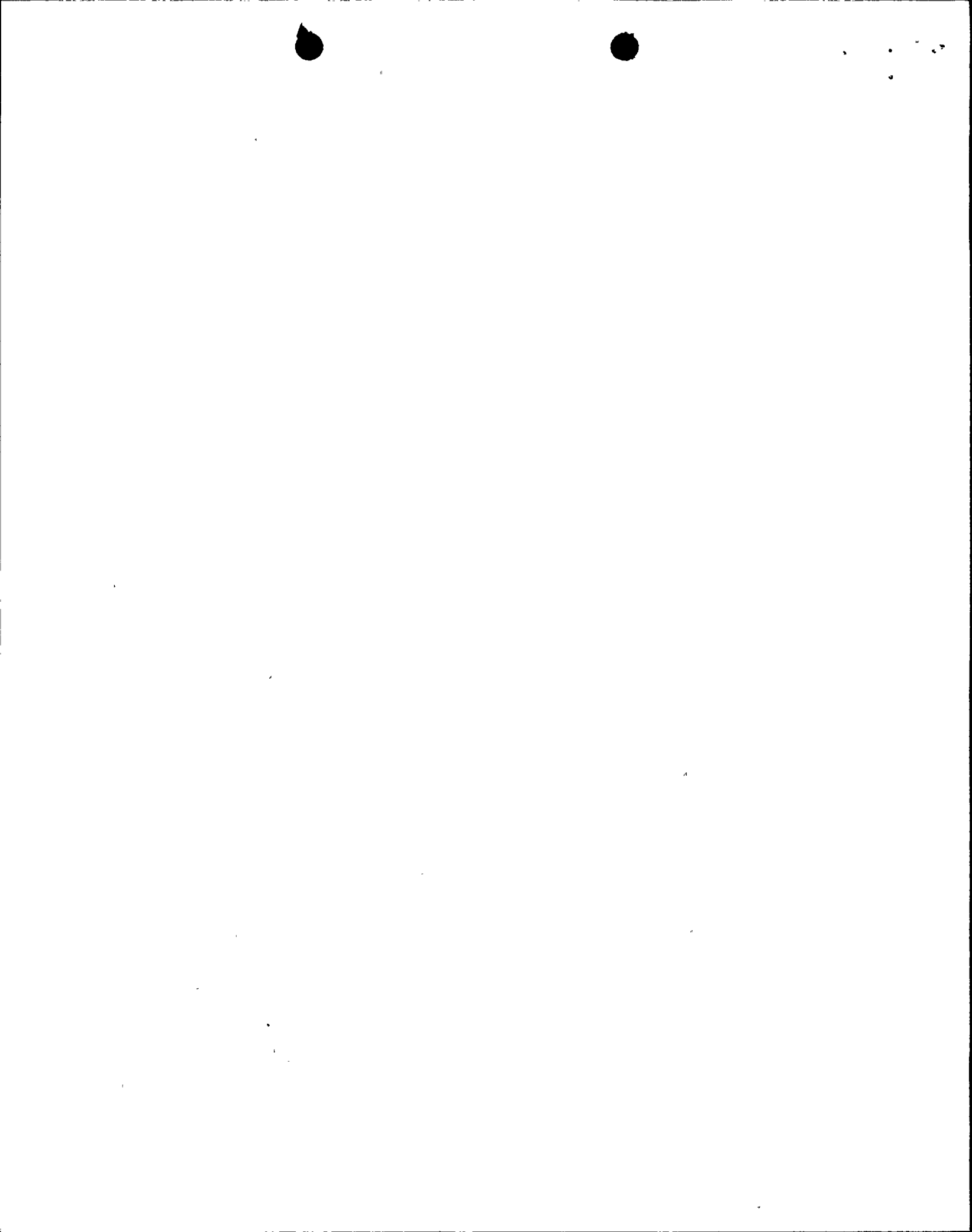
UNITED STATES NUCLEAR REGULATORY COMMISSIONNIAGARA MOHAWK POWER CORPORATIONDOCKET NO. 50-220WITHDRAWAL OF APPLICATION FOR AMENDMENTTO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has granted the request of Niagara Mohawk Power Corporation (the licensee) to withdraw its March 4, 1986 application for amendment to Facility Operating License No. DPR-63 issued to the licensee for operation of the Nine Mile Point Nuclear Station, Unit No. 1 (NMP1) located in Oswego County, New York. Notice of consideration of issuance of this amendment was published in the FEDERAL REGISTER on April 23, 1986 (51 FR 15404).

