

## LIMITING CONDITION FOR OPERATION

## SURVEILLANCE REQUIREMENT

f. Recirculation Loops

During all operating conditions with irradiated fuel in the reactor vessel, at least two (2) recirculation loop suction valves and their associated discharge valves will be in the full open position except when the reactor vessel is flooded to a level above the main steam nozzles or when the steam separators and dryer are removed.

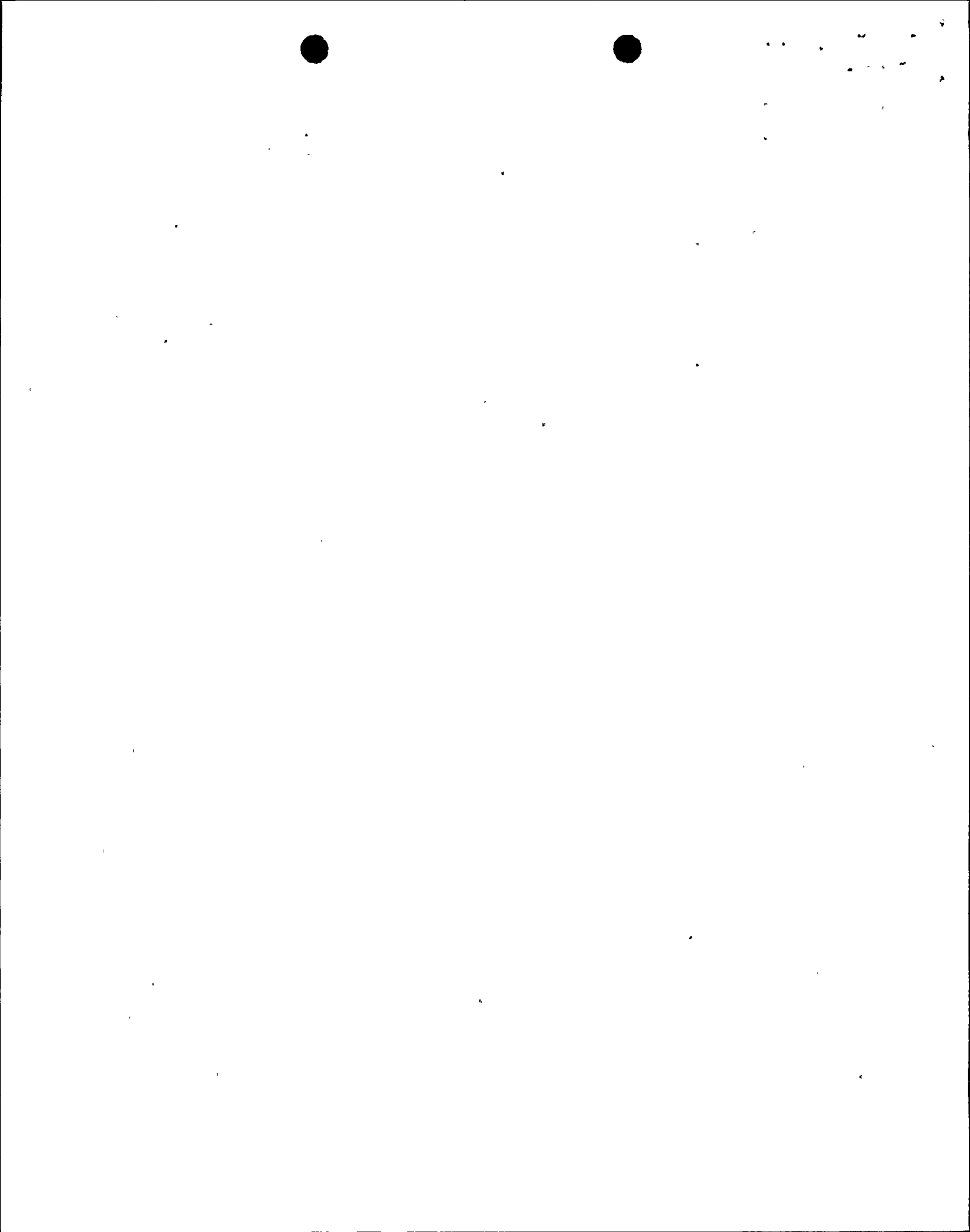
g. Reporting Requirements

If any of the limiting values identified in Specification 3.1.7.a, b, c, d, and e are exceeded, a Reportable Occurrence Report shall be submitted. If the corrective action is taken, as described, a thirty-day written report will meet the requirements of this Specification.

