



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated January 15, 1986, as supplemented March 3, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specification

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 79, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
John A. Zwolinski, Director  
BWR Project Directorate #1  
Division of BWR Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 7, 1986.



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ATTACHMENT TO LICENSE AMENDMENT NO. 79

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

INSERT

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TABLE 3.6.2a

INSTRUMENTATION THAT INITIATES SCRAMLimiting Condition for Operation

<u>Parameter</u>	<u>Minimum No. of Tripped or Operable Trip Systems</u>	<u>Minimum No. of Operable Instrument Channels Per Operable Trip System</u>	<u>Set Point</u>	<u>Reactor Mode Switch Position in Which Function Must Be Operable</u>			
				<u>Shutdown</u>	<u>Refuel</u>	<u>Startup</u>	<u>Run</u>
(1) Manual Scram	2	1			x	x	x
(2) High Reactor Pressure	2	2	1080 psig		x	x	x
(3) High Drywell Pressure	2	2	3.5 psig		x	(a)	(a)
(4) Low Reactor Water Level	2	2(m)	53 inches (Indicator Scale)		x	x	x
(5) High Water Level Scram Discharge Volume	2	2	≤ 45 gal.		(b)	x	x



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NOTES FOR TABLES 3.6.2a and 4.6.2a

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- (a) May be bypassed when necessary for containment inerting.
- (b) May be bypassed in the refuel and shutdown positions of the reactor mode switch with a keylock switch.
- (c) May be bypassed in the refuel and startup positions of the reactor mode switch when reactor pressure is less than 600 psi.
- (d) No more than one of the four IRM inputs to each trip system shall be bypassed.
- (e) No more than two C or D level LPRM inputs to an APRM shall be bypassed and only four LPRM inputs to an APRM shall be bypassed in order for the APRM to be considered operable. No more than one of the four APRM inputs to each trip system shall be bypassed provided that the APRM in the other instrument channel in the same core quadrant is not bypassed. A Travelling In-Core Probe (TIP) chamber may be used as a substitute APRM input if the TIP is positioned in close proximity to the failed LPRM it is replacing.
- (f) Calibrate prior to starting and normal shutdown and thereafter check once per shift and test once per week until no longer required.
- (g) IRM's are bypassed when APRM's are onscale. APRM downscale is bypassed when IRM's are onscale.
- (h) Each of the four isolation valves has two limit switches. Each limit switch provides input to one of two instrument channels in a single trip system.
- (i) May be bypassed when reactor power level is below 45%.
- (j) Trip upon loss of oil pressure to the acceleration relay.
- (k) May be bypassed when placing the reactor mode switch in the SHUTDOWN position and all control rods are fully inserted.
- (l) Only the trip circuit will be calibrated and tested at the frequencies specified in Table 4.6.2a, the primary sensor will be calibrated and tested once per operating cycle.
- (m) One instrument channel in each trip system may be bypassed in the cold shutdown and refuel conditions during the Spring 1986 refueling outage to perform the emergency condenser piping replacement.



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TABLE 3.6.2d

INSTRUMENTATION THAT INITIATES CORE SPRAY(e)Limiting Condition for Operation

<u>Parameter</u>	<u>Minimum No. of Tripped or Operable Trip Systems</u>	<u>Minimum No. of Operable Instrument Channels Per Operable Trip System</u>	<u>Setpoint</u>	<u>Reactor Mode Switch Position in Which Function Must Be Operable</u>			
				<u>Shutdown</u>	<u>Refuel</u>	<u>Startup</u>	<u>Run</u>
<u>START CORE SPRAY PUMPS</u>							
(1) High Drywell Pressure	2	2	$\leq 3.5$ psig	(d)	x	(a)	(a)
(2) Low-Low Reactor Water Level	2	2(f)	$\geq 5$ inches (Indicator Scale)	(b)	x	x	x
<u>OPEN CORE SPRAY DISCHARGE VALVES</u>							
(3) Reactor Pressure and either (1) or (2) above.	2	2	$\geq 365$ psig	x	x	x	x



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NOTES FOR TABLES 3.6.2d AND 4.6.2d

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- (a) May be bypassed when necessary for containment inerting.
- (b) May be bypassed when necessary for performing major maintenance as specified in Specification 2.1.1.e.
- (c) Only the trip circuit will be calibrated and tested at the frequencies specified in Table 4.6.2d, the primary sensor will be calibrated and tested once per operating cycle.
- (d) May be bypassed when necessary for integrated leak rate testing.
- (e) The instrumentation that initiates the Core Spray System is not required to be operable, if there is no fuel in the reactor vessel.
- (f) One instrument channel in each trip system may be bypassed in the cold shutdown and refuel conditions during the Spring 1986 refueling outage to perform the emergency condenser piping replacement.



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