The request proposed changes to Tables 3.6.2k and 4.6.2k, High Pressure Coolant Injection, of the Appendix A Technical Specifications (TS). The amendment would have placed additional surveillance requirements and limiting conditions for operation on NMP1 due to a modification providing high reactor coolant level tripping of the motor-driven feedwater pumps. Rather than proceed with the proposed amendment, Niagara Mohawk intends to incorporate these requirements into procedures in accordance with TS Section 6.0, Administrative Controls.

By letter dated June 27, 1986, the licensee requested, pursuant to 10 CFR 2.107, permission to withdraw its March 4, 1986 application. The Commission has considered the licensee's request and has determined that permission to withdraw the March 4, 1986 application for amendment should be granted.

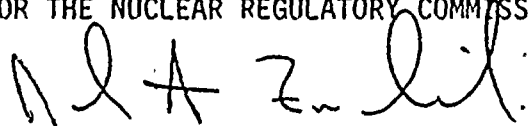
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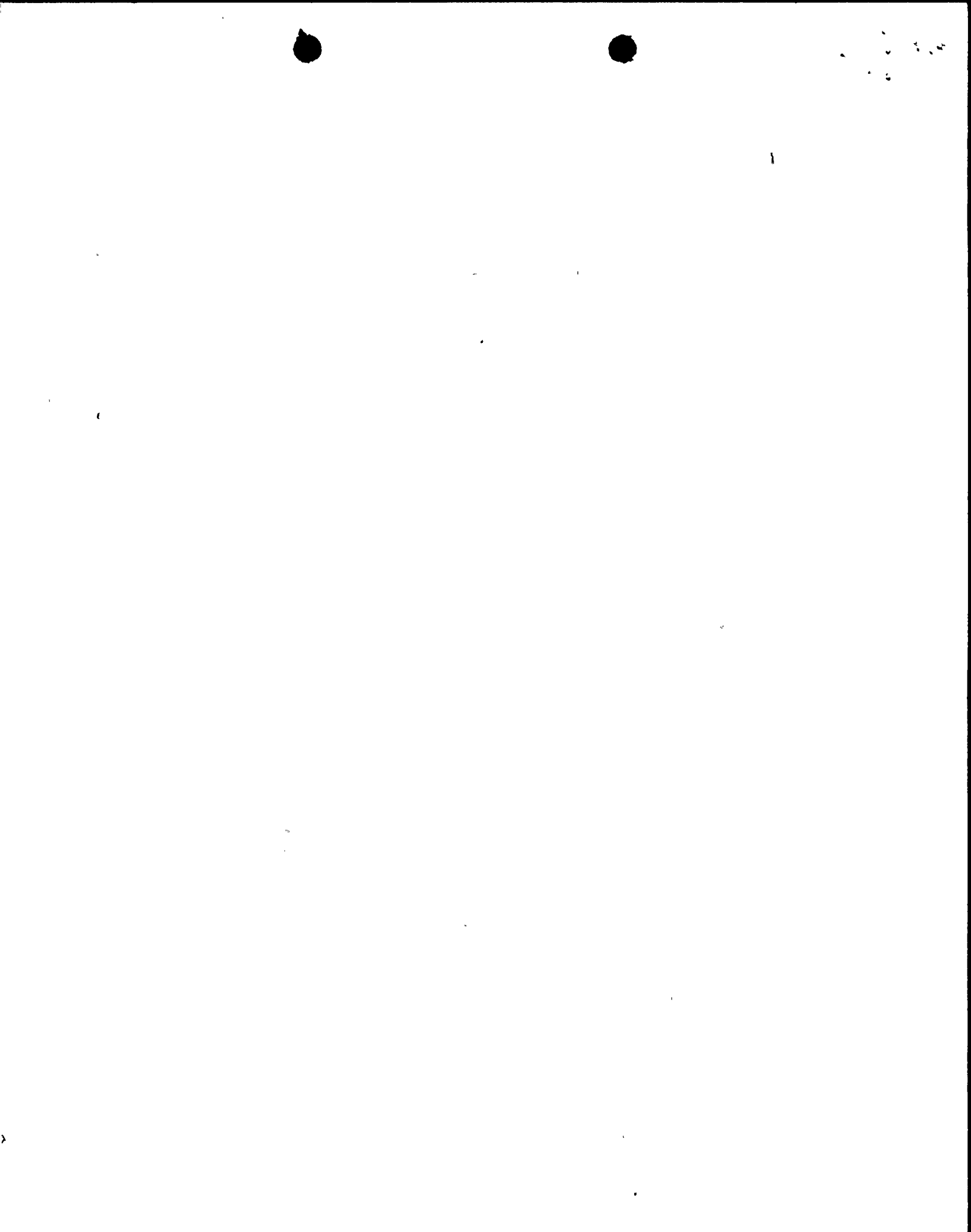
For further details with respect to this action, see (1) the application for amendment dated March 4, 1986, (2) the licensee's request for withdrawal dated June 27, 1986, and (3) our letter dated October 1, 1986. All of the above documents are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, D. C., and at the State University of New York, Penfield Library, Reference and Documents Department, Oswego, New York 13126.

