

REGULATOR INFORMATION DISTRIBUTION SYSTEM (RIDS)

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 FACIL: 50-410 Nine Mile Point Nuclear Station, Unit 2, Niagara Mohawk 05000410  
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 MANAGAN, G. V. Niagara Mohawk Power Corp.  
 RECIPIENT NAME: RECIPIENT AFFILIATION  
 BUTLER, W. Licensing Branch 2

SUBJECT: Forwards revised response to FSAR Question 421-26 re reactor protection sys. Encl will be included in FSAR, Amend 20.

DISTRIBUTION CODE: B001D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 12  
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	NRR ROE, M. L.	1 1	NRR/DE/AEAB	1 0
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	NRR/DE/MEB 18	1 1	NRR/DE/MTEB 17	1 1
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	NRR/DHFS/PSRB	1 1	NRR/DL/SSPB	1 0
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May 17, 1985  
(NMP2L 0410)

Mr. Walter Butler, Chief  
Licensing Branch No. 2  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Dear Mr. Butler:

Re: Nine Mile Point Unit 2  
Docket No. 50-410

Enclosed for your use and information is the Nine Mile Point Unit 2 revised response to Nuclear Regulatory Commission's Final Safety Analysis Report question 421.26. This information has been previously requested by your staff and is submitted to aid your review of the Unit 2 license application for the resolution of these questions.

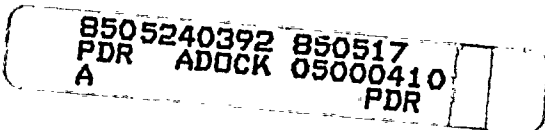
The enclosed will be included in the Final Safety Analysis Report Amendment 20.

Very truly yours,

*C. V. Mangan*

C. V. Mangan  
Vice President  
Nuclear Engineering & Licensing

KK:ja  
Enclosure  
xc: R. A. Gramm, NRC Resident Inspector  
Project File (2)



13001  
1/1



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NINE MILE POINT 2 FSAR

QUESTION

- 421.26 (a) Section 7.3.1.2.2 of the FSAR references Chapter 16 for the minimum number of sensors required to monitor safety-related variables. The final version of Chapter 16 (Technical Specifications) has not yet been submitted. For each monitored variable that provides an input to a safety-related system (i.e., reactor trip, engineered safety features, reactor core isolation cooling, recirc pump trip, control rod block), list the total number of channels provided and the minimum number of channels required to be operable that will be proposed in Chapter 16.
- (7.2)  
(7.3)  
(7.4)  
(7.5)  
(7.6)  
(7.7)
- (b) Confirm that the single failure criterion can be satisfied for each case where the minimum number of operable channel requirement that will be proposed is less than the total number of channels provided.

RESPONSE

- (a) For each monitored variable that provides input to a safety-related system, the total number of channels and the minimum number of channels are shown in Table 421.26.
- (b) NMP2 safety-related systems are designed to conform with the requirements of IEEE Standard 279-1971. Paragraph 4.11 specifically requires that "the system shall be designed to permit any one channel to be maintained, and when required, tested or calibrated during power operation without initiating a protective action at the systems level. During such operation, the active parts of the system shall of themselves continue to meet the single failure criterion."



[The text in this section is extremely faint and illegible. It appears to be a list or a series of entries, possibly organized in columns. Some faint characters and symbols are visible, but they cannot be transcribed accurately.]

NINE MILE POINT UNIT 2 FSAR

TABLE 421.26

REACTOR PROTECTION SYSTEM

<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
1. Intermediate Range Monitors:		
a. Neutron Flux - High	4	3
b. Inoperative	4	3
2. Average Power Range Monitor		
a. Neutron Flux - Upscale, Setdown	3	2
b. Flow Biased Simulated Thermal Power - Upscale	3	2
c. Fixed Neutron Flux - Upscale	3	2
d. Inoperative	3	2
3. Reactor Vessel Steam Dome Pressure - High	2	2
4. Reactor Vessel Water Level - Low, Level 3	2	2
5. Main Steam Line Isolation Valve Closure	4	4
6. Main Steam Line Radiation - High	2	2
7. (Primary Containment) Pressure - High	2	2
8. Scram Discharge Volume Water Level - High		
a. Transmitters/Trip Units	2	2
b. Float Switches	2	2
9. Turbine Stop Valve - Closure	4	4
10. Turbine Control Valve Fast Closure, Valve Trip System Oil Pressure - Low	2	2
11. Reactor Mode Switch Shutdown Position	2	2
12. Manual Scram	2	2



