MAR 3 1986

Docket No. 50-410

Niagara Mohawk Power Corporation
ATTN: Mr. B. G. Hooten
Executive Director
Nuclear Operations
c/o Miss Catherine R. Seibert
300 Erie Boulevard, West
Syracuse, New York 13202

Gentlemen:

Subject: Review of Nine Mile Point, Unit 2, Technical Specifications:
Inspection No. 50-410/86-02

This refers to the Nine Mile Point, Unit 2, Technical Specifications review conducted by Parameter Inc. from January 6-17, 1986. During the period of this review, Parameter Inc. was under contract to the NRC to determine whether the draft Technical Specifications (TS) and the Final Safety Analysis Report were in agreement with the plant's as-built configuration and to determine if the draft TS requirements were definitively measurable. The results of this review were discussed with Mr. R. Abbott and other members of your staff on January 17, 1986.

Overall, this review concluded that the Technical Specifications were compatible with the Final Safety Analysis Report, the facility's procedures and the as-built plant configuration. Also, it was determined that there was sufficient management control to ensure adequate completion of the Technical Specification and implementing procedure preparation process. A copy of Parameter Inc.'s report is attached as enclosure 1. We will use the report as a reference in our inspection efforts, however, some findings were identified which require specific followup by Region I. These items are listed in enclosure 2 and will be reviewed in future NRC inspections.

No violations were identified during this review and no reply to this letter is required. Your cooperation with us in this matter is appreciated.

Sincerely,

Original Signed By:

Samuel J. Collins, Chief
Projects Branch No. 2
Division of Reactor Projects

Enclosures:
1. Technical Review Report NRC Region I Inspection No. 50-410/86-02
2. NRC Inspection Follow Items

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cc w/encls:
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ENCLOSURE 2

NRC Inspector Followup Items

Inspection Report No. 50-410/86-02

The following inspector follow items were developed from a review of Inspection Report Number 50-410/86-02, "Review of Nine Mile Point, Unit 2, Technical Specifications" prepared under contract by Parameter, Inc.

1. Preoperational Test Procedure POT-35, "RCIC System Preoperational Test," Revision 0, verifies that the High Steam Flow Isolation Time Delay relay is set at "approximately 3 seconds" instead of the 3-13 second criteria of TS Table 3.3.3-2. The licensee confirmed that the procedure had been written in accordance with the NSSS preoperational test specification and indicated that the time delay relay would be reset, if necessary, in accordance with the TS surveillance requirement. See Section 2.6.3.1 of Inspection Report No. 86-02. Verification that the RCIC High Steam Flow Isolation Time Delay meets the criteria of TS Table 3.3.3-2 will be performed during a future inspection. (86-02-01)

2. Preoperational Test Procedure POT 35, "RCIC System Preoperational Test," Revision 0, requires verifying stroke times of motor operated valves (numbers 2ICS*MOV-121, 128, 148, and 164) which are greater than (less conservative) than the current stroke time values of TS Table 3.6.3-1. The licensee stated that a review of the preoperational test procedure and actual valve performance would be conducted to ensure TS could be met. See Section 2.6.3.2 of Inspection Report No. 86-02. A review of the licensee's actions and verification that the TS required stroke times are met will be performed in a future inspection. (86-02-02)

3. The loading sequence of the Division I and II Emergency Diesel Generators for a Loss of Coolant Accident (LOCA) with a simultaneous Loss of Offsite Power (LOOP) identified in procedure IOP-72, "Standby and Emergency AC Distribution System Interim Operating Procedure," Revision 0, does not agree with the sequence described in FSAR Tables 8.3-1 and 8.3-2. The licensee stated that the correct loading sequence values would be verified and the procedure corrected. See Section 2.7.3.3 of Inspection Report No. 86-02. Verification that the actual time delay relay settings agree with the FSAR values will be performed during a future inspection. (86-02-3)

4. FSAR Section 5.2.2.10 (through Amendment 23) includes extensive commitments to maintain and test ADS system Safety/Relief Valves (SRV's). Currently available and planned licensee procedures and the draft TS do not appear to meet all the FSAR requirements. The inspector was provided with a January 14, 1986 transmittal to NMPC Licensing responding to the above and recommending a change to the FSAR to delete SRV maintenance and test items believed to be unnecessary. See

Section 2.9.3.1 of Inspection Report No. 86-02. Verification that the proposed FSAR amendment is submitted to NRR and/or the licensee's implementation of SRV maintenance and testing activities meets the as-licensed requirements will be performed in a future inspection. (86-02-04)