



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO REQUEST FOR APPROVAL OF REVISION

OF INSERVICE INSPECTION PROGRAM

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

Introduction

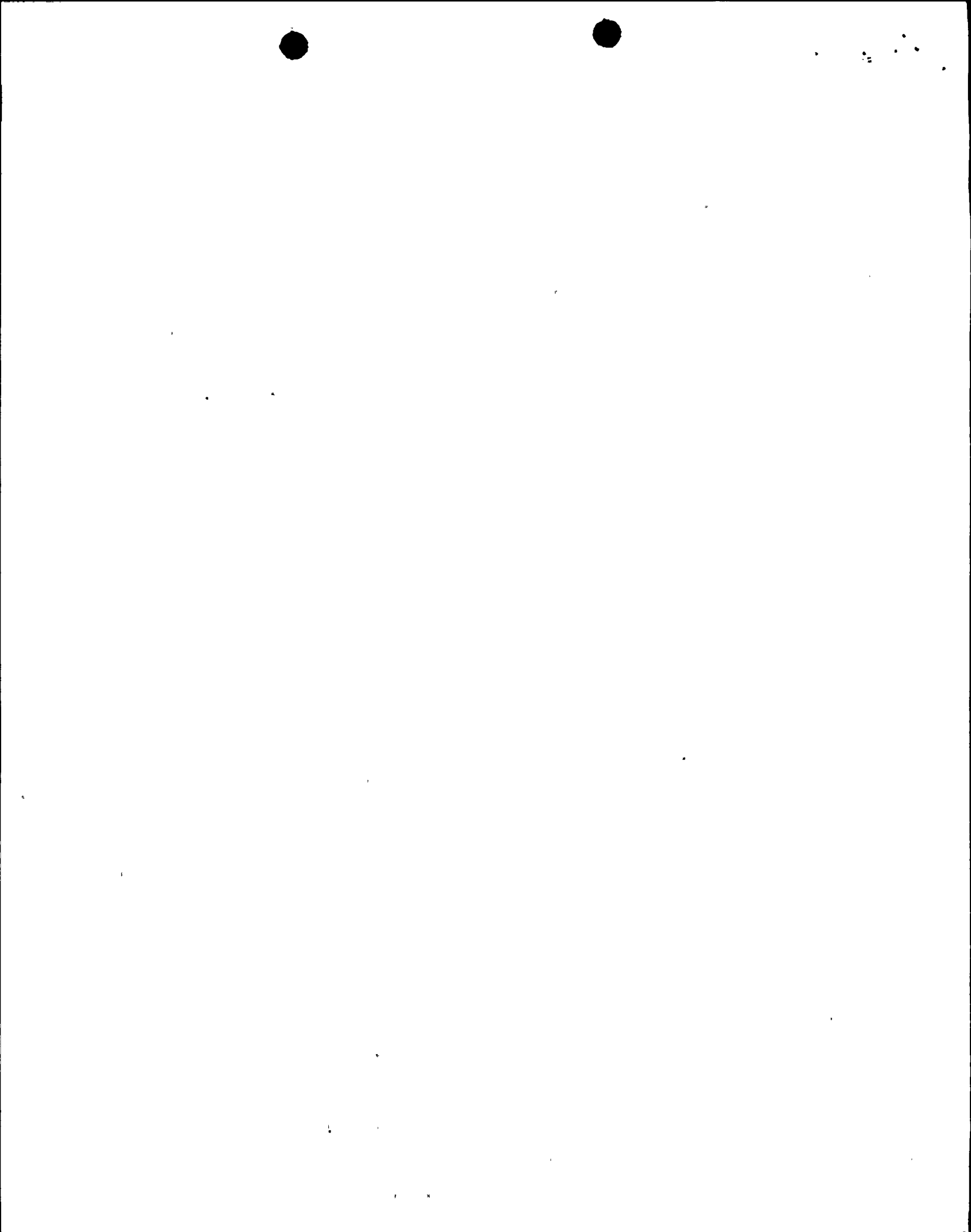
By letters dated March 7, 1984 and November 25, 1985 the Niagara Mohawk Power Company requested a review and approval of revised and updated portions of the Inservice Inspection Program for the Nine Mile Point Unit 1 Nuclear Power Plant. The program revisions have been made in accordance with 10 CFR 50.55a(g)(4)(iv). The regulation allows the use of portions of editions or addenda of the ASME Boiler and Pressure Vessel Code Section XI, provided that all related requirements of the respective editions or addenda are met. Paragraph (b) presently cites the 1980 Edition through Winter 1981 as the latest approved edition and addenda. The revised program incorporated portions of the latest approved edition and addenda. The revisions were necessitated by major modifications to the plant.