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ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
<b>1. <u>PRIMARY CONTAINMENT ISOLATION</u></b>		
a. Reactor Vessel Water Level		
1) Low, Level 3	2	2
2) Low Low, Level 2	2	2
b. Drywell Pressure - High	2	2
c. Main Steam Line		
1) Reactor Vessel Water Level (Level 1)	2	2
2) Radiation - High	2	2
3) Pressure - Low	2	2
4) Flow - High	2/line	2/line
d. Main Steam Line Tunnel Temperature - High	2	2
e. Main Steam Line Tunnel $\Delta$ Temperature - High	2	2
f. Condenser Vacuum - Low	2	2
g. Drywell and Suppression Chamber Radiation High	2	2
h. Manual Initiation	2	2
i. Turb. Bldg. Steam Leads Enclosure	6	4
<b>2. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u></b>		
* a. $\Delta$ Flow - High	1	1
b. $\Delta$ Flow - Timer	1	1
c. Heat Exchanger/Pump Area Temperature - High	3	3
d. Heat Exchanger/Pump Area Ventilation	3	3
$\Delta$ Temp High		
e. SLCS Initiation	1	1
f. Reactor Vessel Water Level - Low Low, Level 2	2	2
g. Manual Initiation (Isolation)	2	1

\* $\Delta$  flow timer is part of flow circuit. Tech specs incorporate the timer in the list for safety reasons (i.e., to check for drifts).

STATE OF CALIFORNIA

County of \_\_\_\_\_  
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THE BOARD OF SUPERVISORS OF THE COUNTY OF \_\_\_\_\_  
DOES HEREBY GIVE NOTICE THAT IT WILL HOLD A PUBLIC HEARING  
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NOTICE OF PUBLIC HEARING ON THE \_\_\_\_\_  
AT \_\_\_\_\_

<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
3. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>		
* a. RCIC Steam Line Flow - High	2	1
b. RCIC Steam Line Flow -	-	1
c. RCIC Steam Supply Pressure - Low	2	2
d. RCIC Turbine Exhaust Diaphragm Pressure - High	2	2
e. RCIC Equipment Room Temperature - High	1	1
f. RCIC Equipment Room $\Delta$ Temperature - High	1	1
g. RCIC Steam Line Tunnel Temperature - High	1	1
h. RCIC Steam Line Tunnel $\Delta$ Temperature - High	1	1
i. RCIC Steam Drywell Pressure - High	2	2
j. Manual Initiation (Isolation)	2	1
4. <u>RHR SYSTEM STEAM CONDENSING MODE ISOLATION</u>		
** a. RHR/RCIC Flow - High	-	1
** b. Manual Initiation	-	1/valve
5. <u>RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Level 3	2	2
b. Reactor Vessels (RHR Cut-in Permissive) Pressure - High	2	2
c. RHR Equipment Area $\Delta$ Temperature - High	2	2
d. RHR Equipment Area Temperature - High	2	2
e. Manual Initiation	2/valve	2/valve

\*  $\Delta$  flow timer is part of flow circuit. Tech specs incorporate the timer in the list for safety reasons (i.e., to check for drifts).

\*\* Tech specs retain this trip function because there are two measurements along this line that branch off to RCIC; (1) RHR/RCIC high flow - used to monitor the flow in the common RCIC/RHR steam supply line and, (2) RCIC high flow - used to monitor the flow in the RCIC line.



1. The first part of the document discusses the importance of maintaining accurate records.

2. This section describes the various methods used to collect and analyze data.

3. The results of the study are presented in the following table.

Year	Q1	Q2	Q3	Q4
2018	15	20	25	30
2019	18	22	28	32
2020	20	25	30	35
2021	22	28	32	38
2022	25	30	35	40