h. Operations Beyond the End-of-Cycle (Coastdown)

For coastdown operations beyond the End-of-Cycle (i.e., when the core reactivity has decreased such that full power cannot be maintained by further control rod withdrawal), steady state thermal power shall be limited to forty (40) percent minimum. Increasing core power level via reduced feedwater heating, once operation in the coastdown mode has begun, is not allowed.

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Reporting Requirements

The LCO's associated with monitoring the fuel rod operating conditions are required to be met at all times, i.e., there is no allowable time in which the plant can knowingly exceed the limiting values of MAPLHGR, LHGR, MCPR, or Power/Flow Ratio. It is a requirement, as stated in Specifications 3.1.7a, b, c, and d that if at any time during power operation, it is determined that the limiting values for MAPLHGR, LHGR, MCPR, or Power/Flow Ratio are exceeded, action is then initiated to restore operation to within the prescribed limits. This action is initiated as soon as normal surveillance indicates that an operating limit has been reached. Each event involving operation beyond a specified limit shall be reported as a Reportable Occurrence. If the specified corrective action described in the LCO's was taken, a thirty-day written report is acceptable.

Operations Beyond the End-of-Cycle (Coastdown)

The General Electric generic BWR analysis of coastdown operation (Reference 14) concludes that operation beyond the end-of-cycle (coastdown) is acceptable. Amendment No. 7 to GESTAR (Reference 15) concludes that the analysis conservatively bounds coastdown operation to forty (40) percent power. The margin to all safety limits analyzed increased linearly as the power decreased.



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REFERENCES FOR BASES 3.1.7 AND 4.1.7 FUEL RODS

- (1) "Fuel Densification Effects on General Electric Boiling Water Reactor Fuel", Supplements 6, 7, and 8, NEDM-10735, August 1973.
- (2) Supplement 1 to Technical Report on Densifications of General Electric Reactor Fuels, December 14, 1974 (USAEC Regulatory Staff).
- (3) Communication: V. A. Moore to I. S. Mitchell, "Modified GE Model for Fuel Densification", Docket 50-321, March 27, 1974.
- (4) "General Electric Boiling Water Reactor Generic Reload Application for 8 x 8 Fuel", NEDO-20360, Supplement 1 to Revision 1, December 1974.
- (5) "General Electric Company Analytical Model for Loss of Coolant Analysis in Accordance with 10CFR50 Appendix K", NEDO-20566.
- (6) General Electric Refill Reflood Calculation (Supplement to SAFE Code Description) transmitted to the USAEC by letter, G. L. Gyorey to Victor Stello, Jr., dated December 20, 1974.
- (7) "Nine Mile Point Nuclear Power Station Unit 1, Load Line Limit Analysis", NEDO-24012.
- (8) Licensing Topical Report General Electric Boiling Water Reactor Generic Reload Fuel Application, NEDE-24011-P-A, August, 1978:
- (9) Final Safety Analysis Report, Nine Mile Point Nuclear Station, Niagara Mohawk Power Corporation, June 1967.
- (10) NRC Safety Evaluation, Amendment No. 24 to DPR-63 contained in a letter from George Lear, NRC, to D. P. Dise dated May 15, 1978.
- (11) "Core Flow Distribution in a General Electric Boiling Water Reactor as Measured in Quad Cities Unit 1", NEDO-10722A.
- (12) Nine Mile Point Nuclear Power Station Unit 1, Extended Load Line Limit Analysis, License Amendment Submittal (Cycle 6), NEDO-24185, April 1979.
- (13) Loss of Coolant Analysis Report for Nine Mile Point Unit One Nuclear Power Station, NEDO-24348, August 1981.
- (14) Communication: R. E. Engel (GE) to T. A. Ippolito (NRC) - "End-of-Cycle Coastdown Analyzed with ODYN/TASC", dated September 1, 1981.
- (15) Amendment No. 7 to GESTAR, NEDE-24011-P-A-7-US, dated August 1985.