Program Revisions

The portions of the 1980 Edition through Winter 1981 Addenda of Section XI to be used in the revised Inservice Inspection Program are as follows;

1. The methods for visual examination shall meet the requirements of Subarticle IWA-2210 (Visual Examinations). The appropriate examination methods of paragraphs IWA-2211 (Visual Examination, VT-1), IWA-2212 (Visual Examination, VT-2), IWA-2213 (Visual Examination, VT-3) and IWA-2214 (Visual Examination, VT-4) will be utilized. All Visual Examination personnel will be qualified to the requirements of the Electric Power Research Institute Nondestructive Examination Center, or an equivalent qualification program approved by the Nuclear Regulatory Commission.

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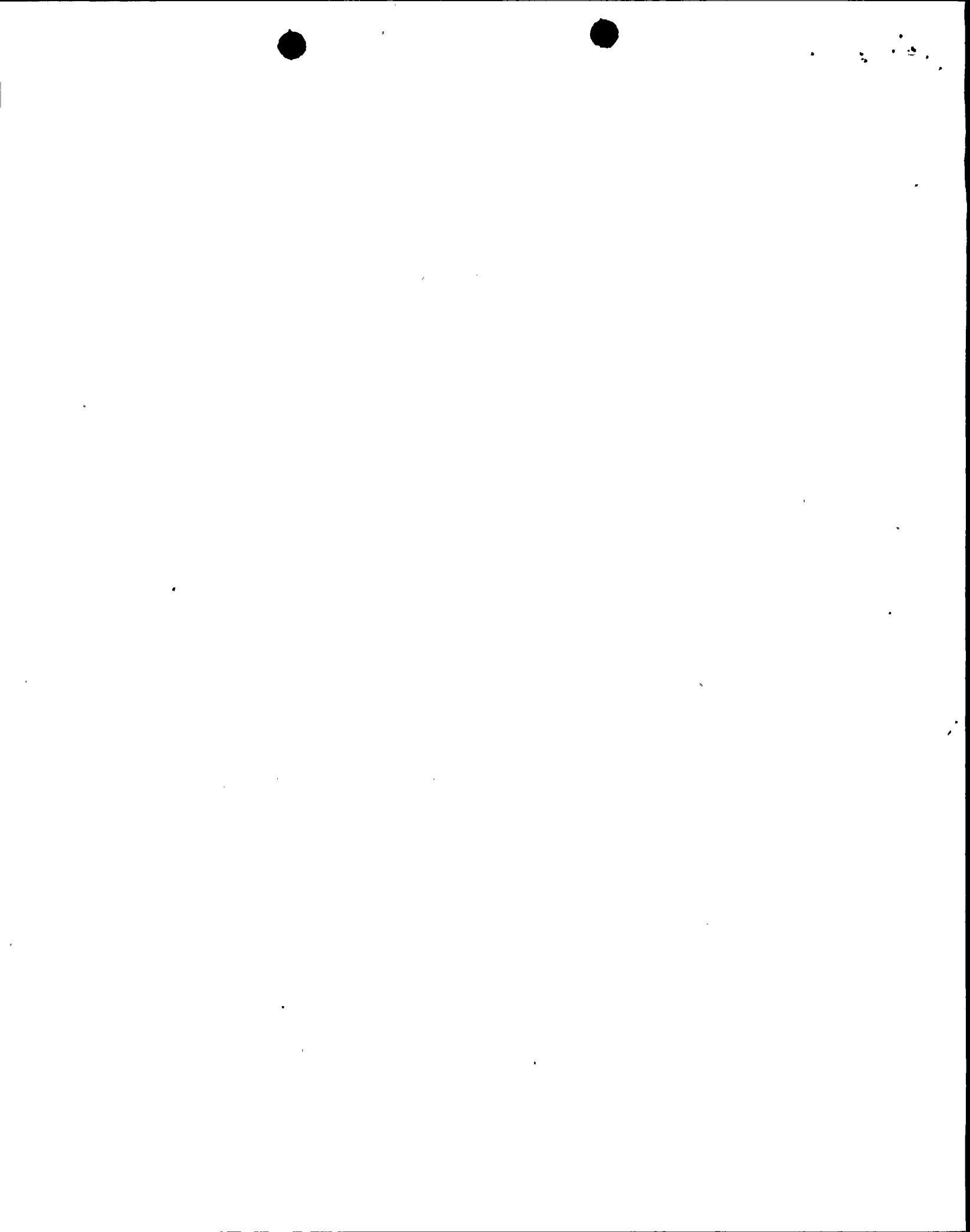