4. The data shows a clear upward trend in the number of records over the five-year period.

5. This increase is attributed to several factors, including improved data collection methods.

6. The following table provides a more detailed breakdown of the data.

Category	Sub-Category	Value
A	A1	10
	A2	5
B	B1	8
	B2	2
C	C1	6
	C2	4

7. The results indicate that the most significant growth occurred in the 'A' category.

8. This growth is primarily driven by the 'A1' sub-category.

9. The overall trend suggests a strong positive correlation between the variables studied.

10. The study concludes that maintaining accurate records is essential for effective data analysis.

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
<b>A. <u>DIVISION 1 TRIP SYSTEM</u></b>		
1. <u>RHR-A (LPCI MODE) &amp; LPCS SYSTEM</u>		
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	2
b. Drywell Pressure - High	2	2
c. Pressure-Low (LPCS Permissive) Reactor Vessel	1	1
d. Pressure-Low (LPCI Permissive) Reactor Vessel	1	1
e. LPCI Pump A Start Time Delay Relay	1	1
f. Division I Bus Power Monitor	1	1
g. Manual Initiation	1	1
2. <u>AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "A"</u>		
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	2
b. Drywell Pressure - High	2	2
c. ADS Timer	1	1
d. Reactor Vessel Water Level - Low, Level 3 (Permissive)	1	1
e. LPCS Pump Discharge Pressure - High (Permissive)	2	2
f. LPCI Pump A Discharge Pressure - High (Permissive)	2	2
g. Manual Initiation	2/system	2/system
<b>B. <u>DIVISION 2 TRIP SYSTEM</u></b>		
1. <u>RHR B (LPCI MODE)</u>		
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	2
b. Drywell Pressure - High	2	2
c. Reactor Vessel Pressure-Low (LPCI Permissive)	1/valve	1/valve
d. LPCI Pump (B) Start Time Delay Relay	1	1
e. Division II Bus Power Monitor	1	1
f. Manual Initiation	1	1

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<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
<b>B. <u>DIVISION 2 TRIP SYSTEM (Continued)</u></b>		
<b>2. <u>AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B"</u></b>		
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	2
b. Drywell Pressure - High	2	2
c. ADS Timer	1	1
d. Reactor Vessel Water Level - Low, Level 3 (Permissive)	1	1
e. LPCI Pump (B and C) Discharge Pressure - High (Permissive)	2/pump	2/pump
f. Manual Initiation	2/system	2/system
<b>C. <u>DIVISION 3 TRIP SYSTEM</u></b>		
<b>1. <u>HPCS SYSTEM</u></b>		
a. Reactor Vessel Water Level - (Low, Low, Level 2)	4	4
b. Drywell Pressure - High	4	4
c. Reactor Vessel Water Level - High, Level (8)	4	4
d. Condensate Storage Tank Level - Low	2	2
e. Suppression Pool Water Level - High	2	2
f. Division III Bus Power Monitor	1	1
g. Manual Initiation	1	1
<b>D. <u>LOSS OF POWER (DIV. 3 ONLY)</u></b>		
1. 4.16 KV Emergency Bus Undervoltage (Loss of Voltage)	3/Bus	2/Bus
2. 4.16 KV Emergency Bus Undervoltage (Degraded Voltage)	3/Bus	2/Bus



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ATWS RECIRCULATION PUMP TRIP SYSTEM INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
1. Reactor Vessel Water Level - Low Low, Level 2	2	2
2. Reactor Vessel Pressure - High	2	2

STATE OF NEW YORK

IN SENATE  
January 12, 1910.

REPORT  
OF THE  
COMMISSIONERS OF THE LAND OFFICE  
IN ANSWER TO A RESOLUTION PASSED BY THE SENATE  
MAY 11, 1899.

END-OF-CYCLE RECIRCULATION PUMP TRIP SYSTEM INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
1. Turbine Stop Valve - Closure	2	2
2. Turbine Control Valve-Fast Closure	2	2

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REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
a. Reactor Vessel Water Level - (Low Low, Level 2)	4	2
b. Reactor Vessel Water Level - High, Level (8)	4	2
c. Condensate Storage Tank Water Level - Low	2	2
d. Manual Initiation	1	1

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CONTROL ROD BLOCK INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
1. <u>ROD BLOCK MONITOR</u>		
a. Upscale	2	2
b. Inoperative	2	2
c. Downscale	2	2
2. <u>APRM</u>		
a. Flow Biased Neutron Flux Upscale	6	4
b. Inoperative	6	4
c. Downscale	6	4
d. Neutron Flux - Upscale, Startup	6	4
3. <u>SOURCE RANGE MONITORS</u>		
a. Detector not full in	4	3, 2*
b. Upscale	4	3, 2
c. Inoperative	4	3, 2
e. Downscale	4	3, 2
4. <u>INTERMEDIATE RANGE MONITORS</u>		
a. Detector not full in	6	6
b. Upscale	6	6
c. Inoperative	6	6
d. Downscale	6	6
5. <u>SCRAM DISCHARGE VOLUME</u>		
a. Water Level-High	2	2
b. Scram Trip Bypass	2	2
6. <u>REACTOR COOLANT SYSTEM RECIRCULATION FLOW</u>		
a. Upscale	4	2
b. Inoperative	4	2
c. Comparator	4	2

\*MOC/TF of 3 is for startup, 2 is for refueling

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PLANT SYSTEMS ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>NORMAL CHANNELS</u>	<u>MINIMUM CHANNELS</u>
1. <u>FEEDWATER SYSTEM/MAIN TURBINE TRIP SYSTEM</u>		
a. Reactor Vessel Water Level - High, Level 8	3	3

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