Dated at Bethesda, Maryland this 1 day of October 1986.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Director  
BWR Project Directorate #1  
Division of BWR Licensing



## BASES FOR 3.1.8 AND 4.1.8 HIGH PRESSURE COOLANT INJECTION

The High Pressure Coolant Injection System (HPCI) is provided to ensure adequate core cooling in the unlikely event of small reactor coolant line break. The HPCI System is required for line breaks which exceed the capability of the Control Rod Drive pumps and which are not large enough to allow fast enough depressurization for core spray to be effective.

One set of high pressure coolant injection pumps consists of a condensate pump, a feedwater booster pump and a motor driven feedwater pump. One set of pumps is capable of delivering 3,800 gpm to the reactor vessel at reactor pressure. The performance capability of HPCI alone and in conjunction with other systems to provide adequate core cooling for a spectrum of line breaks is discussed in the Fifth Supplement of the FSAR.

In determining the operability of the HPCI System, the required performance capability of various components shall be considered.

- a. The HPCI System shall be capable of meeting its pump head versus flow curve.
- b. The motor driven feedwater pump shall be capable of automatic initiation upon receipt of either an automatic turbine trip signal or reactor low-water-level signal.
- c. The Condenser hotwell level shall not be less than 57 inches (75,000 gallons).
- d. The Condensate storage tanks inventory shall not be less than 105,000 gallons.
- e. The motor-driven feedwater pump will automatically trip if reactor high water level is sustained for ten seconds and the associated pump downstream flow control valve and low flow control valve are not closed.

During reactor start-up, operation and shutdown, the condensate and feedwater booster pumps are in operation. At reactor pressures up to 450 psig, these pumps are capable of supplying the required 3,800 gpm. Above 450 psig a motor-driven-feedwater pump is necessary to provide the required flow rate.

The capability of the condensate, feedwater booster and motor driven feedwater pumps will be demonstrated by their operation as part of the feedwater supply during normal station operation. Stand-by pumps will be placed in service at least quarterly to supply feedwater during station operation. An automatic system initiation test will be performed at least once per operating cycle. This will involve automatic starting of the motor driven feedwater pumps and flow to the reactor vessel.