2. Personnel shall be qualified to Article IWA-2000, Subarticle 2300 (Qualification of Nondestructive Examination Personnel).
3. The method for volumetric examinations performed using an ultrasonic examination technique shall meet the requirements of Paragraph IWA-2232 (Ultrasonic Examination), including the use of Appendix III.
4. The examination requirements of Tables IWB-2500-1, IWC-2500-1 and IWD-2500-1 shall be met. However, the selection, extent and frequency of examination for Class 1 and 2 piping welds shall be determined by the 1974 Edition, through Summer 1975 Addenda, of Section XI.
5. Class 2 bolting, Category C-D (Bolting Exceeding Two Inch Diameter) will meet the examination requirements, extent and frequency of IWC-2500-1.
6. Integral attachments, Category B-K-1 (Support Members for Piping, Valves and Pumps) and Category C-C (Integrally Welded Supports) shall meet the examination requirements, extent and frequencies of Tables IWB-2500-1 and IWC-2500-1, respectively.

#### Evaluation

There are no corresponding comparisons to Item 1, 2 and 3 in the 1974 Edition, through Summer 1975 Addenda of Section XI. These items establish personnel qualification requirements and examination requirements not formally incorporated into the Plant Inservice Inspection Program. These items will also fulfill the requirements of the licensee's proposed Technical Specifications of November 4, 1983.

In some cases, the examination requirements of the tables in item 4 above (1980 Edition) include both surface and volumetric examination methods whereas the 1974 Edition required only a volumetric examination.



For Class 2 bolting the 1980 Code concentrates on inspections of bolts two inches in diameter and greater.

For intergral attachments, category B-K-1 and Category C-C, the 1980 Code requires more inspection coverage than the 1974 Code.

The Nuclear Regulatory Commission regulation 10 CFR 50.55a(g)(4)(iv) allows Inservice Inspection programs to use portions of Code editions and addenda provided that all related requirements of the respective editions or addenda are met. The staff considers that the above revisions conform with the regulation.

#### Conclusion

Based upon the above evaluation, the staff concludes that the components and supports to be examined, the extent of the examination and the methods of examination provide reasonable assurance of the structural integrity of the components and supports. The staff further concludes that the revised Inservice Inspection Program conforms to the NRC regulation contained in 10 CFR 50.55a(g)(4)(iv) and, therefore, is acceptable.

Dated: January 6, 1986

