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FACIL: 50-220 Nine Mile Point Nuclear Station, Unit 1, Niagara Powe 05000220
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MANAGAN, C. V. Niagara Mohawk Power Corp.
RECIP. NAME RECIPIENT AFFILIATION
ZWOLINSKI, J. A. BWR Project Directorate 1

SUBJECT: Discusses Spring 1984 & 1986 CRD penetration leakage roll repairs & current operation w/previously rolled CRD penetration leakage. Leak meets leakage criteria. Further actions listed. Meeting planned w/NRC, per 860731 agreement.

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August 1, 1986
NMP1L 0083

Director of Nuclear Reactor Regulation
Attention: Mr. John A. Zwolinski, Project Director
BWR Project Directorate Number 1
Division of BWR Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Dear Mr. Zwolinski:

During the Nine Mile Point Unit 1, Spring 1984 Refueling and Maintenance Outage, leakage from several control rod drive (CRD) penetrations was observed. These penetrations were repaired by roll expanding the CRD housings into the reactor vessel wall. These roll repairs were implemented for the purpose of limiting leakage. The examinations and repairs performed on the stub tubes were reviewed by your staff and documented in a Safety Evaluation, dated June 29, 1984. As discussed in the Safety Evaluation, leakage from the penetrations does not represent a significant safety consideration.

During the Spring 1986 Refueling and Maintenance Outage, one additional CRD penetration was repaired by roll expanding and two previously repaired penetrations were rolled above and below the previously rolled area. The additional rolling was performed on the two penetrations to further limit leakage resulting from joint relaxation during the previous cycle.

As you have been advised, we are presently operating with a CRD penetration which leaked at approximately 4 drops per minute during a 900 psig pressure test. This penetration (46-27) was previously rolled in 1984. While this leak rate meets our leakage criteria, we plan to evaluate and if possible, take further actions to minimize leakage. The specific actions to be considered include the following:

a) Secure cooling water flow to the control rod drive 46-27. Analysis and actual experience shows this will increase contact pressure significantly and thereby minimize leakage.

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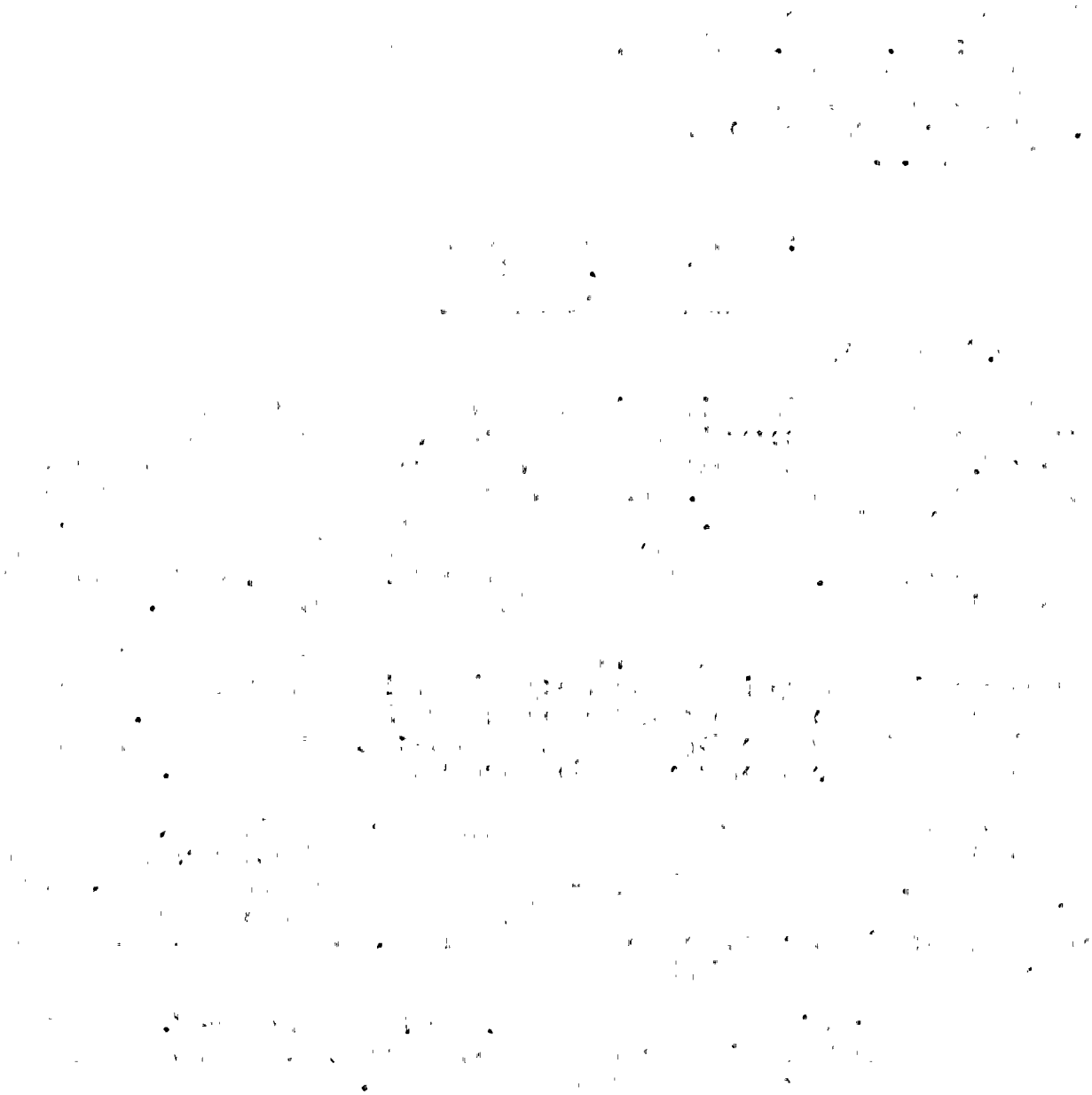

b) Roll above and below the previously rolled area using procedures qualified prior to the 1986 outage. This will be performed during a scheduled outage of sufficient duration (i.e. at least one week) to allow the necessary work activities associated with re-rolling to proceed in an orderly fashion, but not later than the spring 1988 outage.

As we agreed on July 31, 1986, we plan to meet with you in the near future to provide details on the work completed to date and our future plans.

Very truly yours,

cvmangan

C. V. Mangan
Senior Vice President

LK/ar
Attachment
1740J